NRC Form 366
(9/83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	0 5 0 0 0 3 6 1	1 OF 0 2

TITLE (4)
UNANALYZED CHEMISTRY SAMPLES

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 4	1 6	8 5	8 5	— 0 2 6	— 0 0	0 5	1 6	8 5	SONGS UNIT 3	0 5 0 0 0 3 6 2
										0 5 0 0 0

OPERATING MODE (9)	2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
J. G. HAYNES, STATION MANAGER	7 1 4 — 4 9 2 - 7 7 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 4/16/85, a review of effluent samples revealed two instances in which samples had not been properly counted.

Unit 2 steam generator blowdown was initiated on 3/29/85 with the effluent monitor out of service. One of the Tech. Spec. required daily samples was analyzed approximately 19 hours late.

On 3/29/85, a charcoal iodine composite sample from the Unit 3 Air Ejector Gaseous Monitor was inadvertently discarded. The Tech. Spec. requires the sample be analyzed weekly. The 47 hr., 45 min. of lost data will be estimated.

There was no safety significance to these events because the flow paths were continuously monitored by other instruments, and analysis of other available data confirmed the activity levels were within the expected range.

These instances resulted from oversight by Chemistry Technicians. These events were discussed with all of the Chemistry Technicians to emphasize the need to analyze samples promptly as required, and both individuals received appropriate disciplinary action. The administrative controls have been strengthened to reduce the opportunity for similar errors. The monitor status board in the Chemistry Laboratory has been modified to clearly indicate when samples are required. Additionally, a supervisor will verify the proper sample records have been completed prior to the disposal of composite sample cartridges.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQ. NUMBER	REV. NUMBER	
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	0 5 0 0 0 3 6 1	8 5	- 0 2 6	- 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

A review of effluent sample data conducted on April 16, 1985, revealed two cases in which samples had not been properly counted.

Unit 2 Steam Generator blowdown was initiated at 2020 on March 29, 1985, with Blowdown Neutralization Sump Liquid Effluent Monitor 2RT-7817 (EIIS Component Code RR) removed from service. A sample due to be counted at 2115 on March 30, 1985, in accordance with Technical Specification Table 3-3-12, Action Statement 29, was not analyzed until 1650 on March 31, 1985. This analysis was not timely, considering the 24 hour sampling interval required by Action Statement 29. Unit 2 was operating in Mode 5 and had entered Mode 4 at 0815 on March 29, 1985.

At 0715 on March 29, 1985, the charcoal iodine sample cartridge, which had been collecting a composite sample since 0730 on March 27, was removed from the Unit 3 Air Ejector Monitor 3RT-7870 (EIIS Component Code RR). Technical Specification Table 4.11-2 requires the cumulative iodine sample be analyzed weekly. The Chemistry Technician inadvertently disposed of the sample cartridge before it was counted. The 47 hours and 45 minutes of lost data will be obtained by extrapolating the data from other samples taken before and after the cartridge was discarded. Unit 3 was operating at 100% power and had reduced load to 85% at 2330 on March 28, 1985.

The analysis of other available sample data and data from backup instrumentation confirmed that the effluent activity levels remained relatively constant during the time periods indicated. There was no safety significance to these events because the effluent flow paths were continuously monitored, and the activity levels remained within the expected range.

Both of the above occurrences were discussed with all of the Chemistry Technicians to emphasize the need to analyze samples promptly and the individuals involved received appropriate disciplinary action. Although the analysis requirements were provided to the technicians by existing procedures and shift turnover briefings, these controls have been strengthened to reduce the opportunity for similar errors. The monitor status board in the Chemistry Laboratory has been modified to clearly indicate the status of the effluent monitors and the required sample schedule. In addition, a supervisor will verify the proper sample records have been completed prior to the disposal of composite sample cartridges.

Southern California Edison Company

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STATION MANAGER

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May 16, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 85-026
San Onofre Nuclear Generating Station, Units 2 and 3

Pursuant to 10 CFR 50.73(a)(2)(i), this submittal provides the required 30-day written Licensee Event Report (LER) for chemistry samples which were not analyzed as required. Since these events were common to Units 2 and 3, these events have been combined into a single report in accordance with NUREG-1022. Neither the health and safety of plant personnel nor the public were affected by these events.

If you require any additional information, please so advise.

Sincerely,

J. G. Haynes

Enclosure: LER No. 85-026

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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