NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
OYSTER CREEK, UNIT 1	0 5 0 0 0 2 1 1 9	1 OF 0 3

TITLE (4)

THREE OUT OF FOUR ISOLATION CONDENSER ACTUATION PRESSURE SENSORS OUT OF SPEC

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
											0 5 0 0 0
0 5	2 4	8 5	8 5	–	0 1 1	– 0 0	0 6	2 5	8 5		0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Michael G. Kapil, Senior Engineer	6 0 1 9 9 7 1 1 – 4 8 9 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During routine surveillance testing, isolation condenser automatic actuation pressure sensors RE15A, RE15B and RE15C tripped at values greater than specified in Technical Specifications.

The sensors in question were reset to trip within desired setpoint limits. The event had no effect upon public health or safety.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
OYSTER CREEK, UNIT 1	0 5 0 0 0 2 1 9	8 5	— 0 1 1	— 0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Date of Occurrence

The event occurred on May 24, 1985 at approximately 1400 hours.

Identification of Occurrence

During surveillance testing isolation condenser automatic initiation pressure sensors RE15A, RE15B and RE15C tripped at values greater than that specified in the Technical Specification, section 3.8.

This event is considered to be reportable as defined in 10CFR50.73(a)(2)(B).

Conditions Prior to Occurrence

Mode switch position: RUN
Plant Parameters at the time of occurrence were:
Thermal Power: 1475 MWt
Generator Power: 480 MWe

Description of Occurrence

On Friday, May 24, 1985, at approximately 1400 hours while performing the "isolation condenser automatic actuation sensor calibration and test", the RE15A, RE15B and RE15C trip points were found to be less conservative than that specified in the Technical Specifications. Surveillance testing revealed the following data:

Pressure Switch Designation	Technical Specification Limit (See Note)	As Found (psig)	As Left (psig)
RE 15A	1068 psig	1070	1066
RE 15B	1068 psig	1073	1067
RE 15C	1066 psig	1072	1065
RE 15D	1066 psig	1065	1065

Note: These values are obtained by added respective head correction factor for each sensor to Technical Specification limit of 1060 psig.

Apparent Cause of Occurrence

The cause is attributed to instrument drift. The sensor calibration accuracy is ±7.5 psig, which is comprised of instrument accuracy (0.5% over 0-1200 psig) plus calibrating gauge accuracy (0.1% over 0-1500 psig). Sensors RE15A, RE15B and RE15C were last set at 1067 psig, 1066 psig and 1065 psig respectively. During the scheduled surveillance; RE15A tripped at 1070 psig (corresponds to 1062 psig reactor pressure), RE15B tripped at 1073 psig (corresponds to 1065 psig reactor pressure), and RE15C tripped at 1072 psig (corresponds to 1066 psig reactor pressure). Therefore, the switches were operating within the calibration accuracy of the instruments.

Analysis of Occurrence

The purpose of isolation condensers is to depressurize the reactor, and to remove reactor decay heat in the event that the main condensers are unavailable as a heat sink. Four pressure sensors (RE15A, B, C, and D) are provided to generate a reactor high pressure signal for isolation condenser automatic acutation. A high pressure trip on sensor RE15A or RE15C, plus a high pressure trip on sensor RE15B or RE15D, would send a signal to initiate both isolation condensers.

A review of "as found" sensor settings indicate that isolation condensers actuation would have occurred at reactor pressure of 1062 psig, had a high reactor pressure condition existed.

Safety Significance

The setting of 1060 for isolation condenser automatic actuation on high reactor pressure has been established by Technical Specifications to assure that pressure never reaches the reactor coolant system pressure safety limit (which is 1375 psig). Therefore, the safety significance of this event is considered minimal, since the high pressure signal for isolation condenser actuation would have been generated at 1062 psig, instead of 1060 psig.

Corrective Action

The pressure sensors were reset to trip within the desired setpoint limits. Replacement of these sensors with ones having better setpoint repeatability is scheduled as part of the reactor protection system instrument upgrade during the Cycle 11 refueling outage.

Equipment Data

Manufacturer: Barksdale
Model #B2TA12SS

Proof 1800 PSI
Switch Adjustable Range: 50-1200 PSI

(#0979A)

GPU Nuclear

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609 971-4000
Writer's Direct Dial Number:

June 25, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report

This letter forwards one (1) copy of Licensee Event Report (LER) No. 85-011.

Very truly yours,

Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:BH:dam(0979A)
Enclosures

cc: Dr. Thomas E. Murley, Administrator
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