NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Pilgrim Nuclear Power Station - Unit No. 1	0 5 0 0 0 2 9 3	1 OF 0 2

TITLE (4)
HPCI System Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 5	2 9	8 5	8 5	— 0 1 3	— 0 0	0 6	2 6	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)					
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	X	50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	X	50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
		20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
		20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Gregory G. Belmonte - Plant Engineer	AREA CODE 6 1 7 — 7 4 6 - 7 9 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	B J	D P I S	B 0 8 0	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 5/29/85, at approximately 0015 hrs., during steady operation (100% power), the HPCI system isolated due to a false steam line high flow signal from the differential pressure indicating switch (DPIS) #2353 (Barton Model No. 288A).

The apparent cause of the false high flow signal was due to a shift in setpoints. The setpoint shift could not be duplicated after recalibrating the switch.

Corrective action was to replace the internal components of dpis #2353, due to the reliability of the switch being in question. The HPCI system was declared operable on 5/29/85 at approximately 1904 hrs.

The normal surveillance testing is considered adequate to preclude a recurrence of this event.

Redundant systems that were operable include LPCI, Core Spray, ADS, and RCIC.

This event did not impact the health and safety of the public.

IE22
1/1

NRC Form 366A
(9-83)

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Pilgrim Nuclear Power Station Unit No. 1	0 5 0 0 0 2 9 3	8 5	— 0 1 3	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 5/29/85, at approximately 0015 hrs. during steady operation (100% power), the HPCI system (EIIS Code BJ) isolated due to a false steam line high flow signal from the differential pressure indicating switch (EIIS Code DPIS) #2353 (Barton Model No. 288A).

Immediate corrective action was to initiate a Maintenance investigation and to proceed with surveillance tests required for an inoperable HPCI system.

Cause of the false high flow signal was due to a shift (40" H_2O) in setpoints on both the high and low side of the switch. After equipment qualification work was completed on 5/20/85, dpis 2353 tested satisfactorily. In an attempt to determine the cause of the shift in set points, the switch was recalibrated and tested several times. The set point change could not be duplicated. In addition, the redundant switch (dpis #2352) was repeatedly tested with no set point shift evident.

Corrective action was to replace the internal components of dpis #2353 due to the reliability of the switch being in question. The HPCI system was declared operable on 5/29/85 at approximately 1904 hrs.

The normal monthly surveillance testing is considered adequate to preclude a recurrence of this problem.

Redundant systems that were operable included LPCI, Core Spray, ADS, and RCIC.

A search of our records identified no previous LER's of a similar nature.

This event did not impact the health and safety of the public.

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

June 26, 1985
BECo Ltr. #85-116

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket Number 50-293
License DPR-35

Dear Sir:

The attached Licensee Event Report 85-013-00, "HPCI System Isolation," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,

W D Harrington

W. D. Harrington

GB:caw

Enclosure: LER 85-013-00

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Standard BECo LER Distribution

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