

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT II										DOCKET NUMBER (2) 0 5 0 0 0 3 6 6				PAGE (3) 1 OF 0 8											
TITLE (4) FAILURE OF VALVES TO PASS LOCA, LEAK RATE TEST																									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
0	4	0	9	8	5	8	5	0	1	0	0	1	0	6	2	4	8	5	0	5	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																							
5		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)											
POWER LEVEL (10)		20.406(a)(1)(i)				50.36(c)(1)				X 50.73(a)(2)(v)				73.71(c)											
0 0 0		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.406(a)(1)(iii)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(A)															
		20.406(a)(1)(iv)				X 50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)															
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)															
LICENSEE CONTACT FOR THIS LER (12)																									
NAME										TELEPHONE NUMBER															
Steven B. Tipps, Superintendent of Regulatory Compliance										9 1 2 3 6 7 7 8 5 1															
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS															
X	B/O	IIS/V P13	O15	Y		X	L/F	IIS/V Y	O110	Y															
X	B/B	IIS/V F11	30	Y		X	S/J	IIS/V A	585	Y															
SUPPLEMENTAL REPORT EXPECTED (14)											EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR										
YES (If yes, complete EXPECTED SUBMISSION DATE)											X NO														

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During performance of the "PRIMARY CONTAINMENT PERIODIC TYPE B AND TYPE C LEAKAGE TESTS" procedure (42SV-TET-001-2) as required by Tech. Specs. section 4.6.1.2.d, plant personnel determined that certain valves (see narrative for details) were leaking in excess of the limits specified in Tech. Specs. sections 3.6.1.2.b and c and the ASME section XI criteria specified in the procedure.

The cause of the unacceptable leakage of 2E11-F050B was attributed to contract personnel error due to the fact that the internals of its bypass valve (2E11-F122B) had not been reinstalled after the valve had been welded in place during the recirc pipe outage.

The cause of the unacceptable leakage of all of the other valves reported in this LER has been attributed to normal component wear.

All valves mentioned in this document were repaired and successfully leak rate tested prior to reactor startup which commenced on 05/22/85 at approximately 1340 CDT.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

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TEXT (If more space is required, use additional NRC Form 365A's) (17)

This 30 day LER is required by 10CFR50.73(a)(2)(ii) and 10CFR50.73(a)(2)(v)(c) due to the assumption that the plant did not meet the requirements of Tech. Specs. sections 3.6.1.2.b and 3.6.1.2.c, 10CFR50 appendix J and ASME section XI.

When local leak rate testing was being performed per the "PRIMARY CONTAINMENT PERIODIC TYPE B AND TYPE C LEAKAGE TESTS" procedure (42SV-TET-001-2), the unit was in cold shutdown for a refueling outage.

A. At the time of leak rate testing, plant personnel assumed that the leakage of the following valves was such that the plant could not meet the 0.60 L_a requirement of Tech. Specs. section 3.6.1.2.b.1.

1. On 04/09/85, the "B" RHR Return to Recirculation Isolation Valve (2E11-F015B) was tested.
2. On 04/12/85, the Vent Purge Return Inboard Isolation Valve (2T48-F319) was tested.
3. On 04/15/85, the Primary Containment Service Air inboard isolation valve (2P51-F651) was tested.
4. On 04/16/85, the Torus Purge inlet inboard isolation valve (2T48-F309) was tested.
5. On 04/16/85, the Primary Containment Chilled Water supply outboard isolation valve (2P64-F045) was tested.
6. On 04/16/85, the Primary Containment Chilled Water return outboard isolation valve (2P64-F047) was tested.
7. On 04/27/85, RHR "B" heat exchanger thermal relief valve 2E11-TRV "B" was tested.
8. On 04/30/85, the Hydrogen/Oxygen analyzer inboard drywell sample pressure control valve (2P33-F003) was tested.

B. At the time of leak rate testing, plant personnel assumed that the leakage of the following valves was such that the plant could not meet the 0.009 L_a requirement of Tech. Specs. section 3.6.1.2.b.2.

1. On 04/07/85, the "A" Primary Feedwater outboard isolation valve (2B21-F077A) was tested.
2. On 04/08/85, the "A" & "B" Primary Feedwater inboard isolation valves (2B21-F010A and 2B21-F010B, respectively) were tested.
3. On 04/24/85, the drywell floor drain sump inboard isolation valve (2G11-F003) was tested.
4. On 04/26/85, the drywell equipment drain sump outboard isolation valve (2G11-F020) was tested.

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C. At the time of leak rate testing, plant personnel assumed that the leakage of the following tests was such that the plant could not meet the 11.5 SCFH requirement of Tech. Specs. section 3.6.1.2.C.

1. On 04/06/85, the "C" inboard MSIV (2B21-F022C) and the "C" outboard MSIV (2B21-F028C) were tested.
2. On 04/06/85, the "D" inboard MSIV (2B21-F022D) was tested.
3. On 04/07/85, the "B" inboard MSIV (2B21-F022B) and the "B" outboard MSIV (2B21-F028B) were tested.

D. At the time of testing, plant personnel assumed that the leakage of the following valves was such that the plant could not meet the requirements of the individual valve leakage rates specified as a result of ASME section XI.

1. On 04/07/85, the "A" RHR Testable Check Valve (2E11-F050A) was tested.
2. On 04/10/85, the "B" RHR Testable Check Valve (2E11-F050B) was tested.
3. On 04/11/85, the "B" RHR pump suction inboard Isolation Valve (2E11-F004B) was tested.
4. On 04/13/85, the Core Spray "A" loop reactor vessel isolation check valve (2E21-F006A) was tested.
5. On 04/15/85 the Core Spray "B" loop reactor vessel isolation check valve (2E21-F006B) was tested.

There were no actual safety consequences as a result of these events. The health and safety of the public were not affected.

The cause of the unacceptable leakage of 2E11-F050B was attributed to contract personnel error due to the fact that the internals of its bypass valve (2E11-F122B) had not been reinstalled after the valve had been welded in place during the recirc pipe outage.

The cause of the unacceptable leakage of all of the other valves reported in this LER has been attributed to normal component wear.

All valves mentioned in this document were repaired and successfully leak rate tested prior to reactor startup which commenced on 05/22/85 at approximately 1340 CDT.

There have been similar previous events as last reported on LER 50-366/1984-004. The previous corrective action taken was sufficient at the time; however, valve failures of this type are recurrent.

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IDENTIFICATION OF EACH FAILED COMPONENT

<u>MPL NUMBER</u>	<u>MANUFACTURER</u>	<u>MODEL NUMBER</u>
2B21-F010A	ROCKWELL	FIG. 970NTY
2B21-F010B	ROCKWELL	FIG. 970NTY
2B21-F022B	ROCKWELL	1612 JMNTY
2B21-F022C	ROCKWELL	1612 JMNTY
2B21-F022D	ROCKWELL	1612 JMNTY
2B21-F028B	ROCKWELL	1612 JMNTY
2B21-F028C	ROCKWELL	1612 JMNTY
2B21-F077A	ATWOOD AND MORRIL	DWG# 21378-H
2E11-F004B	WALWORTH	FIG. S206WE
2E11-F015B	WILLIAM POWELL	19023WE
2E11-F122A	FISHER	667-DBQ
2E11-F122B	FISHER	667-DBQ
2E11-TRV-"B"	CROSBY	JO-36, TYPE E
2E21-F006A	ROCKWELL	FIG. 970 JMNTY
2E21-F006B	ROCKWELL	FIG. 970 JMNTY
2G11-F003	PACIFIC	153G-7-WE-CC
2G11-F020	CRANE	47.5-XUF
2P33-F003	FISHER	657-ES
2P51-F651	YARWAY	5515B
2P64-F045	WALWORTH	FIG. 5275
2P64-F047	WALWORTH	FIG. 5275
2T48-F309	FISHER	9220
2T48-F319	FISHER	9220

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TEXT (If more space is required, use additional NRC Form 365A's) (17)

LEAK RATE TEST FAILURE AND REPAIR SUMMARY

<u>Valve Description</u>	<u>As Found Leakage</u>	<u>As Left Leakage</u>	<u>Repair Summary</u>
"A" Primary Feedwater Inboard Isolation Valve (2B21-F010A)	140 ACCM	20 ACCM	Cleaned internals; and replaced ring pin, pressure seals and hinge pin gaskets
"B" Primary Feedwater Inboard Isolation Valve (2B21-F010B)	320 ACCM	0 ACCM	Cleaned internals, machined disc, and replaced gaskets and hinge pins
"B" Inboard MSIV (2B21-F022B)	Would Not Pressurize	9 SCFH	Reground body seats, lapped inner and outer poppets
"C" Inboard MSIV (2B21-F022C)	Would Not Pressurize	5 SCFH	Reground body seats, lapped inner and Outer poppets
"D" Inboard MSIV (2B21-F022D)	Would Not Pressurize	3 SCFH	Reground body seats, lapped inner and outer poppets
"B" Outboard MSIV (2B21-F028B)	Would Not Pressurize	9 SCFH	Cleaned internals, reground body seats, lapped inner and outer poppets
"C" Outboard MSIV (2B21-F028C)	Would Not Pressurize	5 SCFH	Cleaned Internals, Reground Body Seats, Lapped Inner and Outer Poppets
"A" Primary Feedwater Outboard Isolation Valve (2B21-F077A)	Would Not Pressurize	0 ACCM	Valve was not leaking. 2B21-F010A was the problem

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

LEAK RATE TEST FAILURE AND REPAIR SUMMARY

<u>Valve Description</u>	<u>As Found Leakage</u>	<u>As Left Leakage</u>	<u>Repair Summary</u>
"B" RHR Pump Suction Inboard Isolation Valve (2E11-F004B)	Would Not Pressurize	2342 ACCM	Lapped wedge and seats, replaced packing, cleaned internals, replaced bonnet gasket.
"B" RHR Return To Recirculation Isolation Valve (2E11-F015B)	Would Not Pressurize	0 ACCM	Reground seat rings and wedge. Replaced pressure seal.
"A" RHR Testable Check Valve (2E11-F050A)	Would Not Pressurize	0 ACCM	Not a failure. See 2E11-F122A.
"B" RHR Testable Check Valve (2E11-F050B)	Would Not Pressurize	0 ACCM	Not a failure. See 2E11-F122B.
Equalizing Valve For 2E11-F050A (2E11-F122A)	Would Not Pressurize	0 ACCM	Replaced seat ring and Gasket. Lapped Disc.
Equalizing Valve For 2E11-F050B (2E11-F122B)	Would Not Pressurize	0 ACCM	Found seat and cage missing. Replaced seat and cage.
"B" RHR Heat Exchanger Thermal Relief Valve (2E11-TRV "B")	1480 ACCM	0 ACCM	Cleaned and lubricated lifting gear.
"A" Core Spray Isolation Check Valve (2E21-F006A)	5219 ACCM	275 ACCM	Adjusted actuator shaft and tightened packing.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

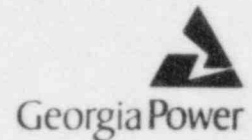
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

LEAK RATE TEST FAILURE AND REPAIR SUMMARY

<u>Valve Description</u>	<u>As Found Leakage</u>	<u>As Left Leakage</u>	<u>Repair Summary</u>
"B" Core Spray Isolation Check Valve (2E21-F006B)	Would Not Pressurize	1300 ACCM	Adjusted actuator shaft and tightened packing.
Drywell Floor Drain Inboard Sump Isolation Valve (2G11-F003)	Would Not Pressurize	60 ACCM	Cleaned valve internals.
Drywell Equipment Drain Sump Outboard Isolation Valve (2G11-F020)	140 ACCM	0 ACCM	Flushed and stroked valve.
Hydrogen - Oxygen Analyzer Inboard Drywell Sample Pressure control valve (2P33-F003)	Would Not Pressurize	60 ACCM	Lapped plug and seat, replaced gaskets.
Primary Containment Service Air Isolation Valve (2P51-F651)	2380 ACCM	20 ACCM	Skim cut disc, lapped seat, and replaced packing.
Primary Containment Chilled Water Supply Outboard Isolation Valve (2P64-F045)	Would Not Pressurize	600 ACCM	Machined disc, lapped seat, and replaced packing.
Primary Containment Chilled Water Return Outboard Isolation Valve (2P64-F047)	Would Not Pressurize	600 ACCM	Machined seat, replaced gaskets and packing.
Torus Purge Inlet Inboard Isolation Valve (2T48-F309)	1720 ACCM	820 ACCM	Cleaned shaft, installed new shaft O-rings.
Vent Purge Return Inboard Isolation Valve (2T48-F319)	Would Not Pressurize	45 ACCM	Replaced shaft O-rings, T-Ring, and 4 flange O-rings.

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Edwin I. Hatch Nuclear Plant

June 24, 1985
GM-85-619

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-010, Rev. 1. This report is required by 10CFR50.73(a)(2)(ii) and 10CFR50.73(a)(2)(v).

H. C. Nix
General Manager

HCN/SBT/vlz

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