

LICENSEE EVENT REPORT (LER)

APPROVED ONS NO. 2186-010N
EXPIRES - 8/31/85

FACILITY NAME (1)

Limerick Generating Station - Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 5 2

PAGE (3)

1 OF 0 3

TITLE (4)

Reactor Water Cleanup System Inboard Valve Isolation

EVENT DATE (5)

LER NUMBER (6)

REPORT DATE (7)

OTHER FACILITIES INVOLVED (8)

MONTH

DAY

YEAR

YEAR

SEQUENTIAL

NUMBER

REVISION

NUMBER

MONTH

DAY

YEAR

FACILITY NAME

DOCKET NUMBER (8)

0 5 0 0 0 0 0 0

0 5 0 0 0 0 0 0

OPERATING

MODE (9)

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

POWER

LEVEL

(10)

0 0 0

20.402 (a)

20.406 (a) (1) (i)

20.406 (a) (1) (ii)

20.406 (a) (1) (iii)

20.406 (a) (1) (iv)

20.406 (a) (1) (v)

20.406 (a) (1) (vi)

20.406 (a)

20.36 (a) (1)

20.36 (a) (2)

20.73 (a) (2) (i)

20.73 (a) (2) (ii)

20.73 (a) (2) (iii)

20.73 (a) (2) (iv)

X

20.73 (a) (2) (v)

20.73 (a) (2) (vi)

20.73 (a) (2) (vii)

20.73 (a) (2) (viii)

20.73 (a) (2) (ix)

20.73 (a) (2) (x)

20.73 (a)

20.73 (a)

OTHER (Specify in Abstract
below and in Title, NRC Form
200-A)

LICENSEE CONTACT FOR THIS LER (12)

NAME

John C. Nagle, Engineer - Special Projects

TELEPHONE NUMBER

AREA CODE

2 1 5 8 4 1 - 5 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED
SUBMISSION
DATE (15)

MONTH

DAY

YEAR

YES (If yes, complete EXPECTED SUBMISSION DATE)

X

NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

Abstract: 85-055

On May 29, 1985 a Reactor Water Cleanup system (RWCU) isolation occurred, with inboard isolation valve HV-44-1F001 closing to the isolation position. The specific cause of the isolation is under investigation. The isolation logic was reset in approximately 10 minutes and the RWCU system was returned to service. There were no adverse effects as a result of the event.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/86

FACILITY NAME (1)

Limerick Generating Station
Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 5 2

LER NUMBER (6)

YEAR

SEQUENTIAL
NUMBERREVISION
NUMBER

8 5

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- 0 0

PAGE (3)

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TEXT (If more space is required, use additional NRC Form 364A (17))

Description of the Event:

On May 29, 1985 at approximately 11:27 a.m., with Unit 1 in cold shutdown, control room alarms indicated an isolation of the Reactor Water Cleanup system (RWCU), and inboard isolation valve HV-44-1F001 closed to the isolation position. The isolation logic was reset in approximately 10 minutes and the RWCU system was returned to normal operation.

The EIIS code designation for the RWCU system is CE.

Consequences of the Event:

The RWCU system isolated properly and reactor water chemistry was not affected due to the short duration of the RWCU isolation.

Cause of the Event:

Surveillance Test, ST-2-25-600-1, "NSSSS-Turbine Enclosure-Main Steam Line Tunnel Temperature-High; Division IA, Channel A Functional Test" (TTS-25-115A, TTS-25-116A, TTS-25-117A, TTS-25-118A, TTS-25-119A) was being performed at the time of the event. RWCU temperature switches and main steam leak detection temperature switches are located on the same panel; however the RWCU isolation logic is not connected with the main steam isolation logic.

As a result of previous similar occurrences, a disturbance analyzer, which records changes of state of contacts, was installed on April 9, 1985 in the inboard and outboard RWCU isolation logics. This analyzer determines which device is generating the isolation signal responsible for the inadvertent isolations. Using the analyzer, the I&C technicians determined that temperature differential transmitter switch TDTS-44-1N602AA (Riley Company) tripped the RWCU isolation. The specific cause of TDTS-44-1N602AA tripping the RWCU isolation is under investigation.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1)

Limerick Generating Station
Unit 1

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

YEAR SEQUENTIAL
NUMBER REVISION
NUMBER

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TEXT (If more space is required, use additional NRC Form 366A 2) (17)

Corrective Actions:

After the RWCU system logic was reset, the steps in ST-2-25-600-1 were repeated, but no further isolation occurred. In addition, I&C personnel inspected panel 10C609 (which contains the temperature switches being tested at the time of the RWCU isolation) for loose wiring that may have been moved during the surveillance test, but none was discovered.

I&C technicians surveillance tested TDTS-44-1N602AA four times in order to determine the cause of the RWCU isolation and found no defects. Because no defects in the switch were identified, repair or replacement of the switch is not believed to be necessary at this time. However, in the event of additional RWCU isolations tripped by TDTS-44-1N602AA, replacement of the switch is being considered.

Previous Occurrences:

LER's 84-035, 85-025, and 85-035.

PHILADELPHIA ELECTRIC COMPANY

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(215) 841-4000

June 24, 1985

Docket No. 50-352

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Washington, DC 20555

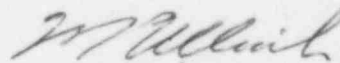
SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER deals with the automatic isolation of the Reactor Water Cleanup system inboard isolation valve.

Reference:	Docket No. 50-352
Report Number:	85-055
Revision Number:	00
Event Date:	May 29, 1985
Report Date:	June 24, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
Resident Site Inspector
See Service List

LE22
11

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