

PPR

OCT 2 2 1975

R. E. Heineman, Director, Division of Technical Review

TECHNICAL ASSISTANCE REQUEST NO. ORB-3-105

Your assistance is requested in the following:

PLANT NAMES: Oyster Creek Nuclear Generating Station, Nine Mile Point Unit 1, Pilgrim 1, Dresden Units 2 and 3, Millstone Unit 1, Quad Cities Units 1 and 2, Monticello, Peach Bottom Units 2 and 3, Browns Ferry Units 1 and 2, Vermont Yankee, Hatch Unit 1, Brunswick Unit 2, Duane Arnold Energy Center, Cooper and Fitzpatrick

DOCKET NOS.: 50-219, 50-220, 50-237, 50-245, 50-249, 50-254, 50-259, 50-260, 50-263, 50-265, 50-271, 50-277, 50-278, 50-293, 50-298, 50-324, 50-331, 50-321, and 50-333

RESPONSIBLE BRANCH: Operating Reactors Branch #3 (ORB-3)

CONTACT: Walter A. Paulson (ext. 7872)

TECHNICAL REVIEW BRANCHES: Containment Systems Branch
Structural Engineering Branch
Mechanical Engineering Branch

TARGET COMPLETION DATE: December 5, 1975

DESCRIPTION OF REQUEST: Background

During testing of the Mark III pressure suppression containment concept, the General Electric Company identified hydrodynamic loads that were not considered in the design of the Mark I containment. In response to our letters of February 15, 1975, and April 17, 1975, identifying our concerns regarding the Mark I containment, the licensees of these plants have initiated a program to demonstrate that the Mark I containment design margins are adequate to withstand the newly identified loads in addition to the loads originally considered in the design. The program consists of two phases. The purpose of the first phase, termed the "Mark I Short Term Program" was to verify whether the Mark I containments can withstand the most probable loads induced by a loss of coolant accident. The second phase, termed the "Mark I Long Term Program", will develop design basis loads based

on the latest pool dynamic information and will assess whether the Mark I containments are capable of sustaining these loads. In addition, loads induced by the actuation of safety/relief valves will be addressed in the long term program.

The short term program has been completed. On October 15, 1975, we received the short term program final report (submitted by GE by letter dated October 6, 1975). This final report consists of the following:

- Volume I - Program Description and Summary of Conclusions, NEDC-20989
- Volume II - LOCA-Related Hydrodynamic Loads, NEDC-20989-2P
- Volume III - Load Application and Screening of Structural Elements, NEDC-20989-3P
- Volume IV - Structural Evaluation, NEDC-20989-4P
- Volume V - Independent Assessment of the Mark I Short-Term Program, NEDC-20989-5

The licensees that have operating plants with Mark I containments have referenced the above report as justification for their conclusion that continued operation during the long term program will not endanger the health and safety of the public. In addition, the licensees have transmitted, by letter, a summary of their proposed long term program which is now scheduled to be completed at the end of the second quarter of 1977.

Request for Assistance

We request that you review the Mark I Short Term Program Final Report and provide a written evaluation of the adequacy of the justification for continued operation of the plants with Mark I containments. Your evaluation should include a determination as to whether continued operation of these plants will not endanger the health and safety of the public.

In addition, we request that you review the proposed long term program and provide a written evaluation of the adequacy of the program.

The above referenced submittals have been distributed to the Technical Review Branches involved.

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

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