

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Pilgrim Nuclear Power Station - Unit No. 1

DOCKET NUMBER (2)

0 5 0 0 0 2 1 9 3 1 OF 0 1 2

TITLE (4) Absolute Versus Gauge Containment
Pressure Transmitters

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)						
0	4	2	4	8	5	8	5	0	1	1	0	5	0	0	0		
0	4	2	4	8	5	0	1	1	0	0	0	5	0	0	0		

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
POWER LEVEL (10) 11010	20.402(b)		20.405(a)		80.73(a)(2)(iv)		73.71(a)			
	20.405(a)(1)(i)		80.38(a)(1)		80.73(a)(2)(v)		73.71(a)			
	20.405(a)(1)(ii)		80.38(a)(2)		80.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 305A)			
	20.405(a)(1)(iii)		80.73(a)(2)(i)		80.73(a)(2)(vii)(A)					
	20.405(a)(1)(iv)		80.73(a)(2)(ii)		80.73(a)(2)(vii)(B)					
	20.405(a)(1)(v)		80.73(a)(2)(iii)		80.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)

NAME Gregory G. Belmonte - Plant Engineer

TELEPHONE NUMBER

AREA CODE

61117 714161-17191010

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A	I/P	I/P D/T	R131710	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) ☒ NO ☐

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 4/24/85, while performing a calibration check of PT 1001-601A & B, questions were raised on the use of absolute pressure transmitters, as opposed to gauge pressure transmitters, to measure containment narrow range pressure for the purpose of post-accident monitoring. Further investigation and analysis identified that two (2) absolute pressure transmitters are installed, as opposed to the required (Ref.: Reg. Guide 1.97, Rev. 3) gauge pressure transmitters.

The cause of the absolute, as opposed to gauge pressure transmitters being installed, was a result of a design change being developed and installed in accordance with Reg. Guide 1.97, Rev. 1, and the proposed Rev. 2, both of which discussed low-range containment pressure monitoring in terms of absolute pressure. Subsequently, Rev. 3 was issued which discussed the low-range monitoring of containment pressure in terms of "gauge" rather than "absolute." This change was not implemented due to an engineering oversight, even though Boston Edison committed to meeting Reg. Guide 1.97, Rev. 3.

The interim corrective action was to recalibrate the transmitters per Temporary Modification 85-27. Planned long-term corrective action is to replace the transmitters (Ref.: PDC 84-17). A review of Reg. Guide 1.97, Rev. 3, was performed with no indication of any further deviations. This is considered to be an isolated event.

This event did not impact the health and safety of the public.

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PDR ADOCK 05000293
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Pilgrim Nuclear Power Station Unit No. 1	DOCKET NUMBER (2) 0 5 0 0 0 2 9 3 8 5 -	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			0 1 1	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On 4/24/85, while performing a calibration check of PT 1001-601A and 601B, questions were raised on the use of absolute pressure transmitters to measure containment narrow range (-5 psig to +5 psig) pressure for the purpose of post-accident monitoring. Further investigation and analysis identified that two (2) absolute pressure transmitters are installed, as opposed to the required (Ref.: Reg. Guide 1.97, Rev. 3), gauge pressure transmitters. These instrument channels serve no actuation function. The concern, therefore, existed that the operator could have been provided with erroneous indications.

With absolute transmitters being installed, the calibration would change with a change in barometric pressure. This could cause the transmitters to be out of calibration and technically inoperable. This condition may have caused the inadvertent violation of Technical Specifications, Table 3.2.F. The reactor power level was approximately 100% at the time of discovery.

The cause of the absolute, as opposed to gauge pressure transmitters being installed, was a result of a design change being developed and installed in accordance with Reg. Guide 1.97, Rev. 1, and the proposed Rev. 2, both of which discussed low-range containment pressure monitoring in terms of absolute pressure. Subsequently, Rev. 3 was issued which discussed the low-range monitoring of containment pressure in terms of "gauge" rather than "absolute." This change was not implemented due to an engineering oversight, even though Boston Edison committed to meeting Reg. Guide 1.97, Rev. 3.

The interim corrective action was to recalibrate the transmitters (to adjust for atmospheric conditions) per Temporary Modification 85-27, which was completed on 4/26/85 at 1800 hrs. Planned long-term corrective action is to replace the transmitters (Ref.: PDC 84-17). A review of Reg. Guide 1.97, Rev. 3, was performed with no indication of any further deviations. This is concluded to be an isolated event.

A search of our records indicates no previous events of a similar nature have occurred.

This event did not impact the health and safety of the public.

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

May 24, 1985
BECO Ltr. #85- 094

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U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

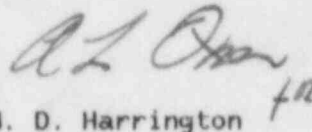
Docket Number 50-293
License DPR-35

Dear Sir:

The attached Licensee Event Report 85-011-00, "Absolute Versus Gauge Containment Pressure Transmitters," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,


W. D. Harrington

GB:caw

Enclosure: LER 85-011-00

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Standard BECO LER Distribution

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