

**Florida  
Power**  
CORPORATION

July 1, 1985  
3F0785-01

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Subject: Crystal River Unit 3  
Docket No. 50-302  
Operating License No. DPR-72  
Final Safety Analysis Report, Revision 6

Dear Mr. Denton:

In compliance with 10CFR 50.71(e), Florida Power Corporation hereby submits one (1) signed original and twelve (12) copies of Revision 6 to the Crystal River Unit 3 Final Safety Analysis Report (FSAR). This document has been updated to reflect changes made to the facility through December 31, 1984. All FSAR figures (e.g., 302 series flow diagrams) have been revised as of April 23, 1985.

Also included is a description and safety evaluation of a facility change made since January 1, 1985 which resulted in a revision to the FSAR.

Sincerely,

G. R. Westafer  
Manager, Nuclear Operations  
Licensing and Fuel Management

GJF/feb

Enclosures

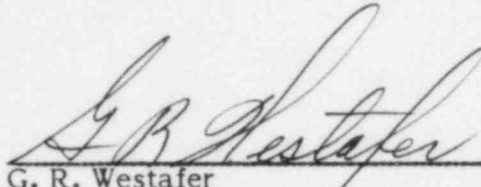
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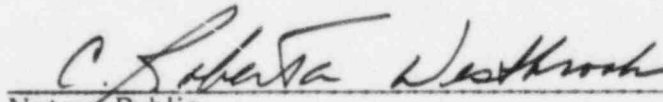
STATE OF FLORIDA

COUNTY OF PINELLAS

G. R. Westafer states that he is the Manager, Nuclear Operations Licensing and Fuel Management for Florida Power Corporation; that he is authorized on the part of said company to sign and file with the Nuclear Regulatory Commission the information attached hereto; and that all such statements made and matters set forth therein are true and correct to the best of his knowledge, information, and belief.

  
\_\_\_\_\_  
G. R. Westafer  
Manager, Nuclear Operations  
Licensing and Fuel Management

Subscribed and sworn to before me, a Notary Public in and for the State and County above named, this 1st day of July, 1985.

  
\_\_\_\_\_  
Notary Public

Notary Public, State of Florida at Large,

My Commission Expires: June 10, 1986

Mar 82-06-23-01, Completed 4/12/85

As a result of the transient of February 26, 1980, at CR-3, it was necessary to reverify the adequacy of specific Nuclear Steam Supply System (NSSS) equipment. No physical modification is required for this MAR.

#### SAFETY EVALUATION

1. Is the probability of an occurrence or the consequence of an accident or malfunction of equipment important to safety as previously evaluated in the Final Safety Analysis Report increased? YES \_\_\_ NO x Because:

This is a documentation change only which does not affect existing equipment materially.

2. Is the possibility for an accident or malfunction of a different type than any previously evaluated in the Final Safety Analysis Report created? YES \_\_\_ NO x Because:

No material or operational parameters are being changed.

3. Is the margin of safety, as defined in the basis for any Technical Specification, reduced? YES \_\_\_ NO x Because:

This is a documentation change only.