

QUAD-CITIES NUCLEAR POWER STATION
UNITS 1 AND 2
MONTHLY PERFORMANCE REPORT
APRIL 1985
COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS & ELECTRIC COMPANY
NRC DOCKET NOS. 50-254 AND 50-255
LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Becky Brown and Dave Kimler, telephone number 309-654-2241, extensions 127 and 192.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

April 1-30: Unit One began the month holding load at full power. On April 6, at 0010 hours, load was dropped to 650 MWe for a Control Rod Pattern adjustment. At 0230 hours the unit began a normal load increase to full power. On April 13, at 0030 hours, load was dropped to 700 MWe for weekly Turbine tests. At 0230 hours load was increased to 760 MWe, and the unit was placed on Economic Generation Control (EGC). At 1955 hours load was dropped to 700 MWe for a Control Rod Pattern adjustment. At 2110 hours the unit began a normal load increase to full power. On April 17, at 0025 hours, load was dropped to 700 MWe for monthly Turbine tests. At 0130 hours load was increased to 740 MWe and the unit was placed on Economic Generation Control (EGC).

B. Unit Two

April 1-30: Unit Two remains shut down for the End of Cycle Seven Refueling and Maintenance Outage for the entire month of April.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY
RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no tests or experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major Safety Related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q34172		Rebuilt Spare CRD HOLD Card #84-241 S/N 6435	NA	NA	Replaced tube & flange Serial No. A4623; Rejected scrap tube
Q40200		Adjusted Pressure Switch on 1-2301- 14 Valve to Make Valve Operate Correctly	Flow switch out of adjustment.	14 Valve will not auto- open when 13 Valve open. 14 Valve would work with Control Switch; HPCI operable.	Adjusted low flow setpoint of flow switch.
Q40201		1-2301-48 Valve	48 Valve would not shut with control switch.	Occurred during testing; HPCI available.	Valve later stroked normally.
Q40203	85-01	HPCI Reset & Trip Pushbuttons	Solenoid coils had broken contacts in SV-8 & SV-12, probably due to Turbine vibration.	Would not trip or reset from Control Room.	Repaired contacts SV-8 and SV-12.
Q40419		1/2-7503A Standby Gas Treatment Heater	Lugs of 2 phases touched in the heater junction box.	The heater breaker tripped while running heater.	Separated the lugs, checked resistance phase-to-phase. All 3 phases satisfactory.
Q40441		1-1001-4B Breaker Tripped with Valve in CLOSED position; did not open	Unknown.		Maintenance checked breaker; operated normally.

UNIT 1 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q40683		1/2 Emergency Diesel Generator to Bus 13-1	Broken snap ring in cubicle linkage.	Breaker auto-trips immediately when closing breaker. Worked O.K. in TEST position; Unit 1 Diesel Generator operable.	Replaced snap ring. Cleaned secondary disconnect switches.
Q40764	85-06	901-10 1A Fuel Pool Radiation Monitor	Surmised to be "electrical noise" inter- ferences.	Failed downscale. 1B monitor verified operable.	Modification 4-1/2- 85-14 installed a capacitor to increase response time constant.
Q40768	85-05	1-1701-16A 1A Fuel Pool Radiation Monitor	Surmised to be "electrical noise" inter- ferences.	Spurious trip.	Modification 4-1/2- 85-14 installed a capacitor to increase response time constant.
Q40769	85-05	1-1701-16B 1B Fuel Pool Radiation Monitor	Surmised to be "electrical noise" inter- ferences.	Spurious trip.	Modification 4-1/2- 85-14 installed a capacitor to increase response time constant.
Q40866	85-12	1-1743A 1A Fuel Pool Radiation Monitor	Surmised to be "electrical noise" inter- ferences.	Spiking high.	Modification 4-1/2- 85-14 installed a capacitor to increase response time constant.
Q41456		1-5745C Middle RHR Service Water Vault Room Cooler Fan	Dirty auxiliary switch in the pump circuit breaker.	Circuit would not close, so the fan would not start. RHR System was operable.	Circuit breaker auxiliary switch was cleaned and breaker returned to service.

UNIT 2 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q40142	85-02	Replaced Torque Switch on Valve 2-2301-8 in Unit Two MSIV Room	The support arm which holds "OPEN" finger contact was broken. Valve travel then limited.	Declared GSEP Unusual Event and began shutting down Unit 2.	Failed torque switch replaced like-for-like. Valve successfully stroked three times.
Q40295		Replaced Solenoid Valve & Tubing on AO 2-1601-20B Valve	Cyclic fatigue deteriorated seals on solenoid valve SO 2-1601-50B	Bad seals prevented air from operating AO 2-1601-20B. 20A Valve was operable.	Solenoid valve replaced like-for-like.
Q40425	85-05	Unit Two MSIV - Isolation Valve Limit Switch Replaced.	Failed limit switch did not stop valve at 10% closed position.	2B MSIV isolation valve went full closed. Increased pressure caused full Reactor scram.	Replaced limit switch like-for-like. 2B MSIV isolation valve functionally tested satisfactorily.
Q40495		2-1001-36B Valve Breaker	Cyclic fatigue of breaker spring. Breaker tripped at high starting current.	Valve breaker tripped with valve in mid-stroke position. All other containment systems available.	Breaker replaced like-for-like, and valve successfully stroked three times.
Q40568		E-Z High Density Fuel Rack Cells A-1 and A-16			Cells A-1 and A-16 blanked off.
Q40639		2-1001-36A Valve Breaker	Auxiliary contacts associated with OPEN position in breaker were binding, preventing proper operation of OPEN contactor.	While testing, the MO 2-1001-36A would not open. 'B' Loop of Containment Cooling available. 36A Valve could have been manually opened if needed.	The auxiliary contacts for both OPEN and CLOSE contactors replaced like-for-like.

UNIT 2 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q41409		2-1001-18A RHR Minimum Flow Bypass Valve	Spring tension out of adjustment on Limitorque operator.	While running valve for Environmental Qualification inspection, valve opened but no light indication. Once full open, valve would not close.	Adjusted spring tension on contacts 7 & 7C and on contacts 8 & 8C.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

UNIT ONE

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-14	4-3-85	1A Fuel Pool Radiation Monitor Spiked High

UNIT TWO

85-10	3-28-85	Reactor Scram Due to Radiography Test and Late NRC Notification
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V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE May 3 1985

COMPLETED BY CAROL KRONICH

TELEPHONE 309-654-2241x193

OPERATING STATUS

0000 040185

1. Reporting period: 2400 043085 Gross hours in reporting period: 719

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>719.0</u>	<u>2879.0</u>	<u>91201.5</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>719.0</u>	<u>2853.7</u>	<u>87889.0</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. Gross thermal energy generated (MWH)	<u>1728856</u>	<u>6717756</u>	<u>182464146</u>
10. Gross electrical energy generated (MWH)	<u>567765</u>	<u>2218890</u>	<u>58996341</u>
11. Net electrical energy generated (MWH)	<u>544301</u>	<u>2126874</u>	<u>55082576</u>
12. Reactor service factor	<u>100.0</u>	<u>100.0</u>	<u>80.2</u>
13. Reactor availability factor	<u>100.0</u>	<u>100.0</u>	<u>83.2</u>
14. Unit service factor	<u>100.0</u>	<u>99.1</u>	<u>77.3</u>
15. Unit availability factor	<u>100.0</u>	<u>99.1</u>	<u>78.1</u>
16. Unit capacity factor (Using MDC)	<u>98.4</u>	<u>96.1</u>	<u>63.0</u>
17. Unit capacity factor (Using Des. MWe)	<u>95.9</u>	<u>93.6</u>	<u>61.4</u>
18. Unit forced outage rate	<u>0.0</u>	<u>.9</u>	<u>5.9</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE May 3 1985

COMPLETED BY CAROL KRONICH

TELEPHONE 309-654-2241x193

OPERATING STATUS

0000 040185

1. Reporting period: 2400 043085 Gross hours in reporting period: 719

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>0.0</u>	<u>1630.8</u>	<u>86536.8</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>0.0</u>	<u>1608.7</u>	<u>83657.8</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated (MWH)	<u>0</u>	<u>3722622</u>	<u>175241689</u>
10. Gross electrical energy generated (MWH)	<u>0</u>	<u>1217469</u>	<u>55870880</u>
11. Net electrical energy generated (MWH)	<u>-1122</u>	<u>1164766</u>	<u>52482751</u>
12. Reactor service factor	<u>0.0</u>	<u>56.6</u>	<u>76.7</u>
13. Reactor availability factor	<u>0.0</u>	<u>56.6</u>	<u>79.4</u>
14. Unit service factor	<u>0.0</u>	<u>55.9</u>	<u>74.2</u>
15. Unit availability factor	<u>0.0</u>	<u>55.9</u>	<u>74.8</u>
16. Unit capacity factor (Using MDC)	<u>-.2</u>	<u>52.6</u>	<u>60.5</u>
17. Unit capacity factor (Using Des. MWe)	<u>-.2</u>	<u>51.3</u>	<u>59.0</u>
18. Unit forced outage rate	<u>0.0</u>	<u>10.6</u>	<u>8.4</u>
19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):			
20. If shutdown at end of report period, estimated date of startup			<u>5-30-85</u>

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE May 3 1985

COMPLETED BY CAROL KRONICH

TELEPHONE 309-654-2241x193

MONTH April 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>793.2</u>
2.	<u>790.6</u>
3.	<u>794.3</u>
4.	<u>805.5</u>
5.	<u>764.1</u>
6.	<u>667.2</u>
7.	<u>780.1</u>
8.	<u>785.5</u>
9.	<u>791.0</u>
10.	<u>800.1</u>
11.	<u>739.3</u>
12.	<u>783.5</u>
13.	<u>745.4</u>
14.	<u>775.0</u>
15.	<u>739.1</u>
16.	<u>736.7</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>733.0</u>
18.	<u>759.5</u>
19.	<u>722.1</u>
20.	<u>725.6</u>
21.	<u>720.7</u>
22.	<u>742.0</u>
23.	<u>766.1</u>
24.	<u>773.5</u>
25.	<u>740.3</u>
26.	<u>748.9</u>
27.	<u>745.3</u>
28.	<u>718.3</u>
29.	<u>750.0</u>
30.	<u>744.1</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE May 3 1985

COMPLETED BY CAROL KRONICH

TELEPHONE 309-654-2241x193

MONTH April 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>-2.6</u>
2.	<u>-2.4</u>
3.	<u>-2.1</u>
4.	<u>-2.1</u>
5.	<u>-1.7</u>
6.	<u>-1.6</u>
7.	<u>-1.5</u>
8.	<u>-1.5</u>
9.	<u>-1.6</u>
10.	<u>-1.5</u>
11.	<u>-1.7</u>
12.	<u>-1.6</u>
13.	<u>-1.7</u>
14.	<u>-1.7</u>
15.	<u>-1.7</u>
16.	<u>-1.8</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>-1.8</u>
18.	<u>-2.2</u>
19.	<u>-2.2</u>
20.	<u>-2.2</u>
21.	<u>-2.3</u>
22.	<u>-2.2</u>
23.	<u>-2.1</u>
24.	<u>-2.0</u>
25.	<u>-2.1</u>
26.	<u>-2.3</u>
27.	<u>-2.2</u>
28.	<u>-2.1</u>
29.	<u>-2.2</u>
30.	<u>-9.8</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 50-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY C Kronich

DATE May 6, 1985

REPORT MONTH APRIL 1985

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
85-23	850406	S	0.0	H	5		RB	CONROD	Reduced load to 650 MWe for Control Rod Pattern Adjustment
85-24	850413	S	0.0	B	5		HA	TURBIN	Reduced load to 700 MWe for weekly Turbine tests
85-25	850413	S	0.0	H	5		RB	CONROD	Reduced load to 700 MWe for Control Rod Pattern Adjustment
85-26	850417	S	0.0	B	5		HA	TURBIN	Reduced load to 700 MWe for monthly Turbine tests

APPROVED
AUG 16 1982

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONSQTP 300-S13
Revision 6
August 1982

DOCKET NO. 50-265

UNIT NAME Quad-Cities Unit 2

COMPLETED BY C Kronich

DATE May 6, 1985

REPORT MONTH APRIL 1985

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
85-20	850317	S	719.0	C	2		RC	FUELXX	Unit remains shutdown for End of Cycle Seven Refueling and Maintenance Outage
APPROVED AUG 16 1982									

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. MAIN STEAM RELIEF VALVE OPERATIONS

There were no Main Steam Relief Valve Operations for the reporting period.

B. CONTROL ROD DRIVE SCRAM TIMING DATA FOR UNITS ONE AND TWO

There was no Control Rod Drive Scram Timing Data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: Q1 Reload: 7 Cycle: 8
2. Scheduled date for next refueling shutdown: 1-2-86
3. Scheduled date for restart following refueling: 4-2-86
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
NOT AS YET DETERMINED.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
DECEMBER 19, 1985; IF LICENSING ACTION REQUIRED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
NONE PLANNED AT PRESENT TIME.
7. The number of fuel assemblies.
a. Number of assemblies in core: 724
b. Number of assemblies in spent fuel pool: 2340
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
a. Licensed storage capacity for spent fuel: 3657
b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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Q. C. O. S. R.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

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1. Unit: Q2 Reload: 7 Cycle: 8
2. Scheduled date for next refueling shutdown: 3-16-85 (Actual)
3. Scheduled date for restart following refueling: 5-30-85
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
Yes. A routine MAPLHGR amendment has been submitted as a preparatory change to allow a 10 CFR 50.59 review.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
Not Applicable.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
None planned at present time.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 176
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3897
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

APPROVED

APR 20 1978

Q. C. O. S. R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCIOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center



Commonwealth Edison

Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

TKT-85-21

May 1, 1985

Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555
Attention: Document Control Desk

Gentlemen:

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of April 1985.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

T. K. Tamlyn
Services Superintendent

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Enclosure