AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE May 6, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

MONTH April, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	484.8	17	484.2
2	482.9	18	483.5
3	484.8	19	482.4
4	484.6	20	482.5
5	484.5	21	481.2
6	484.4	22	480.0
7	485.2	23	480.5
8	485.0	24	483.3
9	485.1	25	485.3
10	485.1	26	486.1
11	485.2	27	486.8
12	484.9	28	487.0
13	484.5	29	486.7
14	484.8	30	485.9
15	485.4	31	-----
16	484.9		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE May 6, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: April, 1985
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	719.0	2,879.0	101,665.0
12. Number Of Hours Reactor Was Critical	719.0	2,852.6	78,132.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	719.0	2,844.7	77,512.1
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,072,284.2	4,187,743.6	98,374,510.6
17. Gross Electrical Energy Generated (MWH)	364,792.0	1,425,986.0	32,195,611.0
18. Net Electrical Energy Generated (MWH)	348,265.9	1,361,277.1	30,772,914.4
19. Unit Service Factor	100.0	98.8	76.2
20. Unit Availability Factor	100.0	98.8	76.2
21. Unit Capacity Factor (Using MDC Net)	101.3	98.9	65.8
22. Unit Capacity Factor (Using DER Net)	101.3	98.9	63.6
23. Unit Forced Outage Rate	0.0	0.0	3.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
1985 Refueling Shutdown is tentatively scheduled for October, 1985 with startup in December, 1985.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE May 6, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

REPORT MONTH April, 1985

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns during the month of April, 1985.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending April, 1985.

1. Scheduled date for next refueling shutdown. — October, 1985

2. Scheduled date for restart following refueling. — December, 1985

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be?

      Technical Specification change to accommodate increased radial peaks due to further reduction in radial leakage.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — _____

   c. If no such review has taken place, when is it scheduled? — _____

4. Scheduled date(s) for submitting proposed licensing action and support information. — September, 1985

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   Methodology Changes — June, 1985

6. The number of fuel assemblies:
   a) in the core — 133 assemblies
   b) in the spent fuel pool — 305 "
   c) spent fuel pool storage capacity — 729 "
   d) planned spent fuel pool storage capacity — May be increased via fuel pin consolidation "

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1996

Prepared by J K Gasper — Date May 1, 1985

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

April, 1985
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power throughout April, 1985. The Technical group continued to conduct a program to identify vacuum leaks on the Fort Calhoun condenser. This program was successful, indicating the primary source of leakage was the dogbone seal which has been coated to reduce condenser inleakage.

Two new auxiliary operators-nuclear were hired in April. Annual NRC licensed operator requalification simulator training at Combustion Engineering in Windsor, Connecticut, began in April.

Hot license training for two engineers and four operators is in progress. The first AON training group using lesson plans are permanently on shift. Training of the initial AON class and the current hot license class continued on schedule. Accreditation efforts continue with emphasis on the C/RP training area.

The NRC conducted a security audit during April which resulted in the potential close out of many long standing areas of concern.

No safety valve or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number	Description
85-001	VIAS Actuation (RM-060).
85-002	VIAS Actuation (RM-061).
85-003	RM-061 VIAS Actuations.

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-FAUD-1	Fuel Assembly Uplift Condition Detection.
	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.
SP-LIOH-1	Lithium Hydroxide Moisture Carryover Test.
	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because the concentration of the lithium in the steam generators was lower than the concentration limit of sodium which is more damaging to the secondary system than lithium.

System Acceptance Committee Package for April, 1985:

Package	Description/Analysis
EEAR FC-84-116	HCV-1930 Replacement.
	This modification replaced the screenwash strainer backwash valve, HCV-1930. The Jamesbury 2" ball valve was replaced by a 2" Red Valve cylinder actuated "pinch" valve. This modification did not effect safety related equipment; therefore, has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

None

F. CHANGES IN PLANT OPERATING STAFF

During April, Mr. Matt Anielak and Mr. Doug Foreman reported to the Fort Calhoun Station as Auxiliary Operators-Nuclear. Mr. Timothy J. McIvor replaced Mr. Alan W. Richard as Supervisor-Technical.

G. TRAINING

During April, C/RP, Maintenance, Auxiliary Operator and Hot License training continued. Three requalification groups attended simulator training at Combustion Engineering. INPO made a two-day training assistance visit. Operator job and task analysis continued.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Package	Description
Amendment No. 86	This amendment revises the Technical Specifications to incorporate the requirements of Appendix I of 10 CFR Part 50 as the Radiological Effluent Technical Specifications (RETS).
Amendment No. 87	This amendment adds new Technical Specifications addressing the operability and surveillance requirements for the new Toxic Gas Monitoring System. The amendment also makes an administrative change such that there are not two tables in the Technical Specifications numbered 2-9.

II. MAINTENANCE (Significant Safety Related)

None

W. Gary Gates

W. Gary Gates
Manager
Fort Calhoun Station