



Commonwealth Edison
LaSalle County Nuclear Station
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Marseilles, Illinois 61341
Telephone 815/357-6761

January 11, 1993

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for December 1992.

Very truly yours,

for Gary F. Spedl
Station Manager
LaSalle County Station

GFS/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

DECEMBER 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor sub-critical, Generator off-line, refuel outage (L1R05) in progress.
31	2400	Reactor sub-critical, Generator off-line, refuel outage (L1R05) in progress.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

Technical Specification Amendment to remove the rod sequence control system and lower the rod worth minimizer setpoint to 10%.

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SGR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-015-00	12/01/92	Loss of Bus 141Y due to personnel error.
92-016-00	12/03/92	Unplanned ESF actuation during reactor protection system bus transfer due to personnel error.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L01685	Low Pressure Core Spray Testable Check Valve 1E21-F006	Valve packing leakage.	None.	Adjusted valve actuator packing.
L06243	Reactor Building Ventilation Isola- tion Check Damper 1VR010Y	Damper closing springs.	None.	Replaced damper springs.
L07282	250 VDC Battery 1DC01E	Battery cell #61.	Cell failed individual cell charge.	Replaced cell.
L09602	Main Steam Line High Radiation Scram And Isolation.	HFA relay.	None.	Replaced relay.
L13046	Main Steam Line Low Pressure Switch Stop Valve	Valve leakage.	None.	Replaced valve.
L13740	Low Power Range Monitor 24-09C	LPRM Card	Spurious upscale alarms.	Replaced card.
L15254	Residual Heat Removal Heat Exchanger Outlet Valve 1E12-F068A	Valve packing leakage.	None.	Replaced packing.
L15387	Low Pressure Core Spray Full Flow Bypass Valve 1E21-F012	Valve packing leakage.	None.	Repacked valve.

(No SOR failures this month.)

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MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L15826	Residual Heat Removal Pump Seal Cooler Inlet Valve 1E12-F333C	Valve leakage.	Unable to isolate seal cooler.	Replace valve disc and stem.
L18241	Main Steam Isolation Valve Air Accumulator Check Valve 1B21-F029B	Valve leakage.	None.	Replaced valve disc.
L18288	Main Steam Isolation Valve Air Control Valves 1B21-F022B	Air leakage.	None.	Replaced air control valves.
L18329	Main Steam Isolation Valve Accumulator Check Valve 1B21-F024D.	Valve leakage.	Failed pressure drop test.	Replaced valve o-ring.
L18736	Control Rod Drive Hydraulic Control Unit 02-31 Isolation Valve	Valve leakage.	None.	Replaced valve disc.
L18928	Division III Switch- gear Room Ventilation	Temperature controller.	Low room temperature.	Replaced controller.
L18991	Low Pressure Core Spray Suction Drain Valve 1E21-F331	Broken Yoke Bushing.	Unable to close valve.	Replaced bushing.
L19102	Reactor Protection Shutdown Scram Reset Interlock.	Relay.	None.	Replaced relay.

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L19107	Control Rod Drive Hydraulic Control Unit 18-47	Nitrogen charging valve leakage.	Unable to charge accumulator.	Replaced valve cartridge.
L19171	Standby Gas Treatment System Cooling Fan	Cooling fan breaker	Cooling fan failed to auto start.	Replaced breaker.
L19220	Circuit Breaker 1412	Control switch.	Circuit breaker failed to open on under voltage condition.	Replaced control switch.
L19241	Auxiliary Electric Equipment Room Compressor OVE04CA	Temperature switch.	Switch tripping below trip setpoint.	Replaced switch.
L19244	Standby Liquid Control Pump Pump 1C41-C001B	Pump packing.	Packing wall temperatures exceeded limit of 130 degrees fahrenheit.	Repacked pump.
L19279	Control Rod Drive Hydraulic Control Unit 38-15	Accumulator level switch.	None.	Replaced level switch.

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TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE January 11, 1993
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD:	December 1992		
GROSS HOURS IN REPORTING PERIOD:	744		
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323		
MAX DEPENDABLE CAPACITY (MWe-Net):	1,036		
DESIGN ELECTRICAL RATING (MWe-Net):	1,078		
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):	N/A		
4. REASON FOR RESTRICTION (IF ANY):			
	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	0.0	6,568.3	53,924.8
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	0.0	6,529.3	52,907.6
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWh _t)	0	19,903,179	155,060,158
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	0	6,702,577	51,792,749
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	-8,307	6,452,945	49,648,090
12. REACTOR SERVICE FACTOR (%)	0.0	74.8	68.3
13. REACTOR AVAILABILITY FACTOR (%)	0.0	74.8	70.4
14. UNIT SERVICE FACTOR (%)	0.0	74.3	67.0
15. UNIT AVAILABILITY FACTOR (%)	0.0	74.3	67.0
16. UNIT CAPACITY FACTOR (USING MDC) (%)	-1.1	70.9	60.7
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	-1.0	68.1	58.3
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	1.4	6.9
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):	No Outages Scheduled		
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:	01/22/93		

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE January 11, 1993
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: December 1992

DAY	POWER	DAY	POWER
-----	-----	-----	-----
1	-11	17	-11
2	-11	18	-11
3	-11	19	-11
4	-11	20	-11
5	-11	21	-11
6	-11	22	-11
7	-11	23	-11
8	-11	24	-12
9	-11	25	-12
10	-11	26	-11
11	-11	27	-12
12	-11	28	-11
13	-11	29	-11
14	-11	30	-11
15	-11	31	-12
16	-11		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
4	921003	S	744.0	C	I	Refuel outage (L1R05)

SUMMARY OF OPERATION:

The unit entered a scheduled refueling outage on 10/03/92. Scheduled return to service is 01/22/93.

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F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations
(None.)
2. ECCS System Outages
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual
(None.)
4. Major changes to Radioactive Waste Treatment Systems.
(None.)
5. Indications of Failed Fuel Elements.
(None.)

(Unit 1)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
1062	1E22-F031	Remove flushing spool.
1503	1E51-F064	Administrative control.
1691	1E51-F031	Replaced seal on limitorque.
1716	1E51-F045	Install spring pack.
1878	1E12-F017A	Replaced motor.
1879	1E22-F004	Administrative control.
1890	1E51-F063	Replaced seal on actuator.
1898	1E12-F090B	Replaced damaged conduit.
1902	1E12-F063A	Fill and vent the RHR system.
1905	1E12-F068A	Replaced motor.
1912	1DG01K	Replace air start regulator pressure gauge.
1921	1E51-F360	Inspection and votes testing.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

DECEMBER 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1140 Mwe.
	0237	Reduced power level to 895 Mwe due to system load.
	10 X0	Increased power level to 1135 Mwe.
2	0100	Reduced power level to 850 Mwe due to system load.
	1400	Increased power level to 1135 Mwe.
3	0300	Reduced power level to 1080 Mwe due to system load.
	1000	Increased power level to 1135 Mwe.
4	0100	Reduced power level to 930 Mwe due to system load.
	1030	Increased power level to 1135 Mwe.
7	0200	Reduced power level to 945 Mwe due to system load.
	1200	Increased power level to 1138 Mwe.
9	0230	Reduced power level to 1030 Mwe due to system load.
	1100	Increased power level to 1140 Mwe.
10	0300	Reduced power level to 900 Mwe due to system load.
	1100	Increased power level to 1138 Mwe.
11	0100	Reduced power level to 835 Mwe due to system load.
	1100	Increased power level to 1135 Mwe.
16	0100	Reduced power level to 995 Mwe due to system load.
	1130	Increased power level to 1138 Mwe.
20	0100	Reduced power level to 990 Mwe due to system load.
	1000	Increased power level to 1138 Mwe.
21	0030	Reduced power level to 995 Mwe due to system load.
	0900	Increased power level to 1135 Mwe.
22	0300	Reduced power level to 890 Mwe due to system load.
	1100	Increased power level to 1135 Mwe.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2) (CONTINUED)

Day	Time	Event
23	0200	Reduced power level to 1025 Mwe due to system load.
	0930	Increased power level to 1135 Mwe.
25	0100	Reduced power level to 850 Mwe due to system load.
	1500	Increased power level to 1135 Mwe.
27	0130	Reduced power level to 850 Mwe due to system load.
	1100	Increased power level to 990 Mwe.
	1400	Reduced power level to 950 Mwe due to system load.
	1700	Increased power level to 1075 Mwe.
28	0100	Reduced power level to 965 Mwe due to system load.
	1000	Increased power level to 1035 Mwe.
	2100	Increased power level to 1135 Mwe.
29	0130	Reduced power level to 850 Mwe due to system load.
	1030	Increased power level to 1132 Mwe.
30	0100	Reduced power level to 850 Mwe due to system load.
	1200	Increased power level to 1132 Mwe.
31	0030	Reduced power level to 950 Mwe due to system load.
	0300	Reduced power level to 850 Mwe due to system load.
	1230	Increased power level to 1133 Mwe.
	2400	Reactor critical, Generator on-line at 1133 Mwe.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

Technical Specification Amendment to remove the rod sequence control system and lower the Rod Worth Minimizer Setpoint to 10%.

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)
LER Number Date Description
(None.)

E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.
(See Table 2)
2. Average Daily Unit Power Level.
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.
(See Table 4)

C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L15638	Auxiliary Electric Equipment Room Ventilation	Compressor 0VE04CB.	None.	Replaced compressor.
L19178	Residual Heat Removal Full Flow Test Valve 2E12-F024A	Valve motor.	Motor thermals tripped during testing.	Replaced motor and actuator.

(No SOR Failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE January 11, 1993
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

- | | |
|--|---------------|
| 1. REPORTING PERIOD: | December 1992 |
| GROSS HOURS IN REPORTING PERIOD: | 744 |
| 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): | 3,323 |
| MAX DEPENDABLE CAPACITY (MWe-Net): | 1,036 |
| DESIGN ELECTRICAL RATING (MWe-Net): | 1,078 |
| 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): | N/A |
| 4. REASON FOR RESTRICTION (IF ANY): | |

	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	6,077.7	51,013.5
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	744.0	5,851.8	50,056.4
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh _t)	2,394,204	17,772,488	150,014,887
10. ELECTRICAL ENERGY GENERATED (MWh _e -Gross)	816,874	6,011,559	49,901,309
11. ELECTRICAL ENERGY GENERATED (MWh _e -Net)	793,754	5,781,429	47,912,933
12. REACTOR SERVICE FACTOR (%)	100.0	69.2	70.9
13. REACTOR AVAILABILITY FACTOR (%)	100.0	69.2	73.3
14. UNIT SERVICE FACTOR (%)	100.0	66.6	69.6
15. UNIT AVAILABILITY FACTOR (%)	100.0	66.6	69.6
16. UNIT CAPACITY FACTOR (USING MDC) (%)	103.0	63.5	64.3
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	99.0	61.1	61.8
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	8.4	12.6

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
No Outages Scheduled

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE January 11, 1993
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: December 1992

DAY	POWER	DAY	POWER
-----	-----	-----	-----
1	1,065	17	1,103
2	1,024	18	1,102
3	1,092	19	1,103
4	1,049	20	1,069
5	1,101	21	1,072
6	1,102	22	1,059
7	1,066	23	1,081
8	1,099	24	1,095
9	1,087	25	1,019
10	1,050	26	1,097
11	1,023	27	941
12	1,103	28	1,023
13	1,102	29	1,031
14	1,102	30	1,019
15	1,102	31	1,023
16	1,069		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

YEARLY SEQUENTIAL DATE NUMBER	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
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None.

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system load and maintenance activities.

F. . UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations
(None.)
2. ECCS System Outages
(See Table 5.)
3. Changes to the Off-Site Dose Calculation Manual.
(None.)
4. Major changes to Radioactive Waste Treatment Systems.
(None.)
5. Indications of Failed Fuel Elements.
(None.)

(Unit 2)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
2238 2239	2E12-F024A	Replace motor.
2241	2E12-F003A 2E12-F048A	Administrative control.
2264	2E22-S001	Calibrations.
2265	2E22-S001	Replace oil line tubing.
2266	2E22-S001	HFA relay inspection.