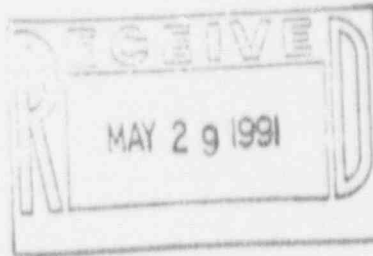


May 24, 1991



Certified Mail
Return Receipt Requested

Mr. Robert D. Martin
Regional Administrator
Region IV
U.S. NUCLEAR REGULATORY COMMISSION
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

RE: License No. SUB-1010; Docket No. 40-8027
Incident of April 24, 1991
10 CFR 20.405 Report

Dear Mr. Martin:

On April 24, 1991 Sequoyah Fuels Corporation (SFC) informed the NRC Region IV that the incident described herein had occurred; the NRC Operations Center Duty Officer was also notified. Early in the morning of April 24, 1991, an SFC employee discovered uncontained uranium on the fourth level of A-line hydrofluorination. The associated process was shut down until the situation could be assessed and corrective measures taken. It is believed that no significant quantities of material were released. A 10 CFR 20.403(b)(3) notification was made because of the potential for this section of the process to be down for repair for more than 24 hours.

The attached report is submitted pursuant to the requirements of 10 CFR 20.405(a)(1)(iv). Should you have any questions on this matter, please contact me at 918/489-3207.

Sincerely,

Lee R. Lacey

Lee R. Lacey
Vice President
Regulatory Affairs

91806060205 XA 368

LRL:nv

Attachment

xc: Keith E. Asmussen, General Atomics
Charles J. Haughney, NRC - NMSS

m8
TX-36

SEQUOYAH FUELS CORPORATION

INCIDENT OF APRIL 24, 1991

(10 CFR 20.405)

1. Estimate of each individuals exposure:

The SFC health physics staff has evaluated the situation and determined that it did not contribute any exposure to personnel above normal operating conditions.

2. Levels of radiation and concentrations of radioactive materials involved:

In-situ radiation measurements were performed to characterize the areal extent of contamination. No direct surface measurements were made. The maximum removable reading on the roof was 5790 disintegrations per minute per one hundred square centimeters (dpm/100 cm²) alpha. The maximum removable reading on the fourth level of hydrofluorination was 3200 dpm/100 cm² alpha.

During decontamination activities both area air samples and breathing zone air samples were collected. An area air sample collected immediately upon recognition of the release recorded 10.4 MPC in the immediate vicinity of the release. The highest air sample result from a fixed station air sampler was 0.4 MPC on fourth level hydrofluorination. The maximum breathing zone air sample recorded 0.07 MPC-hours.

The fenceline air samples did not reveal the presence of airborne radioactivity above normal operating levels.

3. The cause of exposure, levels, or concentrations:

The cause of the leak is attributed to a crack around the upper half of the circumference of the weld where the screw inlet nozzle (UO₂ feed) enters the first stage A-line HF reactor shell. The crack was most likely caused by improper alignment of the UO₂ inlet screw conveyor and fatigue created by movement of the UO₂ seal bin (up and down on load cells) in relation to the stationary reactor.

4. Corrective steps taken or planned to prevent recurrence:

In order to provide for more adequate detection of these incidents, a system will be installed to detect leaks in the cooling ducts. The system will be designed to provide the capability to detect leaks into the system commensurate with the level of monitoring performed

elsewhere at SFC. Also, consideration will be given to amending the periodic preventative maintenance schedule to include alignment of the UO₂ inlet screw conveyor as necessary. Considering the similarity between this incident and the Unusual Event of April 3, 1991, SFC has placed a high priority on resolving the problem and implementing corrective actions.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION IV
URANIUM RECOVERY FIELD OFFICE
BOX 25325
DENVER, COLORADO 80225

AUG 16 1991

URFO:GRK
Docket No. 40-8027

MEMORANDUM FOR: A. Bill Beach, Director
Division of Radiation Safety and Safeguards
Region IV

FROM: Gary R. Konwinski, Project Manager

SUBJECT: SFC-GORE SOIL SAMPLE RESULTS

As part of the inspection and independent measurement efforts at the SFC facility, ten soil samples, designated as SFC-A to SFC-J, were collected on June 10, 1991. These samples were sent to Oak Ridge National Laboratory for thorium-230, total uranium, and total radium analysis. A copy of the laboratory results, dated August 1, 1991, is attached. Additionally, a map showing the approximate locations of the soil samples, as well as SFC laboratory results of the split sampling, are attached.

For comparison purposes, the NRC and SFC data have been converted to like units and are shown below:

Soil Sample	NRC Th-230 (nCi/gm)	SFC Th-230 (nCi/gm)	NRC Ra (pCi/gm)	SFC Ra (pCi/gm)	NRC U (ug/gm)	SFC U (ug/gm)
A	0.8	1.7	2.1	0.5	3.9	3.6
B	4.1	5.4	2.6	1.6	19.5	23.0
C	2.1	4.7	2.3	0.6	22.4	22.0
D	3.2	14.0	2.4	1.6	52.6	49.0
E	137.8	260.0	3.5	3.5	324.3	220.0
F	0.2	2.1	2.7	1.0	1.8	4.8
G	0.4	1.7	3.2	1.8	3.0	3.8
H	0.6	1.8	1.5	0.8	27.2	31.0

m-9

9109180252

AUG 15 1991

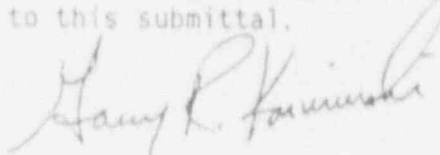
I	0.1	1.4	2.2	1.1	10.2	17.0
J	0.3	1.0	1.9	1.1	12.6	28.0

The data indicate that there is fairly good comparison between the laboratories. The thorium and radium concentrations are elevated above background and indicate that some transport of these constituents has taken place. Similarly, the uranium concentrations are elevated above background. The uranium data indicates that more of this constituent is available for transport and that it is more mobile.

A comparison of the 1991 and 1986 data tabulated on the SFC submittal indicates a decrease in radium, thorium, and uranium. This is probably due to a combination of diverting site runoff from the monitored outfalls to the combination stream, as well as the erosion of resident soils that had accumulated the various radionuclides. Regardless of the reason for these decreases, the radionuclides have been transported to downstream locations and have had their concentration diluted.

The Facility Investigation Findings Report does not quantify the amount of radionuclides in this area or their distribution. Rather, it concentrates on selected areas within or adjacent to the fenced area shown on the enclosed map. Because elevated concentrations of the various radionuclides were noted to exist during the 1986 sampling program and continue to exist as verified by the 1991 data, a more complete characterization of the area should be conducted and remediation proposed. Considering the storm-water retention work that has been completed at the site, future concentrations of radionuclides transported with eroded soils, along the flow paths characterized in this transmittal, should be minimal.

I am available to assist in further characterization and verification work as well as licensing issues in relation to this submittal.



Gary R. Konwinski
Project Manager

Attachments:

Oak Ridge National Laboratory Analyses
SFC Laboratory Analyses
Soil Sample Location Map

cc:

G. M. Vasquez, RIV
L. Yandell, RIV
C. J. Haughney
M. L. Horn
S. L. Uttal, OGC
T. R. Combs, OCA

Oak Ridge National Laboratory
Analytical Chemistry Division
Results of Analyses
Low Level Radiochemical Analysis

LLL12990 Page

Customer Name
Request Number
Project Number
Series

GARY KONWINSKI
LLL12990

Date Received 12-Jun-1991 14:55
Charge Number 33453837
Dept Number 3345
Date of Report 1-AUG-91

Approved By: *[Signature]*

Date 3/3/91

Customer ID	ACD Number	Date Sampled	TH-230	TOT/U	TOTAL RADIUM
Sample Matrix	Frequency		29	97	79
Material Description			+/- 3	+/- 18	+/- 14
			BQ/KG	BQ/KG	BQ/KG
ISPC-A	910612-071		12 31871	12 31921	EPA-903.0
ISOIL					
ISOIL					
ISPC-B	910612-072		TH-230	TOT/U	TOTAL RADIUM
ISOIL			150	480	96
ISOIL			+/- 10	+/- 10	+/- 16
			BQ/KG	BQ/KG	BQ/KG
			12 31871	12 31921	EPA-903.0
ISPC-C	910612-073		TH-230	TOT/U	TOTAL RADIUM
ISOIL			79	550	85
ISOIL			+/- 5	+/- 40	+/- 15
			BQ/KG	BQ/KG	BQ/KG
			12 31871	12 31921	EPA-903.0
ISPC-D	910612-074		TH-230	TOT/U	TOTAL RADIUM
ISOIL			120	1300	88
ISOIL			+/- 10	+/- 100	+/- 15
			BQ/KG	BQ/KG	BQ/KG
			12 31871	12 31921	EPA-903.0
ISPC-E	910612-075		TH-230	TOT/U	TOTAL RADIUM
ISOIL			5.1E3	8000	130
ISOIL			+/- .1E3	+/- 500	+/- 30
			BQ/KG	BQ/KG	BQ/KG
			12 31871	12 31921	EPA-903.0

[illegible]

CONFIDENTIAL INTERNAL CORRESPONDENCE

ENVIRONMENTAL DEPARTMENT

TO: Lee Lacey

DATE: June 25, 1991

FROM: Carol Couch

SUBJECT: NRC Split Soil Samples
06/10/91

Mr. Gary Konwinski, USNRC, was at the Facility on June 10, 1991, to take soil/sediment samples from the old 005 discharge stream bed, along the river bank, and from Outfall 001 stream bed.

This data is provided below:

Sample I.D.	Location	Ra-226(t) pCi/g	Th-230(t) pCi/g	U(t) ug/g
SFC-A	≈ 20' West 005	0.5	1.7	3.6
SFC-B	≈ 300' West 005	1.6	5.4	23.0
SFC-C	≈ 600' West 005	0.6	4.7	22.0
SFC-D	≈ 900' West 005	1.6	14.0	49.0
SFC-E	Conf. old 005 & Headwaters	3.5	260.0	220.0
SFC-F	Conf. old 004 & Headwaters	1.0	2.1	4.8
SFC-G	Headwaters River Bank	1.8	1.7	3.8
SFC-H	Conf. 001 and Headwaters	0.8	1.8	31.0
SFC-I	Downstream Port Road Bridge	1.1	1.4	17.0
SFC-J	Inflow 001 to Drainage Ditch	1.1	1.0	28.0

In 1986 testing of these same areas at the locations within close proximity indicates the following levels:

Sample I.D.	Location	Ra-226 pCi/g	Th-230 pCi/g	U(Nat) pCi/g	U(Nat) ug/g
005-00	Fence Line 005	2.22	49.0	159	238.5
005-03	300' West	1.30	73.0	170	255.0
005-06	600' West	2.40	354.0	277	415.5
005-09	900' West	1.60	106.0	505	757.5
004-06	600' West	1.14	0.6	6	9.0
001-R	Sediment from River	0.92	1.1	19	28.5
001-39 (Avg)	Downstream/Port Rd.Brg.	1.41	3.6	30	45.0
001-33 (Avg)	Current Discharge Point to Ditch	0.73	2.7	47	70.5

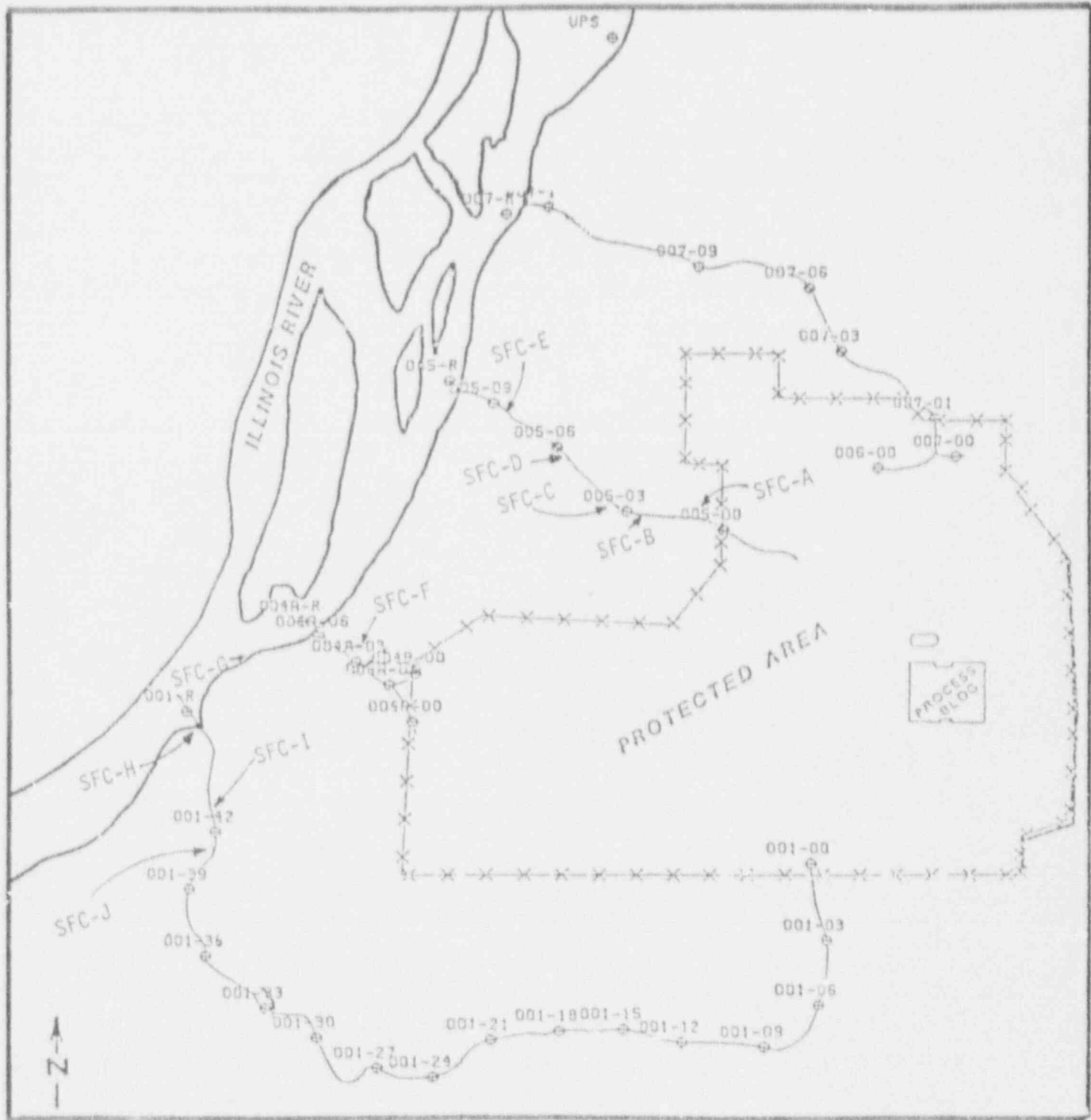
Comparison of 1986 and 1991 data show decreases for all constituents.

Attachment: Map

CLC:mmmp

xc: R. Adkisson
R. Graves
J. Mestepey
S. Munson
File

Soil Sample Locations





UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION IV
URANIUM RECOVERY FIELD OFFICE
BOX 25325
DENVER, COLORADO 80225

OCT 21 1991

URFO:GRK
Docket No. 40-8027

MEMORANDUM FOR: L. J. Callan, Director
Division of Radiation Safety and Safeguards
Region IV

THROUGH: Ramon E. Hall, Director *RA*
Uranium Recovery Field Office
Region IV

FROM: Gary R. Konwinski, Project Manager

SUBJECT: SFC-GORE SOIL SAMPLES RESULTS FOR POND #2 DECOMMISSIONING

As part of the inspection and independent measurement efforts at the SFC facility, 10 soil samples were collected on July 18, 1991. These samples were sent to Oak Ridge National Laboratory for thorium-230, total uranium, and radium-226 and 228 analysis. A copy of the laboratory results, dated October 8, 1991, is attached. The soil samples were taken in pairs, representing a shallow sample and a deep sample at the same location. The location numbers are keyed to the grid system developed by SFC for Pond #2. A map showing the sampled locations is also attached.

For comparison purposes, the samples that were split between the NRC and SFC have had their data converted to like units and are shown below:

Soil Sample	Depth in Feet	pCi/gm Ra-226		pCi/gm Thorium-230		pCi/gm Uranium	
		NRC	SFC	NRC	SFC	NRC	SFC
#2	0 - 0.5	62.2	*	232.4	*	1081	*
#2	3.5 - 4.0	1.4	0.9	2.3	10	7.6	275
#7	0 - 0.5	129.7	33	2188	850	459	429
#7	1.5 - 2.0	45.9	160	7025	6820	1108	1119
#9	0 - 0.5	218.9	69	5674	2950	513	422
#9	0.5 - 1.0	124.3	72	2107	1220	265	446
#13	0 - 0.5	9.7	160	100	3050	20	402
#13	1.5 - 2.0	56.7	2.6	1459	61	197	16
#21	0 - 0.5	297.2	140	7025	4740	1162	231
#21	2.0 - 2.5	3.8	12	21.9	36	4.3	18

*No split sample obtained. See attached October 11, 1991, SFC memorandum.

44-2030306

BFB

m-10

OCT 21 1991

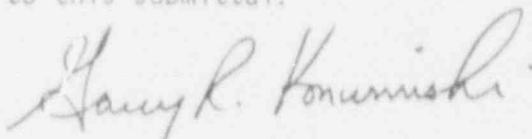
2

The data have a rather poor correlation between labs. Some of this can be explained. It is very likely that either the laboratory misreported or the samples were mistagged for NRC sample 13. This explanation is supported by lower constituent concentration in upper layers which is unlikely to occur as well as a comparison with the SFC laboratory results. For similar reasons, it is likely that the SFC laboratory transposed the radium-226 data for sample #7.

It should also be noted that all of the soil samples consisted of a saturated "paste." This consistency made separation techniques very difficult. If the samples had been oven dried, crushed, and then separated, the laboratory results would likely have had a much better comparison. Unfortunately, field conditions and time constraints precluded utilizing such protocol. Future soil samples collected at the site should follow a drying, crushing, and separating methodology.

By letter dated March 3, 1989, SFC submitted a Pond #2 remediation plan. Within this plan, SFC indicated that they would sample "the pond bottom, sides, and other potentially contaminated areas..." "to determine residual uranium, thorium, and nitrogen concentrations. Soils exhibiting significantly elevated concentrations of natural uranium (greater than 2000 pCi/gm) will be removed..." In an August 11, 1989, reply to this letter and remediation plan, the NRC discussed coordinating a split sampling program with the URFO office. The results of the SFC work and the NRC split indicate that the 2000 pCi/gm natural uranium concentration in the sludge materials has been achieved.

I am available to assist in further characterization and verification work as well as licensing issues in relation to this submittal.



Gary R. Konwinski
Project Manager

Attachments:

1. Oak Ridge National Laboratory Analyses
2. SFC Laboratory Analyses and memorandum
3. Soil Sample Location Map

cc:

G. M. Vasquez, RIV
J. P. Jaudon, RIV
C. J. Haughney, NMSS
M. L. Horn, NMSS
S. L. Uttal, TGC
T. R. Combs, OCA

INTERNAL CORRESPONDENCE

SEQUOYAH FUELS CORPORATION	BUSINESS DEVELOPMENT
TO: Lee Lacey	DATE: October 11, 1991
FROM: Ron Adkisson	SUBJECT: Pond #2 Sample Data

Attached for your review and submittal to NMSS is a copy of the summary analytical results from samples of residual sludge in Pond #2 taken on July 23-24, 1991. A reference guide map identifying the location of the twenty-one sample locations is also provided. This data indicates all areas of the pond are less than 2,000 pCi/gram uranium.

A total of 144 samples were taken from the twenty-one locations. These samples represent core samples taken at 6" intervals to a depth of four feet or until auger refusal. The pond slopes to the south end and a depth of four feet is well into the clay liner material.

Of the 144 samples collected, 63 samples were selected and analyzed for uranium, Th-230, Ra-226, Fluorides and Nitrates. Thirty of the 63 samples were analyzed for Arsenic and Molybdenum.

One area, Location #2, analyzed above 2,000 pCi/gram uranium. This area was resampled at five foot intervals around the location and it was determined the high levels were isolated within a ten foot arc from the sample location. This material was removed to a depth of 6" and transferred to Clarifier 2A for sludge processing.

Mr. Gary Konwinski selected ten samples for splitting and independent verification. Those sample locations are identified on the analytical summary with the letter "NRC" next to the sample location. For purposes of comparing data, since one NRC sample represents the excavated location noted above, SFC's original assay was 3,640 ug/g (2,438 pCi/g uranium and rerun at 3,415 ug/g (2,288 pCi/g). After removal of the top 6" the surface, now at 6", was 39, 48, and 42 pCi/g at three sample locations taken within the ten foot area.

If you need additional information, please advise.

Rn

RJA:jp
Attachments

xc: Decommissioning File
D. K. Isham
S. Munson
K. Schlag

POND #2 FINAL SAMPLE RESULTS

Location	Depth "	Uranium pCi/gm	Th-230 pCi/gm	Ra-226 pCi/gm	Fluoride ug/gm	Nitrate ug/gm	Arsenic ug/gm	Molybdenum ug/gm
3	0-0.5	17	210	31	2,680	3,240		
	0.5-1.0	8.5	12	2	1,550	3,410		
	1.0-1.5							
	1.5-2.0							
	2.0-2.5							
	2.5-3.0							
	3.0-3.5	<4	2.5	1.2	519	4,570		
	3.5-4.0	No Sample						
2	0-0.5	NRC Removed Top 6"						
	0.5-1.0	48	390	20	1,140	3,750	<10	21
	1.0-1.5							
	1.5-2.0							
	2.0-2.5							
	2.5-3.0							
	3.0-3.5		10	0.9	300	3,500	<10	<5
	3.5-4.0	NRC						
1	0-0.5							
	0.5-1.0	110	140	15	1,960	2,440		
	1.0-1.5	9	60	7.1	683	3,090		
	1.5-2.0							
	2.0-2.5							
	2.5-3.0							
	3.0-3.5	4.5	4.0	1.8	371	4,620		
	3.5-4.0							

POND #2 FBVAL SAMPLE RESULTS

Location	Depth *	Uranium pCi/gm	Th-230 pCi/gm	Ra-226 pCi/gm	Fluoride ug/gm	Nitrate ug/gm	Arsenic ug/gm	Molybdenum ug/gm
6	0-0.5	563	1,730	55	11,400	10,250	56	641
	0.5-1.0							
	1.0-1.5	375	118	2.3	5,040	7,000	15	185
	1.5-2.0							
	2.0-2.5							
	2.5-3.0	33	52	4.3	862	6,840	<10	<5
	3.0-3.5							
	3.5-4.0	No Sample						
5	0-0.5							
	0.5-1.0	1,099	5,350	190	12,600	25,000		
	1.0-1.5							
	1.5-2.0	1,180	6,600	190	13,800	21,750		
	2.0-2.5							
	2.5-3.0							
	3.0-3.5	106	180	2	1,980	2,300		
	3.5-4.0							
4	0-0.5							
	0.5-1.0	570	3,380	104	30,600	12,250	81	945
	1.0-1.5	282	290	11	2,200	7,750	<10	107
	1.5-2.0							
	2.0-2.5	20	500	11	1,290	7,840	<10	<5
	2.5-3.0							
	3.0-3.5	No Sample						
	3.5-4.0	No Sample						

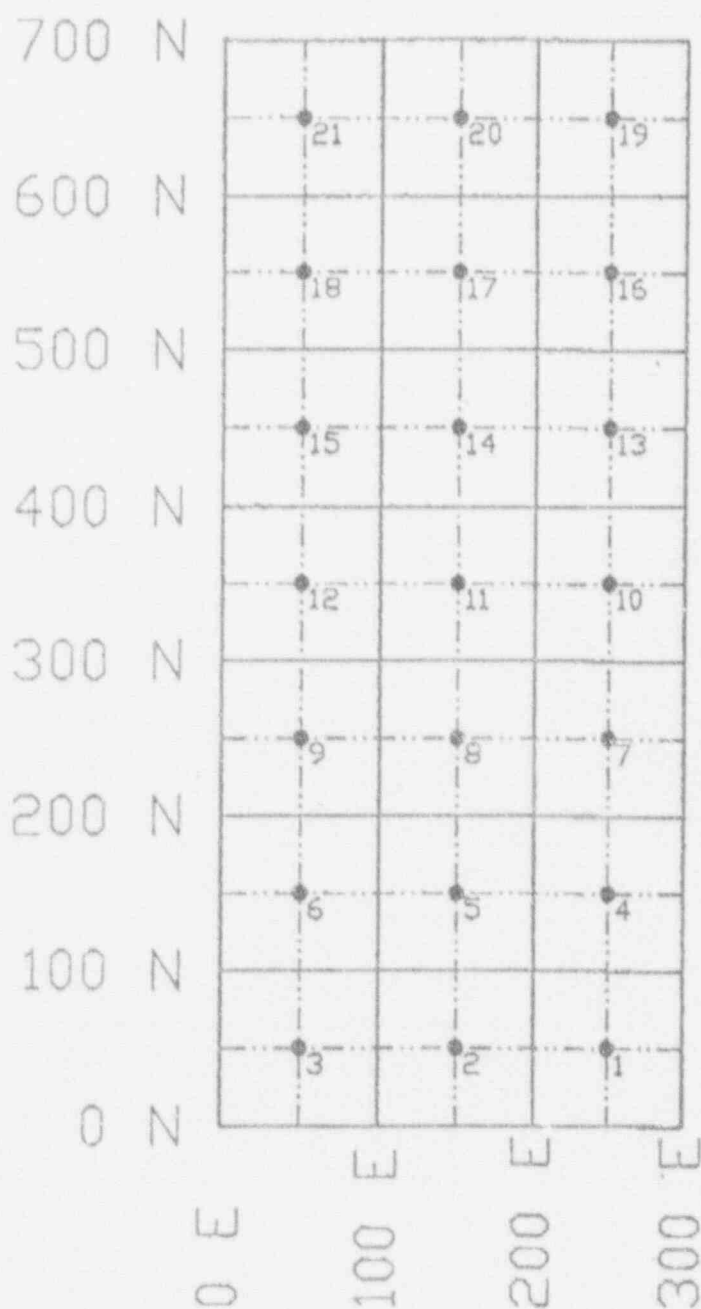
POND #2 FINAL SAMPLE RESULTS

Location	Depth *	Uranium pCi/gm	Tl-230 pCi/gm	Ra-226 pCi/gm	Fluoride ug/gm	Nitrate ug/gm	Arsenic ug/gm	Molybdenum ug/gm
9	0 - 0.5	422	2,950	69	3,600	9,000	66	656
	0.5 - 1.0	446	1,220	72	7,680	12,600	74	360
	1.0 - 1.5							
	1.5 - 2.0							
	2.0 - 2.5							
	2.5 - 3.0							
8	3.0 - 3.5	364	2,240	50	4,080	9,750	54	399
	3.5 - 4.0	No Sample						
	0 - 0.5	764	5,720	230	11,400	35,000	176	1,454
	0.5 - 1.0							
	1.0 - 1.5							
	1.5 - 2.0	1,059	5,740	180	13,800	13,500	205	1,694
7	2.0 - 2.5							
	2.5 - 3.0							
	3.0 - 3.5	39	9.8	1.2	778	1,010	14	11
	3.5 - 4.0							
	0 - 0.5	429	850	33	4,780	14,875	15	197
	0.5 - 1.0							
7	1.0 - 1.5	1,119	6,820	160	12,000	12,750	139	1,109
	1.5 - 2.0							
	2.0 - 2.5							
	2.5 - 3.0							
	3.0 - 3.5	119	400	18	2,500	5,880	13	157
	3.5 - 4.0	No Sample						

POND #2 FINAL SAMPLE RESULTS

Location	Depth *	Uranium pCi/gm	Th-230 pCi/gm	Ra-226 pCi/gm	Fluoride ug/gm	Nitrate ug/gm	Arsenic ug/gm	Molybdenum ug/gm
15	0 - 0.5	< 4	9.0	0.4	382	4,130		
	0.5 - 1.0	< 4	3.6	1.2	288	4,720		
	1.0 - 1.5		5.8	3.5	324	7,350		
	1.5 - 2.0	< 4						
	2.0 - 2.5	No Sample						
	2.5 - 3.0	No Sample						
	3.0 - 3.5	No Sample						
	3.5 - 4.0	No Sample						
14	0 - 0.5	737	4,190	150	8,400	42,500	164	1,405
	0.5 - 1.0							
	1.0 - 1.5							
	1.5 - 2.0							
	2.0 - 2.5	23	7.3	1.5	400	954	< 10	< 5
	2.5 - 3.0	< 268	160	3.1	528	1,770	< 10	< 5
	3.0 - 3.5							
	3.5 - 4.0							
13	0 - 0.5	NRC	402	3,050	160	5,340	13,000	72
	0.5 - 1.0							547
	1.0 - 1.5							
	1.5 - 2.0	NRC	16	61	2.6	533	5,520	< 10
	2.0 - 2.5						< 10	10
	2.5 - 3.0	6	33	5.1	411	4,890		< 5
	3.0 - 3.5							
	3.5 - 4.0	No Sample						

Pond No. 2 Sampling Plan To Confirm Solids < 2000 pCi/g Uranium (Nat)



21 Locations:
Sample to
depth of
refusal at 6"
intervals.

Analyze For:
U-total
Th-230
Ra-226

Oak Ridge National Laboratory
Analytical Chemistry Division
Results of Analyses
Low Level Radiochemical Analysis

LLL13176 Page

Customer Name
Request Number
Project Number
Series

GARY KONWINSKI
LLL13176

Date Received
Charge Number
Dept Number
Date of Report

22-Jul-1991 15:47
33053837
7345
8-OCT-91

Approved By

Date

Customer ID ACD Number Date Sampled
Sample Matrix Frequency
Material Description

102 0-0.5 910722-192

ISOIL
ISO9IL

IRA-226	IRA-228	ITH-230	ITOT-U
2300	150	8600	4.0E4
+/- 500	+/- 30	+/- 300	+/- .1E4
BQ/KG	BQ/KG	BQ/KG	BQ/KG
EPA-903.0	EPA-901.1	12 31871	12 31921

102 3.5-0.0 910722-193

ISOIL
ISO9IL

IRA-226	IRA-228	ITH-230	ITOT-U
53	32	84	280
+/- 11	+/- 11	+/- 4	+/- 10
BQ/KG	BQ/KG	BQ/KG	BQ/KG
EPA-903.0	EPA-901.1	12 31871	12 31921

107 0-0.5 910722-194

ISOIL
ISO9IL

IRA-226	IRA-228	ITH-230	ITOT-U
4800	190	8.1E4	1.7E4
+/- 1200	+/- 30	+/- .2E4	+/- .2E4
BQ/KG	BQ/KG	BQ/KG	BQ/KG
EPA-903.0	EPA-901.1	12 31871	12 31921

107 1.5-2.0 910722-195

ISOIL
ISO9IL

IRA-226	IRA-228	ITH-230	ITOT-U
1700	640	2.6E5	4.1E4
+/- 500	+/- 50	+/- .1E5	+/- .5E4
BQ/KG	BQ/KG	BQ/KG	BQ/KG
EPA-903.0	EPA-901.1	12 31871	12 31921

109 0-0.5 910722-196

ISOIL
ISO9IL

IRA-226	IRA-228	ITH-230	ITOT-U
8100	640	2.1E5	1.9E4
+/- 1500	+/- 50	+/- .1E5	+/- .3E4
BQ/KG	BQ/KG	BQ/KG	BQ/KG
EPA-903.0	EPA-901.1	12 31871	12 31921

Customer ID	ACD Number	Date Sampled	Sample Matrix	Frequency	Material Description
109 0.5-1.0	910722-197		ISOIL		
109 0.5-1.0	910722-197		ISOIL		
113 0-0.5	910722-198		ISOIL		
113 0-0.5	910722-198		ISOIL		
113 1.5-3.0	910722-199		ISOIL		
113 1.5-3.0	910722-199		ISOIL		
121 0-0.5	910722-200		ISOIL		
121 0-0.5	910722-200		ISOIL		
121 2.0-2.5	910722-201		ISOIL		
121 2.0-2.5	910722-201		ISOIL		

End of data for Request Number LLL13176

Total pages = 2

Cust. Copy

File Copy