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GCT-93-01

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U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2  
Monthly Performance Report  
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of December 1992.

Respectfully,

COMMONWEALTH EDISON COMPANY  
QUAD-CITIES NUCLEAR POWER STATION

  
Gerald C. Tietz  
Technical Superintendent

GCT/MB/dak

Enclosure

cc: A. B. Davis, Regional Administrator  
T. Taylor, Senior Resident Inspector

*Handwritten initials/signature*

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

December 1992

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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## I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

## II. SUMMARY OF OPERATING EXPERIENCE

### A. Unit One

Quad Cities Unit One ended its refuel outage Q1R12 by bringing the reactor critical at 2110 hours on December 14, 1992. The generator was synchronized to the grid at 0945 hours on December 16. The unit had been off line since the start of Q1R12, on September 20, 1992.

During December, Unit One reduced load twice on orders from Chicago Load Dispatch (CLD). The first reduction was on December 24, to 500 MWe, and the second was on December 28, to 340 MWe. The only other reductions of more than 20% power on unit one were to 290 MWe on December 29 in order to re-null the thrust bearing wear detector on the main turbine, and the 550 MWe on December 31 for monthly turbine testing.

### B. Unit Two

Quad Cities Unit Two was on line for the entire month of December. A back wash and precoat of the condensate demineralizers caused a load drop to 650 MWe on both December 2 and 28. A monthly test of the main turbine, performed on December 13, required a load drop to 600 MWe.

Seven additional load drops, of more than 20% power, were ordered by the CLD during the month of November for Unit Two. Each of these power reductions are listed below:

DATE	POWER
12/07/92	600 MWe
12/17/92	600 MWe
12/18/92	650 MWe
12/20/92	600 MWe
12/23/92	550 MWe
12/24/92	600 MWe
12/25/92	493 MWe
12/28/92	650 MWe

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS,  
AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

In accordance with Technical Specification Section 6.8.B.1, Revision O.K has been made to the Offsite Dose Calculation Manual (ODCM) on December 28, 1992. Information to support the change, and documentation that the change has been On-Site and Off-Site review approved is included in On-Site Review Report No. 92-54.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.



UNIT 1 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q03037	7800	Replace TIP channel #1 drive circuit breaker	Replaced circuit breaker.
Q03213	5700	Fabricate upper and lower head covers for 1B core spray room cooler.	Made new covers and replaced old ones.
Q03257	7800	Repair U1 MCC 18/19-5 cubicle F2 Reactor water recirc manifold crosstie valve.	Found local green indicating light missing its outer lock nut. Replaced local green indicating light.
Q03408	1700	Troubleshoot U1 "B" main steam line log rad monitor.	Installed new chassis.
Q03410	8100	Repair cracked contractor coil in MCC 18-1A cubicle A2.	Replaced old coil with new coil.
Q03510	8100	Replace overload on breaker in MCC 18-1A cubicle B2 for MO-1-1001-186A.	New overload relay was installed and passed test requirements.
Q03549	302	Repair scram discharge volume flow control indicator level switches.	Extensive troubleshooting was performed. The problem disappeared after reassembling the circuit boards. A water drip from the CRD's is the suspected cause of the original alarm.
Q03557	330	Repair stripped threads on CRD grid work.	Removed stripped nut and welded on a new nut.
Q03650	1200	Replace snap ring on ISI hanger 120Z-M110.	Found the external retaining ring missing. A new retainer ring was installed.
Q03653	1000	Cut out RHRSW valve upstream from 1A heat exchanger.	Removed valve and replaced with a new valve.

UNIT 1 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q03832	6700	Replace broken counter on 4kV horizontal magneblast breaker. (NEN #001)	Removed old counter and installed a new one.
Q03849	7800	Replace defective ring tongue terminal on U1 202-6B cubicle D4 MCC 18/19-5.	Replaced ring tongue terminal with new ring tongue.
Q03916	1700	Repair U1 signal isolator for SJAЕ offgas computer point.	Installed a new signal isolator and calibrated.
Q03947	203	Replace ruptured diaphragm on 1C main steam line safety valve.	Replaced disc.
Q03948	203	Replace ruptured diaphragm on 1D main steam line safety valve.	Replaced disc.
Q03949	203	Replace ruptured diaphragm on 1D main steam line safety valve.	Replaced disc.
Q03992	756	Repair LPRM 16-09B power supply.	Found blown fuse. Replaced fuse and adjusted power supply to the proper voltage.
Q04055	6700	Replace missing position indicator spring on 4kV horizontal magneblast breaker. (NEN #064)	Replaced position indicator spring.
Q04064	7800	Replace breaker in MCC 18-1A cubicle D3 main steam line drain valve 1A.	Installed new circuit breaker and repaired splice.
Q04158	1000	Realign U1 1001-42D limit switch for 1D RHR pump CCST suction valve.	Realigned limit switch.



# UNIT 1 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q04176	7800	Replace transformer lead splices in MCC 18-1A cubicle G2 for 1/2 DG vent fan.	Splices were cut out and taped. Motor starter and auxiliary contacts were replaced.
Q04193	6700	Replace missing position indicator spring on 4kV horizontal magneblast breaker. (NEN #105)	Replaced position indicator spring.
Q04284	202	Troubleshoot and repair Limitorque 1A recirc pump suction valve.	Found blue conductor of cable #12499 not landed at MCC 18/19-5 cubicle B1 terminal 8. Landed blue conductor at correct terminal.
Q04370	901	Find and fix hard ground in annunciator cabinet 901-34.	Found knife switch SW5 in the 901-34 panel door 4F had a lug on the bottom shorted to a mounting screw. Knife switch SW5 was relocated and ground cleared.
Q04459	7100	Replace switchgear cubicle 4A 480VAC feed to MCC 18-2.	Breaker was replaced.
Q47759	8300	Replace battery cell #9.	Replaced battery cell #9 and #10.
Q78927	4600	Repair 1-4699-203 and 1-4699-204 valves.	Replaced the valves with new ones.
Q79246	7800	Repair MCC 19-1 cubicle H2 RWCJ pump 1B.	Replaced the cracked coil with a new one.

UNIT 1 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q79819	3900	Repair 1/2 Diesel generator cooling water header drain valve.	Replaced valve and nipple.
Q89350	8350	Repair or replace terminal strip and plastic dividers for MCC 240VDC for 2301-5.	The terminal strip was replaced with a like part.
Q90254	1001	Repair RHRSW heat exchanger flow control valve.	As found: Body and disc exhibited damage from cavitation. Stem was bent beyond use. Body had pits. Anti rotational pin was bent on both ends and was tight against slots which prevented the valve from fully closing. As Left: Disassembled valve body. Installed new trim and stem. Belzond valve body at pitted areas. Packed valve with graphoil set and spacer set stroke. Reassembled valve and verified operation.
Q90680	2300	Repair HPCI testable check valve solenoid.	Replaced sealtite from check valve to junction box ITB95. Also replaced 45 degree connector and installed all bolts in junction box.
Q90964	202	Repair or replace sealtite for Limitorque 1B recirc pump suction valve.	Removed broken sealtite and replaced with new sealtite and fitting. Also installed cover bolt.

UNIT 2 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q04010	300	Clear accumulator alarm on HCU 34-55.	Found rust and water on top of accumulator level switch. Changed out level switch and charged instrument block with nitrogen to remove water in block.

#### IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

##### UNIT 1

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
92-029	10/29/92	H1 Rad Door Propped Open Consealing Posting Sign
92-030	12/13/92	1B Recirc PP tripped during ADS Logic Test
92-032	12/21/92	Admin requested commitments for fire protection not met

##### UNIT 2

There were no Licensee Event Reports for Unit 2 for this reporting period.

## V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

APPENDIX C  
OPERATING DATA REPORT

DOCKET NO 50-254  
UNIT One  
DATE January 04, 1993  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

OPERATING STATUS

0000 120192  
1. REPORTING PERIOD: 2400 123192 GROSS HOURS IN REPORTING PERIOD: 744  
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769  
DESIGN ELECTRICAL RATING (MWn-Net): 789  
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (Mwe-Net): N/A  
4. REASONS FOR RESTRICTION (IF ANY):  
5. NUMBER OF HOURS REACTOR WAS CRITICAL ..... THIS MONTH YR TO DATE CUMULATIVE  
6. REACTOR RESERVE SHUTDOWN HOURS ..... 0.0 0.0 3421.9  
7. HOURS GENERATOR ON LINE ..... 374.3 6160.5 138391.6  
8. UNIT RESERVE SHUTDOWN HOURS ..... 0.0 0.0 909.2  
9. GROSS THERMAL ENERGY GENERATED (MWH)..... 698191.2 13514800.8 297581823.8  
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)..... 223643.0 4344754.0 96419090.0  
11. NET ELECTRICAL ENERGY GENERATED (MWH)..... 210700.0 4166144.0 90875512.0  
12. REACTOR SERVICE FACTOR..... 55.22 71.15 78.59  
13. REACTOR AVAILABILITY FACTOR..... 55.22 71.15 80.47  
14. UNIT SERVICE FACTOR ..... 50.31 70.13 76.18  
15. UNIT AVAILABILITY FACTOR ..... 50.31 70.13 76.68  
16. UNIT CAPACITY FACTOR (Using MDC) ..... 36.83 61.68 65.05  
17. UNIT CAPACITY FACTOR (Using Design MWe) ..... 35.89 60.11 63.40  
18. UNIT FORCED OUTAGE RATE ..... 0.00 7.87 5.82  
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_  
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____



APPENDIX C  
OPERATING DATA REPORT

DOCKET NO 50-265  
UNIT Two  
DATE January 04, 1993  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

OPERATING STATUS

0000 120192  
1. REPORTING PERIOD: 2400 123192 GROSS HOURS IN REPORTING PERIOD: 744  
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2511 MAX. DEPEND. CAPACITY: 769  
DESIGN ELECTRICAL RATING (MWe-Net): 789  
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A  
4. REASONS FOR RESTRICTION (IF ANY):  
5. NUMBER OF HOURS REACTOR WAS CRITICAL ..... THIS MONTH YR TO DATE CUMULATIVE  
6. REACTOR RESERVE SHUTDOWN HOURS ..... 0.0 0.0 2985.80  
7. HOURS GENERATOR ON LINE ..... 744.00 5621.55 135641.45  
8. UNIT RESERVE SHUTDOWN HOURS ..... 0.0 0.0 702.90  
9. GROSS THERMAL ENERGY GENERATED (MWH)..... 1811916.00 12579523.20 292663409.20  
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)..... 595479.00 4095071.00 94025261.00  
11. NET ELECTRICAL ENERGY GENERATED (MWH)..... 571571.00 3896568.00 89030743.00  
12. REACTOR SERVICE FACTOR..... 100.00 64.81 77.28  
13. REACTOR AVAILABILITY FACTOR..... 100.00 64.81 78.94  
14. UNIT SERVICE FACTOR ..... 100.00 64.00 75.32  
15. UNIT AVAILABILITY FACTOR ..... 100.00 64.00 75.71  
16. UNIT CAPACITY FACTOR (Using MDC) ..... 99.90 57.69 64.29  
17. UNIT CAPACITY FACTOR (Using Design MWe) ..... 97.37 56.22 62.66  
18. UNIT FORCED OUTAGE RATE ..... 0.0 0.0 7.78  
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_  
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254  
UNIT One  
DATE January 4, 1993  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

MONTH December 1992

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	<u>-8</u>
2.	<u>-8</u>
3.	<u>-8</u>
4.	<u>-8</u>
5.	<u>-8</u>
6.	<u>-8</u>
7.	<u>-8</u>
8.	<u>-8</u>
9.	<u>-8</u>
10.	<u>-8</u>
11.	<u>-8</u>
12.	<u>-8</u>
13.	<u>-8</u>
14.	<u>-8</u>
15.	<u>-8</u>
16.	<u>65</u>

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	<u>143</u>
18.	<u>157</u>
19.	<u>360</u>
20.	<u>454</u>
21.	<u>693</u>
22.	<u>771</u>
23.	<u>780</u>
24.	<u>768</u>
25.	<u>495</u>
26.	<u>702</u>
27.	<u>783</u>
28.	<u>721</u>
29.	<u>620</u>
30.	<u>654</u>
31.	<u>726</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265  
UNIT Two  
DATE January 04, 1993  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

MONTH December 1992

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	785
2.	758
3.	790
4.	792
5.	787
6.	789
7.	753
8.	790
9.	789
10.	790
11.	790
12.	783
13.	739
14.	777
15.	786
16.	783

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	751
18.	763
19.	786
20.	738
21.	738
22.	784
23.	727
24.	732
25.	635
26.	787
27.	759
28.	754
29.	789
30.	790
31.	788

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D  
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-265

UNIT NAME Unit One

DATE January 4, 1993

REPORT MONTH December 1992

COMPLETED BY Mathew Benson

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-043	12/24/92	S	32.2	F	5				Load Drop per CLD
92-044	12/28/92	S	10.1	F	5				Load Drop per CLD
92-045	12/29/92	F	7.7	B	5				Null Thrust Bearing Wear Detector
92-046	12/31/92	S	5.3	B	5				Turbine Testing

**APPENDIX D  
UNIT SHUTDOWNS AND POWER REDUCTIONS**

DOCKET NO. 050-265

UNIT NAME Unit Two

DATE January 4, 1993

COMPLETED BY Mathew Benson

TELEPHONE 309-654-2241

REPORT MONTH December 1992

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-090	12/02/92	F	4.3	B	5				Cond Demin BW & PC
92-091	12/07/92	S	5.1	F	5				Load Drop per CLD
92-092	12/13/92	S	6.3	B	5				Load Drop per LD
92-093	12/17/92	S	3.4	F	5				Load Drop per CLD
92-094	12/18/92	S	2.9	F	5				Load Drop per CLD
92-095	12/20/92	S	14.6	F	5				Load Drop per CLD
92-096	12/23/92	S	5.4	F	5				Load Drop per CLD
92-097	12/24/92	S	5.7	F	5				Load Drop per CLD
92-098	12/25/92	S	17.3	F	5				Load Drop per CLD
92-099	12/28/92	F	4.6	B	5				Cond Demin BW & PC

## VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

### A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit: One

Date: 12/16/92

#### Valves Actuated

#### No. & Type of Actuation

1-203-3A	Post Maint. on Start-up from QIR12
1-203-3B	"
1-203-3C	"
1-203-3D	"
1-203-3E	"

#### Plant Conditions:

Start-up

#### Description of Events:

Rx Press 920 psig

### B. Control Rod Drive Scram Timing Data for Units One and Two

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 PSIG.



RESULTS OF SCRAM TIMING MEASUREMENTS  
 PERFORMED ON UNIT 1 & 2 CONTROL  
 ROD DRIVES, FROM 1-1-92 TO 12-31-92

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		
		0.375	0.900	2.00	3.5	7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
2-19-92	2	0.28	0.67	1.43	2.48	J-2 2.55	Drive Replacement (J-2), Scram Valve N-7
2-20-92	1	0.32	0.69	1.45	2.45	N-5 2.45	Scram Valve Work N-5
5-12-92	177	0.31	0.69	1.47	2.58	L-13 3.43	Start-up Scram Timing Unit Two
6-02-92	1	0.31	0.65	1.37	2.35	N-5 2.35	Scram Valve Work
6-26-92	2	0.27	0.62	1.32	2.32	C-13 2.26	ACCUM C-13, N-7
7-13-92	88	0.28	0.63	1.35	2.36	K-13 2.79	Unit 1 "B" Sequence
7-22-92	1	0.29	0.64	1.36	2.41	D-2 2.43	127 Diaphragm D-2 Unit 2
11-22-92	88	0.32	0.69	1.45	2.53	F-5 3.31	Unit 2 "B" Sequence
12-17-02	177	0.30	0.68	1.47	2.58	G-13 3.16	Unit 1 Start Up Scram Timing

## VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING  
INFORMATION REQUEST

QTP 300-S32  
Revision 2  
October 1989

1. Unit: 01 Reload: 12 Cycle: 13
2. Scheduled date for next refueling shutdown: 3-14-94
3. Scheduled date for restart following refueling: 6-13-94
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:  
  
NOT AS YET DETERMINED
5. Scheduled date(s) for submitting proposed licensing action and supporting information:  
  
NOT AS YET DETERMINED
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
  
NONE AT PRESENT TIME.
7. The number of fuel assemblies.
  - a. Number of assemblies in core: 724
  - b. Number of assemblies in spent fuel pool: 1557
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
  - a. Licensed storage capacity for spent fuel: 3657
  - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

QUAD CITIES REFUELING  
INFORMATION REQUEST

QTP 300-S32  
Revision 2  
October 1989

1. Unit: Q2 Reload: 11 Cycle: 12
2. Scheduled date for next refueling shutdown: 03/06/93
3. Scheduled date for restart following refueling: 06/05/93
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:  
NOT AS YET DETERMINED.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:  
NOT AS YET DETERMINED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
NONE AT PRESENT TIME.
7. The number of fuel assemblies.
  - a. Number of assemblies in core: 724
  - b. Number of assemblies in spent fuel pool: 2439
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
  - a. Licensed storage capacity for spent fuel: 3897
  - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

## VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRs
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCOMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRS
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center