



Carolina Power & Light Company

Company Correspondence

Brunswick Nuclear Project
P. O. Box 10429
Southport, N.C. 28461-0429

JAN 08 1993

FILE: B09-13510C
Serial: BSEP-93-0004

10CFR50.73

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-92-028

Gentlemen:

In accordance with Title 10 of the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is submitted in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

J. M. Brown, Plant Manager - Unit 2
Brunswick Nuclear Project

SFT/

Enclosure

cc: Mr. S. D. Ebner
Mr. R. H. Lo
BSEP NRC Resident Office

120115

9301130168 930110
PDR ADOCK 05000325
S PDR

EXPIRES: 5/31/95

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20565-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Brunswick Steam Electric Plant, Units 1 and 2

DOCKET NUMBER (2)

05000325

PAGE (3)

1

TITLE (4)

Group 6/Reactor Building Ventilation System Isolations and SBTG Initiation For Both Units Occurred During Plant Modification Wire Lift

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
12	11	92	92	- 028 -	000	1	10	93	BSEP Unit 2	50-0324
									FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following)(11)
04	20.402(b) <input type="checkbox"/> 20.405(c) <input checked="" type="checkbox"/> 50.73(a)(2)(iv) <input type="checkbox"/> 73.71(b) <input type="checkbox"/>
POWER LEVEL (10) 00	20.405(a)(1)(ii) <input type="checkbox"/> 50.36(c)(1) <input type="checkbox"/> 50.73(a)(2)(v) <input type="checkbox"/> 73.71(c) <input type="checkbox"/>
	20.405(a)(1)(iii) <input type="checkbox"/> 50.36(c)(2) <input type="checkbox"/> 50.73(a)(2)(vii) <input type="checkbox"/> OTHER <input type="checkbox"/>
	20.405(a)(1)(iii) <input type="checkbox"/> 50.73(a)(2)(i) <input type="checkbox"/> 50.73(a)(2)(viii)(A) <input type="checkbox"/>
	20.405(a)(1)(iv) <input type="checkbox"/> 50.73(a)(2)(ii) <input type="checkbox"/> 50.73(a)(2)(viii)(B) <input type="checkbox"/>
	20.405(a)(1)(v) <input type="checkbox"/> 50.73(a)(2)(iii) <input type="checkbox"/> 50.73(a)(2)(x) <input type="checkbox"/>

LICENSEE CONTACT FOR THIS LER (12)

NAME

Steve F. Tabor, Regulatory Compliance Specialist

TELEPHONE NUMBER

(919) 457-2178

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS

SUPPLEMENTAL REPORT EXPECTED (14)

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X	YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	02	15	93

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines) (16)

Units 1 and 2 were in Cold Shutdown for a dual unit outage that started April 21, 1992. On December 12, 1992 at approximately 0936 hours, while performing a modification to install a hardened wetwell vent, a wire lift resulted in the actuation of the Containment Atmospheric Control (CAC) isolation logic. By design, the CAC isolation resulted in a loss of power to the stack radiation monitor isolation circuit which caused both Units 1 and 2 to experience an isolation of the CAC and Containment Atmospheric Dilution systems primary containment isolation valves, reactor building ventilation system isolations, and initiations of the Standby Gas Treatment systems.

Immediate corrective actions included the restoration of all affected systems to the normal standby lineup. A root cause analysis is in progress. The results of this analysis including further corrective actions will be reported in a supplement to this LER.

The safety significance of this event is considered minimal since both Units were in Cold Shutdown and the affected systems responded in accordance with design. Previous similar events have been reported in LERs 1-90-025 and 1-90-027.

EXPIRES: 5/31/96

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20585-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Brunswick Steam Electric Plant Unit 1	05000325	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2
		92	- 028 -	000	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

TITLE

Group 6/Reactor Building Ventilation System Isolations and SGBT Initiation For Both Units Occurred During Plant Modification Wire Lift

INITIAL CONDITIONS

Units 1 and 2 were in Cold Shutdown for a dual unit outage that started April 21, 1992.

EVENT NARRATIVE

On 12/11/92, wiring modifications in the Unit 2 Control Room Panel XU-53 were in progress in accordance with the installation instructions of the Hardened Wetwell Plant Modification, PM 92-073. At approximately 0936 hours, during performance of a step in the plant modification installation procedure which required lifting and moving a neutral connection for Terminal Block K24 to Terminal Block K22 to support the addition of a relay to the Containment Atmospheric Control (CAC) System Isolation System, relays located in Panel XU-53 unexpectedly operated. The lifted wire was immediately reterminated to its original termination point and the Operations Shift Supervisor was notified.

The lifting of the neutral connection at Terminal Block K24 resulted in the de-energization of the CAC isolation logic relay 2-CAC-63-3. By design, this resulted in a loss of power to the stack radiation monitor isolation circuit. The loss of the stack radiation monitor isolation circuit resulted in isolation of the Units 1 and 2 CAC and Containment Atmospheric Dilution (CAD) related Primary Containment Isolation System (PCIS) valves, isolation of the Reactor Building Ventilation System for Units 1 and 2, and the start of Units 1 and 2 Standby Gas Treatment (SBGT) Systems.

Following the event, Operations verified that the affected systems had responded in accordance with plant design. Upon retermination of the lifted wire to its original termination point, the isolation logic was reset and the systems were realigned to their configuration.

CAUSE OF EVENT

A root cause analysis is currently in progress. The results of this analysis will be reported in a supplement to this LER.

CORRECTIVE ACTIONS

Corrective actions to prevent recurrence of this event will be reported following completion of the root cause analysis.

SAFETY ASSESSMENT

The safety significance of this event is considered minimal in that both Units were in Cold Shutdown at the time of the event and the affected systems responded in accordance with design.

PREVIOUS SIMILAR EVENTS

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20666-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Brunswick Steam Electric Plant Unit 1	05000325	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3
		92	- 028 -	000	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Previous similar events have been reported in LERs 1-90-025 and 1-90-027.

ELIS COMPONENT IDENTIFICATION

<u>System/Component</u>	<u>ELIS Code</u>
PCIS	JM
CAC/CAD	IK
Reactor Building Ventilation	VA
Standby Gas Treatment	BH