

Iowa Electric Light and Power Company

December 31, 1992

NEP-92-0910

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington DC 20555

Project: Duane Arnold Energy Center
Subject: Transmittal of Emergency Planning Documents
File: A-304

To Whom It May Concern:

We are forwarding, in accordance with Appendix E to 10CFR50, three controlled copies (one to NRR and two to NRC Region III) of our

<input type="checkbox"/> Duane Arnold Energy Center Emergency Plan	Revision _____
<input type="checkbox"/> Emergency Plan Implementing Procedure	Revision _____
<input type="checkbox"/> Iowa Electric Light & Power Co. Corporate Emergency Response Plan	Revision _____
<input checked="" type="checkbox"/> Corporate Plan Implementing Procedure	Index Revision 37_____ CPIP 1.3 Revision 12_____ Revision _____
<input type="checkbox"/> Duane Arnold Energy Center Emergency Telephone Book The Emergency Telephone Book is considered to be proprietary to Iowa Electric.	Revision _____

Insert the revised document(s) in your files and discard the obsolete one(s). Please acknowledge the receipt of the enclosed documents by signing and dating the section below and returning a copy of this letter to me.

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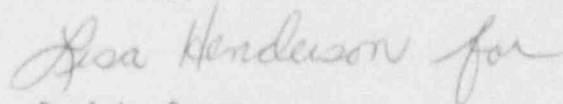
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☒ No proprietary information is contained in these revisions.

☐ The Emergency Telephone Book is, in total, considered to be proprietary to Iowa Electric.

Sincerely,



Paul L. Serra
Manager, Emergency Planning

cc: IE Supervisor, Emergency Planning
IE Manager, Nuclear Licensing
IE Emergency Planner - Procedures
NRC Region III (2)
NRC Resident Inspector

The document(s) listed above have been received by the NRC.

- | | | |
|-----------------------|-----------------------|-------|
| 1. Headquarters, NRR, | _____ | _____ |
| | Control Copy #91 | Date |
| 2. NRC Region III, | _____ | _____ |
| | Control Copy #161-162 | Date |

PLS/lld

MEMORANDUM

SUBJECT: CORPORATE PLAN IMPLEMENTING PROCEDURES MANUAL

DAEC Procedures (

Series)

Manual No.

Revisions to your controlled copy of the manual, as checked above, are attached. As indicated below, please remove and discard the superseded material and insert the revised material, or perform the changes as directed herein.

Please acknowledge that the above action has been taken by signing below and returning this memorandum to the Document Control Resource Center, Duane Arnold Energy Center, 3277 DAEC Road, Palo, IA 52324

Date _____

Iowa Electric Light and Power Company	REVISION: 37
	PAGE 1 of 1
CORPORATE PLAN IMPLEMENTING PROCEDURES INDEX	DATE: 12/22/92

<u>DOCUMENT NUMBER</u>	<u>TITLE</u>	<u>REV.</u>	<u>DATE</u>
1.2	Corporate Notification	12	12/11/92
1.3	Activation and Operation of the EOF	12	12/22/92
1.4	Release of Emergency-Related Information	6	09/14/90
1.5	Building Security Implementation	6	09/12/91
1.6	Communication and Data Transmission Systems Coordination	2	07/25/90
1.7	FTS-2000 and Emergency Response Related Communications	0	11/25/92
2.1	Dose Assessment and Protective Action Recommendations	11	12/11/92
2.2	Technical Support Coordination	4	01/07/91
2.3	Administrative Services and Corporate Response Coordination	5	11/25/92
2.4	Quality Assurance and Quality Control Activities	4	03/04/92
2.5	Insurance and Claims	2	06/11/90
2.6	Legal Assistance	2	07/20/90
4.1	Maintenance of Emergency Preparedness	8	02/17/92
4.2	Training, Drills and Exercises	6	12/20/91
4.3	Public Information Program	4	07/09/90
5.1	Emergency Response and Recovery Director	5	07/19/90
5.2	Radiological and EOF Manager	10	07/08/91
6.1	Recovery and Re-entry	0	04/01/91

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 1 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

1.0 PURPOSE

- 1.1 This procedure provides instructions for activation and operation of the Emergency Operations Facility (EOF).
- 1.2 The key response functions performed in the EOF are as follows:
 - 1.2.1 Coordinate emergency response activities with local, State and Federal agencies and support organizations.
 - 1.2.2 Coordinate technical and logistics support for response actions.
 - 1.2.3 Monitor and evaluate offsite radiological consequences of an emergency at the DAEC.

2.0 APPLICABILITY

- 2.1 This procedure shall be implemented upon declaration of a SITE or GENERAL EMERGENCY and is applicable to Corporate and Site personnel who function as members of the Corporate Emergency Response Organization.
- 2.2 During an event classified as NOTIFICATION OF UNUSUAL EVENT or ALERT, the Emergency Response and Recovery Director (ER&RD) may, at his discretion, initiate partial or full activation of the Emergency Operations Facility.

3.0 RESPONSIBILITIES

3.1 Emergency Response and Recovery Director

- 3.1.1 Is responsible for the overall direction and control of IELP's integrated emergency response and recovery effort.
- 3.1.2 Activate the EOF, when required.
- 3.1.3 Upon activation, make decisions regarding notification and recommendation of protective actions to offsite authorities.
- 3.1.4 Has the full authority and responsibility to make commitments for the company related to emergency response and the DAEC recovery activities.
- 3.1.5 Can also act as Company Spokesperson.

3.2 Corporate Management Representative

- 3.2.1 The primary responsibility is to assist the Emergency Response

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 2 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

and Recovery Director (ER&RD) to ensure requisite activities can be taken to protect the health and safety of the public.

3.2.2 May conduct communications with briefings for the Corporate Board of Directors and officials of industry liaison groups.

3.2.3 May act as the interface for the ER&RD when required for the legal, financial and insurance concerns of the company.

3.2.4 Can also act as Company Spokesperson.

3.3 Radiological and EOF Manager

3.3.1 Supervise the activation of the EOF.

3.3.2 Inform local, State and Federal authorities in the EOF and located offsite of the event status and response actions being taken.

3.3.3 Develop Protective Action Recommendations for approval by the Emergency Response and Recovery Director.

3.4 Technical and Engineering Support Supervisor

3.4.1 If communicator and recorder positions are not filled prior to activation of the EOF, designate personnel to fill these positions.

3.4.2 Coordinate with the Technical and Engineering Supervisor at the TSC to provide engineering and analytical support which may be required.

3.4.3 Provide technical analysis and trends to assist the ER&RD in anticipating conditions which may develop or for recovery response.

3.5 Radiological Assessment Coordinator

3.5.1 Upon activation, assume control of the Iowa Electric Radiological Monitoring Field Teams.

3.5.2 Coordinate with the Site Radiation Protection Coordinator during the performance of dose projection activities.

3.5.3 Coordinate offsite radiological monitoring and dose assessment activities performed by Iowa Electric with those being performed by local, State and Federal organizations.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 3 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

3.6 Emergency Support Manager

- 3.6.1 Provide assistance to the ER&RD and/or Rad and EOF Manager in activating the EOF and staffing the Corporate Emergency Response Organizations.
- 3.6.2 Coordinate, as required, with appropriate DAEC and Corporate Emergency Response Organization Supervisory personnel to ensure that technical, administrative and logistics needs required to support response actions are being met.
- 3.6.3 Coordinate the activities, resources, and capabilities of all company organizations and response groups to perform tasks in support of the overall response and recovery effort.

3.7 Support Services Coordinator

- 3.7.1 Ensure that building security has been established. Coordinate activities with building management as needed.
- 3.7.2 Coordinate, as necessary, with the Security and Support Supervisor in the TSC and Emergency Support Manager in the EOF to ensure that administrative and logistics support is provided, as required.

4.0 INSTRUCTIONS

4.1 Initial Notification

- 4.1.1 The Emergency Response and Recovery Director shall initiate notification of the Corporate Emergency Response Organization, as described in CPIP 1.2, "Corporate Notification", and direct activation of the EOF.

NOTE

EOF activation is required for all events classified as a SITE or GENERAL EMERGENCY and may, at the discretion of the ER&RD, be activated for events of a lower classification.

4.2 Staffing

- 4.2.1 Positions to be staffed in the EOF are defined in Attachment 3, "Corporate Emergency Response Organization."
 - a) A Corporate Emergency Response Organization chart listing

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 4 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

key positions will be posted in the EOF.

- b) Members of the Corporate ERO will sign-in on the EOF staffing board as they assume their positions.
- c) The Event Historian will be responsible for updating names for the key positions on the Corporate Emergency Response Organization chart as personnel are rotated.

4.2.2 Staff support personnel shall report to their designated work locations; review instructions associated with their job function, as appropriate; and advise their functional supervisor of their readiness.

4.2.3 Communication positions to be manned shall be as follows:

- a) TSC "Dedicated" Communicator - receive unique or specialized plant status information, transmit information requests from the EOF to the TSC, and act as the ER&RD's personal communicator.
- b) Field Team Director - relay directions and information to Iowa Electric Radiological Monitoring Field Teams and receive radiological survey information obtained by the State Field Teams.
- c) Radiological Status Communicator - provide dose projection information and radiological survey results to the State Field Team Captain and receive radiological survey information obtained by the State Field Teams.
- d) Engineering Information Communicator - receive and transmit detailed technical information regarding plant status and engineering solutions, coordinate engineering activities at the EOF with those being accomplished in the TSC, and act as the Technical and Engineering Support Supervisor's personal communicator.
- e) NRC ENS Communicator - transmit information as necessary and monitor conversations and information being transmitted between ENS extensions at the DAEC in the TSC and Control Room and NRC Emergency Operations Centers in Bethesda, MD and Glen Ellyn, IL. (Use Attachment 8 as guideline.)
- f) NRC HPN Communicator - transmit information as necessary and monitor conversations and information being transmitted between HPN extensions at the DAEC in the TSC and NRC

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 5 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

Emergency Operations Centers in Bethesda, MD and Glen Ellyn, IL. (Use Attachment 7 as guideline.)

- g) Technical Recorder - relays information regarding plant status, protective actions, dose projections to Technical Liaisons in the ENC, and the Linn County, Benton County and State EOCs. Develop written report within 8 hours to all offsite Emergency Operation Centers per section 4.5.4. Reviews ACP 1402.3 for other 10 CFR reporting requirements (Use Attachment 10 as a guideline.)

4.2.4 County, State and Federal agencies may dispatch representatives to the EOF upon declaration of a SITE or GENERAL EMERGENCY or upon being advised that the EOF is being activated at a lower classification.

- a) Upon arrival of the representatives at the 14th Floor access door, security personnel shall verify the identity of each of the representatives, as prescribed in CPIP 1.5, and inform the Switchboard Operator of their arrival.
- b) The Switchboard Operator shall advise the Radiological and EOF Manager of their arrival.
- c) These representatives shall be briefed by or under the direction of the Rad and EOF Manager and directed to their respective work spaces in the EOF.

4.2.5 Additionally, the NRC's Incident Response plan defines the NRC's responsibilities during an emergency, which includes a monitoring only role, an inform role, an advisory role, and a limited direction role. A detailed description of these roles is listed in Attachment 11, "NRC Roles During a Nuclear Power Plant Emergency".

- a) The Federal Radiological Emergency Response Plan (FRERP) also establishes the NRC as Lead Federal Agency (LFA) for response to nuclear power plant accidents. As LFA, the roles assigned to the NRC include the following:
 - 1) Coordinates federal technical evaluations and assessments.
 - 2) Acts as Lead Technical Spokesperson for the Federal Government.
 - 3) Assists the state in interpretation and analysis of technical information.
 - 4) Keeps the White House informed of technical assessments.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 6 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

NOTE

There is not necessarily a direct correlation between an Emergency Class and a NRC Response Mode. Response Mode depends on such factors as: the quality and completeness and clarity of licensee's event description; NRC's perception on the appropriateness of the classification; if the event is over or ongoing; prognosis of event and professional judgement; etc.

- 4.2.6 NRC Site Team members initially dispatched who are expected to be assigned to the EOF are listed in Attachment 12, "Typical Organization of the NRC Site Team".

NOTE

Initial briefings expected from licensee staff are not intended to adversely impact licensee's response efforts.

- 4.2.7 Upon arrival of the NRC Site Team personnel at the EOF, a briefing shall be conducted by or under the direction of the ER&RD which covers:
- a) Offsite radiological monitoring activities and results.
 - b) Dose projection results and protective action recommendations that have been made.
 - c) Protective actions that have been implemented by offsite authorities in the EPZ.
 - d) Media briefings and press release status.
 - e) Response actions in progress at the EOF to assist in mitigating/terminating the event at the site.
 - f) Local and State interfaces that have been established.
 - g) Prognosis of the event.
 - h) Potential need for Technical Specifications/License exemptions.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 7 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 20 1982

4.3 Activation

- 4.3.1 The ER&RD should brief the Radiological and EOF Manager and the Emergency Support Manager on plant status and response actions underway and identify/prioritize corporate response actions to be taken in support of the response and recovery activities at the DAEC.
- 4.3.2 Each of the above managers should further brief Corporate Emergency Response Organization functional supervisory personnel under their direct supervision regarding plant conditions and response action status, and define specific tasks, as appropriate, to be accomplished.
- 4.3.3 Functional supervisors responsible for response actions that already may be in progress at the site, shall contact their counterparts at the DAEC to determine the status of the particular functions involved. These personnel and their contacts include:
 - a) Radiological Assessment Coordinator - Site Radiation Protection Coordinator
 - b) Technical and Engineering Support - Technical and Engineering Supervisor
 - c) Support Services Coordinator - Security and Support Supervisor
- 4.3.4 Functional supervisors should verify that their staffs are ready to initiate response actions; conduct staff briefings on plant conditions, and the status of response actions underway; and, once accomplished, advise their respective managers of their readiness to initiate response activities.
- 4.3.5 The Radiological and EOF Manager and Emergency Support Manager shall advise the ER&RD of their readiness to assume their assigned responsibilities upon:
 - a) Assuring that their functional groups are fully aware of plant status and response actions underway.
 - b) Assuring that adequate staffing is in place to carry out their assigned functions.
 - c) The Support Services Coordinator shall ensure that the security area is established with the items listed in Attachment 4, "Security Post Equipment and Supplies" of CPIP 1.5.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 8 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

4.3.6 The ER&RD shall declare the EOF activated and shall inform the Emergency Coordinator, Iowa Electric Spokesperson once satisfied that the following conditions have been met:

- a) Staffing is in place to perform the following functions:
 - (1) Offsite communications, including follow-up notifications with local, State and Federal agencies.
 - (2) Interface with offsite support organizations, industry support groups and contract firms.
 - (3) Dose projection and dose assessment activities.
 - (4) Offsite radiological monitoring and assessment activities including coordination and interface with local, State and Federal organizations involved.
 - (5) Development and transmittal of protective action recommendations for the public within the Emergency Planning Zone (EPZ).
 - (6) Upgrading and downgrading of the event classification.

NOTE

Full activation of the EOF does not preclude the EOF from assuming any of the functions listed in 4.3.6, if the capability exists in the EOF, prior to full EOF activation.

- b) EOF staff members have been briefed.
- c) All equipment in the EOF is operational.

4.3.7 Upon declaring the EOF activated:

- a) The ER&RD shall specifically inform the Emergency Coordinator that the EOF will assume responsibility for all offsite communications and that he has assumed responsibility for the decision to notify and recommend protective actions to offsite authorities.

NOTE

This responsibility may not be delegated.

- b) The Radiological and EOF Manager shall inform the county, State and Federal Emergency Operations Centers (EOCs) that the EOF is activated and that the EOF has assumed responsibility for all offsite interfaces and offsite dose

CORPORATE PLAN IMPLEMENTING PROCEDURE	NO. CPIP - 1.3 Rev. 12
	PAGE 9 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

assessment, radiological monitoring and protective action decision making activities.

- c) The Radiological Assessment Coordinator shall contact the Site Radiation Protection Coordinator to relieve the site of the responsibility for offsite radiological monitoring and, upon turnover of such responsibility, initiate offsite radiological assessment activities.

4.3.8 Upon activation, the Radiological and EOF Manager shall assume responsibility for maintenance of the EOF Log provided as Attachment 4, "EOF Log Sheet" or equivalent.

- a) Log entries should not duplicate information contained in messages, status reports, etc., but should contain a chronological history of actions, decisions, and important communiques which occur.
- b) Attachment 5, "Log Entry Topics", identifies typical types of information which should be required.
- c) In support of the Radiological and EOF Manager, additional logs should be kept by functional supervisors which reflect activities in progress, problems and their resolutions, and miscellaneous information which may be important from a historical perspective.

4.4 Status Monitoring and Communications

4.4.1 Corporate Emergency Response Organization personnel in the EOF should be apprised of plant status on a periodic basis using overhead projection of Emergency Data System (EDS) or a pre-established report form (Plant Status Form) contained in EPIP 2.5.

- a) Normally these reports are initiated by the Computer Services Representative in the TSC when the Emergency Data System (EDS) is turned on to the NRC Emergency Response Data System (ERDS) to provide in plant data to the NRC and will be transmitted from the VAX computer and be available for printout at both the TSC and EOF.
- b) If the computer is unavailable, such reports should be transmitted via telecopier or verbally to the Plant Status Recorder.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12 PAGE 10 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

NOTE

Trends of important plant variables should be maintained by the Plant Status Recorder or as otherwise directed by the Technical and Engineering Support Supervisor, as described in CPIP 2.2, "Technical Support Coordination".

- c) Monitors for SPDS display of plant computer points will be used for display of various trends of Reactor or Containment or effluent parameters.
- d) The EOF Public Address system will be utilized to highlight significant changes in the plant status.

4.4.2 Communications between the TSC and EOF shall be recorded by the EOF Communicator on Attachment 6, "Status Update Message", or equivalent as follows:

- a) Information transmitted on the TSC Dedicated Circuit shall be recorded on sequentially numbered message forms, initialed by the ER&RD, copied and distributed in accordance with indicated distribution.

NOTE

The TSC Dedicated Communicator should identify information contained on these message forms which should be transcribed by the Event Historian on the Emergency Status Chart.

- b) The original copies of such communications shall be provided to the Event Historian for filing.
- c) The Event Historian shall also transcribe those items identified by the ER&RD on each message form on the Emergency Status Chart for display to all personnel in the EOF.

4.4.3 Information provided to the Engineering Information Communicator shall be recorded, as prescribed in CPIP 2.2, "Technical Support Coordination".

- a) The Technical and Engineering Support Supervisor shall keep the Emergency Support Manager apprised of pertinent technical data communicated.
- b) Records of such communications shall be provided to the Event Historian at periodic intervals.

4.4.4 Offsite radiological status information to be displayed in the EOF should include the following:

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 11 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

- a) Effluent release information and dose projection summaries.
- b) Protective action recommendations
- c) Offsite Radiological Monitoring Team results.
- d) Wind direction and Met conditions.

4.4.5 Development of offsite radiological information and maintenance of such status is prescribed in CPIP 2.1, "Dose Assessment and Protective Action Recommendations".

- a) Status reports containing information related to release rates, dose projections and protective action recommendations will normally be developed on the VAX computer as part of the dose projection program or as described in CPIP 2.1, Attachment 2.
- b) The Radiological Assessment Coordinator shall review the status report printout with the Radiological and EOF Manager who is responsible for developing protective action recommendations.

NOTE

- 1) Significant changes from the preceding report shall be highlighted or otherwise uniquely identified.
- 2) Parameter trending shall be accomplished, as specified in CPIP 2.1.
- c) The protective action recommendations developed shall be approved by the ER&RD, copied and distributed in accordance with the indicated distribution.
- d) Protective action recommendations will be displayed on an overhead transparency along with the time the recommendation was made.
- e) The EOF Public Address system shall be utilized to notify personnel in the EOF of changes in protective action recommendations.
- f) The original copy of the report shall be provided to the Event Historian for filing.

4.4.6 Communications associated with offsite radiological monitoring and dose projection activities shall be recorded, as described in CPIP 2.1, "Dose Assessment and Protective Action Recommendations".

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 12 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE DEC 22 1992

- a) Information contained on the Radiological Data forms should be reviewed by the Radiological and EOF Manager and the Radiological Assessment Coordinator, as prescribed in CPIP 2.1.
- b) Records of such communications shall be provided to the Event Historian at periodic intervals.

4.4.7 Information regarding plant status, offsite radiological data, protective action recommendations, and response actions underway shall be provided on a periodic basis to Linn and Benton Counties, the State of Iowa, FEMA and the NRC.

- a) Information to be officially transmitted shall be recorded on Attachment 5, "Emergency Action Level Notification Form" from CPIP 1.2 by the Radiological and EOF Manager as directed by the ER&RD.
- b) This information will be transmitted via the Admin Hotline with a confirming copy being sent to the counties, State and TSC via fax.
- c) Sufficient copies of the forms shall be made for distribution to the Iowa Electric Spokesperson and Corporate Management Representative for their use in briefing the media and other officials.
- d) The Radiological and EOF Manager will normally provide this information to the county, State and Federal representatives in the EOF who are then responsible for subsequent transmittal to their respective EOCs.

NOTE

The Radiological and EOF Manager shall ensure that the representatives in the EOF are aware of this responsibility.

- d) Original copies of such communications shall be provided to the Event Historian for filing.

4.4.8 The Event Historian shall maintain all original copies of the following forms and logs for record purposes:

- a) Plant Status
- b) Status Update Message
- c) Emergency Status Chart
- d) Offsite Radiological Status

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 13 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

- e) EOF log
- f) Offsite Agency Notification Form
- g) Radiological Data Forms

NOTE

Additional logs, graphs, computer runs, etc., shall be collected by the Event Historian to be catalogued and maintained for record purposes.

4.5 Corporate Emergency Response Operations

4.5.1 Principal functions to be provided by Corporate Emergency Response Organization personnel located in the EOF include:

- a) Overall management of the emergency response activities.
- b) Setup primary communications with offsite agencies, support organizations and industry groups.
- c) Engineering activities in support of those being accomplished at the site in the TSC and coordination/management of contract organizations which may be called upon for engineering support.
- d) Conduct of offsite radiological monitoring activities, performance of dose projection, and development of protective action recommendations.
- e) Development of technical information for use by the Iowa Electric Spokesperson during media briefings and for preparing press releases.
- f) Develop and implement recovery and re-entry actions.

4.5.2 The principal functions of the NRC Site Team related to Iowa Electric emergency response activities include:

- a) Monitor the licensee to assure appropriate protective action is being taken with respect to offsite recommendations.
- b) Support the licensee (technical analysis and logistic support).
- c) Support offsite authorities, including confirming the licensee's protective action recommendation to offsite authorities.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 14 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

- d) Keep other federal agencies and entities informed of the status of the incident.
- e) Keep the media informed of the NRC's knowledge of the status of the incident, including coordination with other public affairs groups.

4.5.3 The Site Team Leader may be authorized by the NRC Emergency Director (Chairman of the NRC) to function as the Director of Site Operations (DSO). In that capacity, the DSO has:

- a) Authority to represent the entire NRC by:
 - 1) Acting as the primary spokesperson for the NRC in responding to the media
 - 2) Supervising all NRC personnel at the site.
 - 3) Representing the NRC in interactions with other Federal agencies.
- b) Responsibility for all on site technical aspects of the Federal response.
- c) Authority to recommend protective measures for the public health and safety in coordination with Iowa Electric.
- d) Authority to recommend actions to key State and local agencies in addition to those of Iowa Electric.
- e) Authority to direct the licensee to take specific actions but, only in rare and unusual circumstances (NRC decision maker is convinced that licensee is not taking an action to protect the public health and safety).

NOTE

Authorization for the Director of Site Operations to implement items c, d, and e must be specifically granted by the NRC Emergency Director.

4.5.4 Upon activation of the EOF, the ER&RD shall make all final determinations with respect to event reclassifications.

- a) Generally, recommendations for event escalation or de-escalation will be provided by the Emergency Coordinator or Radiological and EOF Manager using the Emergency Action Level Flowchart, Attachment 1.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12 PAGE 15 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

- b) Time permitting, the ER&RD should advise the NRC Director of Site Operations of his intent to reclassify the emergency prior to doing so.
- c) Additionally, the Radiological and EOF Manager shall contact the Linn and Benton County Civil Defense/Emergency Management Offices and the Administrator, Disaster Services Division, Iowa Department of Public Defense and advise them of such reclassifications.
- d) In addition to the verbal report, Attachment 5, "Emergency Action Level Notification Form" from CPIP 1.2 shall be completed, copied and distributed, as specified in paragraph 4.4.7.
- e) For all events classified as an ALERT or greater, a written report shall be developed by the Technical Recorder for distribution to all offsite EOCs within 8 hours of de-escalation (per NUREG-0654, Appendix 1) which summarizes the event, response actions taken and in progress by Iowa Electric and significant problems which require further evaluation before resolution. Also, review ACP 1402.3 (Plant Regulatory Reporting Activities) for other 10 CFR reporting requirements. These reports should be prepared by someone with a licensing background.

4.5.5 When significant changes in plant status occur or when new information relevant to response actions being taken at the site or at offsite locations become known, the ER&RD should ensure that such information is disseminated to all personnel in the EOF, the Iowa Electric Spokesperson, and Corporate Management Representative.

- a) In addition to displaying such information on status report overheads, briefings should be conducted in the EOF to ensure that all personnel are aware of such changes, using the EOF PA system.
- b) The Emergency Coordinator should be advised of actions underway and decisions being made in the EOF and at offsite Emergency Operations Centers.

4.5.6 Engineering activities including the means to be used to coordinate engineering support for the site and coordination/management of technical activities by offsite contract firms is described in CPIP 2.2.

- a) Engineering, licensing and fuels related activities shall be conducted as directed by the Technical and Engineering Support Supervisor.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 16 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

- b) Unless directed otherwise by the ER&RD, such activities conducted in support of the emergency shall be performed in accordance with standard practices defined by the QA Plan and further described in related administrative and project procedures. Any deviation from normal QA procedures will be documented in accordance with CPIP 2.4.

4.5.7 Dose projections, radiological monitoring and dose assessment activities, and development of protective action recommendations shall be conducted, as described in CPIP 2.1.

- a) Such offsite environmental radiological activities shall be conducted under the overall direction of the Radiological Assessment Coordinator as developed by the Rad and EOF Manager.
- b) As directed by the Radiological and EOF Manager, the Radiological Assessment Coordinator shall be responsible for coordinating offsite Iowa Electric activities with those being accomplished by local, State and Federal support organizations.

NOTE

The development of protective action recommendations shall be the responsibility of the Radiological and EOF Manager who shall obtain the concurrence of the ER&RD before transmittal to offsite officials.

4.5.8 The ER&RD shall provide support to the Iowa Electric Spokesperson in developing media briefings and press release information.

- a) The ER&RD is responsible for ensuring that technical information regarding the event and that response actions being taken by Iowa Electric at the site and by support personnel in the EOF is accurately reflected in press releases and presented correctly at media briefings.
- b) The ER&RD or his designee should be available at all media briefings and be responsible for responding to technical and response activity questions that may be asked.
- c) Further details regarding the conduct of media briefings and development of press releases is contained in CPIP 1.4, "Release of Emergency-Related Information".

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 17 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

4.5.9 The Support Services Coordinator shall be responsible for ensuring that necessary logistics, administrative, procurement and manpower services are provided to support emergency activities being conducted at the DAEC and by Iowa Electric personnel at offsite locations.

- a) The Support Services Coordinator shall maintain contact with the Security and Support Supervisor in the TSC and Emergency Support Manager in the EOF to ensure that required needs are being addressed and promptly executed.
- b) Further details regarding such support functions are provided in CPIP 2.3, "Administrative Services and Corporate Response Coordination".

4.5.10 The ER&RD in conjunction with the Emergency Coordinator should periodically assess the adequacy of response actions being taken by both the DAEC and Corporate Emergency Response Organizations.

- a) Where functional support capabilities can be improved by additional manpower and/or equipment and/or where additional technical or craft support is required, the Emergency Support Manager should be directed to coordinate with the respective functional supervisor and obtain the resources which are required.
- b) Where response emphasis needs to be redirected due to the type of event, event phase, or other extenuating conditions, the ER&RD should take action to modify the structure of either or both the DAEC and Corporate Emergency Response Organizations to enable adequate response.

4.5.11 Where response actions will be required over a protracted period of time, the ER&RD shall ensure that provisions are made for continuous coverage of required functions at all response center locations.

4.5.12 As plant conditions begin to stabilize and reclassification of the emergency to a lower classification is being conducted, the ER&RD shall initiate action to develop a recovery plan.

- a) Recovery planning shall be accomplished as specified in CPIP 6.1, "Recovery and Reentry".
- b) Deactivation of both the DAEC and Corporate Emergency Response Organizations in part or in whole and transition to the normal operational organization or to a recovery organization shall be specified as part of the recovery plan.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 18 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

5.0 REFERENCES

- 5.1 DAEC Emergency Plan
- 5.2 Iowa Electric Light and Power Company Corporate Emergency Response Plan
- 5.3 NUREG 0654, Rev. 1
- 5.4 NRC Region III, Emergency Response Plan and Implementing Procedures
- 5.5 CPIP 1.2, "Corporate Notification"
- 5.6 CPIP 2.3, "Administrative Services and Corporate Response Coordination"
- 5.7 CPIP 1.4, "Release of Emergency-Related Information"
- 5.8 CPIP 2.1, "Dose Assessment and Protective Action Recommendations"
- 5.9 CPIP 2.2, "Technical Support Coordination"
- 5.10 CPIP 6.1, "Recovery and Reentry"
- 5.11 NRC Incident Response Plan (NUREG 0845)
- 5.12 ACP 1402.3 Plant Regulatory Reporting Activities

6.0 ATTACHMENTS

- 6.1 Attachment 1, "Emergency Action Level Flowchart"
- 6.2 Attachment 2, "EOF Working Area Layout"
- 6.3 Attachment 3, "Corporate Emergency Response Organization"
- 6.4 Attachment 4, "EOF Log Sheet"
- 6.5 Attachment 5, "Log Entry Topics"

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 19 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

- 6.6 Attachment 6, "Status Update Message"
- 6.7 Attachment 7, "NRC-HPN Communicator Checklist"
- 6.8 Attachment 8, "NRC-ENS Communicator Checklist"
- 6.9 Attachment 9, "Event Notification Worksheet"
- 6.10 Attachment 10, "Technical Recorder Checklist"
- 6.11 Attachment 11, "NRC Roles During a Nuclear Power Plant Emergency"
- 6.12 Attachment 12, "Typical Organization of the NRC Site Team"

Approved by: Lisa Henderson Date: 12-17-92
Supervisor, Emergency Planning

Approved by: Fauz Date: 12/17/92
Manager, Emergency Planning

Approved by: J.P. Ghosh Date: 12/18/92
Manager, Engineering

Approved by: V. R. ... Date: 12/18/92
Manager, Quality Assurance

Approved by: John ... Date: 12/18/92
Vice President, Nuclear Division

CORPORATE PLAN IMPLEMENTING PROCEDURE

No. CPIP - 1.3

Rev. 12

ACTIVATION AND OPERATION OF THE EOF

PAGE 20 of 41

EFFECTIVE DATE: DEC 2 2 1992

Page 1 of 4

ATTACHMENT 1

"EALS" - RADIOLOGICAL EMERGENCY ACTION LEVEL (EAL)

CONDITION	UNUSUAL EVENT (A)	ALERT (B)	SITE EMERGENCY (C)	GENERAL EMERGENCY (D)
CATEGORY: EOP ENVT 1, 2 & 4 COMMON AREA OR LOCA	Unusual events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occur.	Events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant. Any release expected to be limited to small fractions of the EPA Protection Action Guideline exposure levels.	Events are in progress or have occurred which involve actual or potential substantial core degradation or melting with potential for loss of containment integrity. Releases are not recoverable excepted to exceed EPA Protective Guideline levels except near site boundary immediate area.	Events are in progress or have occurred which involve actual or potential substantial core degradation or melting with potential for loss of containment integrity. Releases are not recoverable excepted to exceed EPA Protective Guideline levels except near site boundary immediate area.
	A 1 Reactor Coolant System leak rate greater than 50 gpm but not greater than 100 gpm.	B 1 Reactor Coolant System leak rate greater than 50 gpm but without makeup capacity. RPS level being maintained.	C 1 LOCA greater than makeup capacity.	D 1 LOCA with failure of ECCS to perform, leading to core degradation or melt in minutes or hours. Loss of containment integrity may be imminent.
	A 6 Steam generator safety or relief valve stuck open safety or relief valve.	B 2 Main Steam Line break with MSIV malfunction causing leakage.	C 2 Main Steam Line break without isolation.	
	A 10 Any combination of safety system failures that initiate shutdown by Tech Specs.			
	A 7 Any ECCS activation that results in water being discharged to the reactor vessel that is either not expected or is required to recover and maintain RPS water level.	B 7 Complete loss of any function needed for plant shutdown.	C 6 Complete loss of any function needed for plant hot shutdown.	D 6 Shutdown occurs but requires decay heat removal systems (e.g. RHRS) or non-safety systems for heat removal. Core degradation or melt occurs in about 10 hours with subsequent containment failure.
		B 11 Failure of RPS to initiate and complete a reactor shutdown.		D 7 Transition phase failure of required core shutdown systems could have for core melt in several hours with containment failure likely. More severe consequences if pump trip does not function.
		B 23 Evacuation of the Control Room required for any reason. Shutdown outside Control Room initiated.	C 9 Transition requiring operation of shutdown systems with failure to occur (continued power generation, but no core damage is immediately evident).	
			C 22 Evacuation of the Control Room required for any reason. Shutdown outside Control Room not implemented within 15 minutes.	
RPS FAB USE				
CONTROL ROOM EVACUATION				

CORPORATE PLAN IMPLEMENTING PROCEDURE

ACTIVATION AND OPERATION OF THE EOP

Page 2 of 4

"EALS" - RADIOLOGICAL EMERGENCY ACTION LEVEL (EAL)

CONDITION	UNUSUAL EVENT (A)	ALERT (B)	SITE EMERGENCY (C)	GENERAL EMERGENCY (D)
CONTAINMENT	A 11 Loss of primary containment integrity requiring initiation of plant shutdown.			D 2 LOCA, containment performance is unsatisfactory, effecting larger term increase of the ECCS. Could lead to core degradation or melt in several hours without containment boundary.

A 12
Loss of secondary containment integrity requiring initiation of plant shutdown.

A 24
Significant loss of containment or communication capability.

A 23
Isolation or alarm or presence of effluent parameters in the Control Room which require plant shutdown.

A 14
Loss of main power capability.

A 13
Loss of offsite power.

CATEGORY RADIOLOGICAL EOP 1 A B

A 4
Reactor coolant activity greater than 1000 $\mu\text{Ci/gm}$ or 1.2 $\mu\text{Ci/gm}$ of dose equivalent T-131 which requires shutdown.

A 5
Air sparge monitor activity greater than $5 \times 10^5 \mu\text{Ci/gm}$ or an increase of $1 \times 10^5 \mu\text{Ci/gm}$ within a 30 minute time period.

ONSTAFF RAD CONSIDERATIONS

C 21
Loss of all safety related alarm and communication capability, communication with plant transient.

C 8
Total loss of 125 VDC power, restoration not possible within 15 minutes.

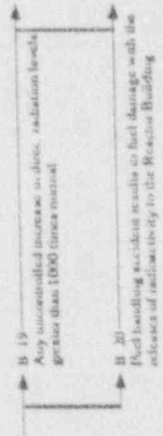
C 7
Loss of all AC power, restoration not possible within 15 minutes.

C 5
Degraded core with possible loss of credible geometry.

D 5
Loss of 2 of the 3 following passive product barriers with potential loss of the shield:
a) RCS Integrity Leak rate greater than 20 gpm
b) Cool Failure See B5
c) Containment Integrity
1) See A11, or
2) Unable to restore and maintain Drywell pressure below 50 psig, or
3) Unable to remove and maintain Drywell temperature below 281°F.

C 18
Major damage to spent fuel in the Reactor Building.

C 19
Uncontrolled decrease in Pool water level below fuel level.



CORPORATE PLAN IMPLEMENTING PROCEDURE

No. CIP - 1.3

Rev. 12

ACTIVATION AND OPERATION OF THE EOF

PAGE 22 of 41

EFFECTIVE DATE: DEC 2 2 1992

Page 3 of 4

ATTACHMENT 1

"EALS" - RADIOLOGICAL EMERGENCY ACTION LEVEL (EAL)

CONDITION	UNUSUAL EVENT (A)	ALERT (B)	SITE EMERGENCY (C)	GENERAL EMERGENCY (D)
OFF-SITE RADIATION CONDITIONS	A. 2 Airborne effluents exceed 10 CFR 20 Appendix B limits.	B. 3 Airborne effluents greater than 10 times 10 CFR 20 Appendix B limits.	C. 3 Measured dose rates and estimated duration of release indicate integrated dose at Site Boundary (or beyond) shall be between 1000 and 5000 mrem.	D. 3 Measured dose rates and estimated duration of release indicate integrated dose at Site Boundary (or beyond) shall be greater than 5000 mrem.
	A. 3 Liquid discharge effluents exceed 10 CFR 20 Appendix B limits.	B. 4 Liquid discharge effluents greater than 10 times 10 CFR 20 Appendix B limits.	C. 4 Calculated dose rates and estimated duration of release indicate integrated dose at Site Boundary (or beyond) shall be between 1000 and 5000 mrem.	D. 4 Calculated dose rates and estimated duration of release indicate integrated dose at Site Boundary (or beyond) shall be greater than 5000 mrem.
			C. 25 High radiological indications (MHIASIS or observed releases) Off-gas stack release rate of 0.3 to 2000 C/gsec.	D. 10 High radiological indications (MHIASIS or observed releases) Off-gas stack release rate of > 2000 C/gsec.
			or Reactor/ turbine building vent release rate of 0.03 to 14 C/gsec.	or Reactor/ turbine Building vent release rate of > 14 C/gsec.
CATEGORY - OTHER EMERGENCIES			or (KAMAN/SPPS readings) Off-gas monitor reading of 0.16 to 1300 pCi/lc	or (KAMAN/SPPS readings) Off-gas monitor reading of > 1300 pCi/lc.
			or Reactor/ turbine building vent monitor reading of 1.0 x 10 ⁻³ to 0.47 pCi/lc.	or Reactor/ turbine Building vent monitor reading of > 0.47 pCi/lc.
			or Drywell monitor reading of 1660 to 3199 R/hr.	or Drywell monitor reading or > 3200 R/hr.
			or Torus monitor reading of 100 to 199 R/hr.	or Torus monitor reading of > 200 R/hr.
FIRE	A. 8 Loss of fire suppression function initiating plant shutdown.	B. 25 Fire not extinguished in 10 minutes, or an area that could affect safety related equipment.	C. 24 Fire compromising the functions of safety systems.	
	A. 28 Fire within the Protected Area that cannot be extinguished within 10 minutes by the Fire Brigade.			
INERT AND SECURITY	A. 26 Transportation of contaminated material uncontrolled to the hospital.	B. 26 Confirmed ongoing security compromise.		
	A. 27 Security threat, attempted sabotage, or unauthorized forcible entry.		C. 23 Inherent loss of physical control of the plant.	D. 8 Loss of physical control of the plant.

CORPORATE PLAN IMPLEMENTING PROCEDURE

No. CIP - 1.3

Rev. 12

ACTIVATION AND OPERATION OF THE EOF

PAGE 23 of 41

EFFECTIVE DATE: DEC 2 9 1988

Page 4 of 4

ATTACHMENT I "EALS" - RADIOLOGICAL EMERGENCY ACTION LEVEL (EAL)

CONDITION	UNUSUAL EVENT (A)	ALERT (B)	SITE EMERGENCY (C)	GENERAL EMERGENCY (D)
NATURAL DISASTERS	A 15 Earthquake of sufficient magnitude to activate the Seismic Monitoring System	B 12 Earthquake greater than DBE	C 10 Earthquake greater than DBE with plant not in cold shutdown.	
	A 16 Cedar River level greater than 733 as measured at the DBEC Bank Structure	B 13 Cedar River level greater than 737 as measured at the DBEC Bank Structure	C 11 Cedar River level greater than 747 as measured at the DBEC Bank Structure, or loss of flood protection for safety related structures with plant not in cold shutdown.	
	A 17 Cedar River flow less than 100 CFS as measured by the R.R. Ave. Bridge (Cedar Rapids) VSDS Monitoring Station and area less than 127 at Bank Structure. Flow is action 3 daily by Nuclear Licensing	B 14 Cedar River flow less than 500 CFS as measured by the R.R. Ave. Bridge (Cedar Rapids) VSDS Monitoring Station. Monitor daily by Nuclear Licensing	C 12 Cedar River flow less than 130 CFS with plant not in cold shutdown as measured by the R.R. Ave. Bridge (Cedar Rapids) VSDS Monitoring Station. Monitor daily by Nuclear Licensing	
	A 18 Tornado strike	B 15 Tornado strikes facility causing damage	C 14 Tornado which damages safety related systems or structures and compromises the functions of safety systems	
OTHER HAZARDS AND HAZARDOUS MATERIALS	A 19 Human aircraft activity over facility or crash onsite	B 16 Aircraft crash or missile impact which causes facility damage	C 15 Aircraft crash or missile impact which damages safety related systems or structures with plant not in cold shutdown.	
	A 21 Fire or smoke explosion	B 17 Explosion onsite which causes facility damage affecting plant operation	C 16 Explosion onsite which damages safety related structures with plant not in cold shutdown	
	A 20 Train derailment onsite	B 18 Uncontrolled entry of toxic or flammable gases into the facility enclosure	C 17 Uncontrolled entry of toxic or flammable gases into critical plant areas where lack of access to the area constitutes a safety problem which plant not in cold shutdown condition	
	A 22 Toxic release of toxic or flammable gas	B 19 Toxic release causing precipitation	C 18 Toxic release causing precipitation	
	A 23 Other plant conditions exist that warrant increased surveillance or involve other than normal controlled shutdown	B 22 Other plant conditions exist that warrant preliminary activation of ESC and placing EOP and other key personnel on standby	C 21 Other plant conditions exist that warrant activation of emergency control and monitoring systems or a preliminary shutdown to the public near the site	

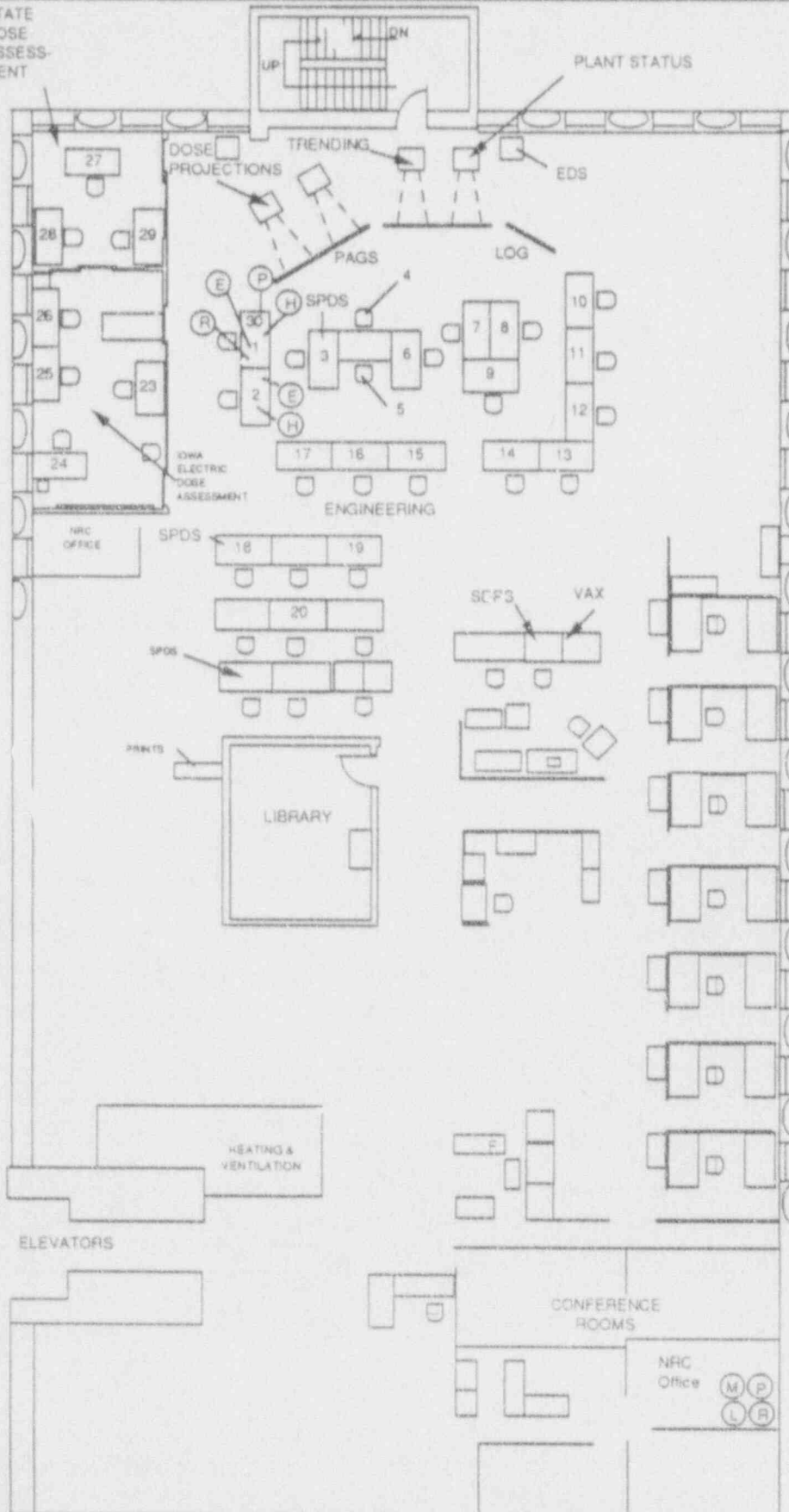
* NOTE: If the level of flow over flow is measured at Cedar Rapids, is misoperation of either the Cedar Rapids Hydro Facility flow area in Waukegan. If a malfunction of the flow area in Waukegan is suspected, the DBEC can be verified by observation of flow flow and height at the Cedar Rapids structure and determination of area flow area in Waukegan.

STATE
DOSE
ASSESS-
MENT

PLANT STATUS

ATTACHMENT 2

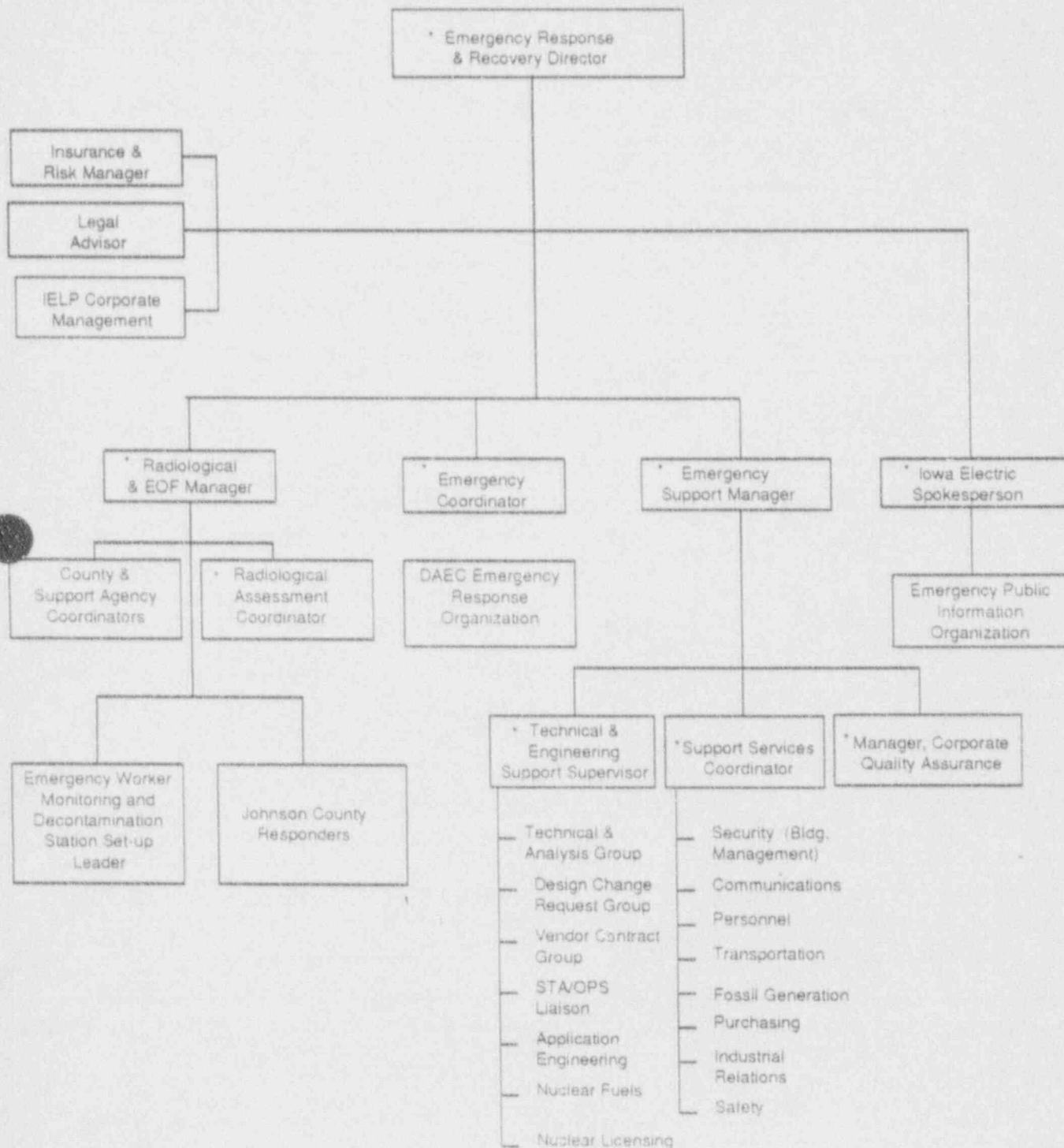
EOF WORKING AREA LAYOUT



1. NRC HPN Communicator
2. NRC ENS Communicator
3. Rad & EOF Mgr.
4. TSC Comm.
5. ER&RD
6. Corp. Mgmt. Rep.
7. Linn County
8. Benton County
9. Iowa DSD
10. Event Historian
11. Support Services Coord.
12. FEMA
13. State Health
14. QA
15. Tech. Recorder
16. Emerg. Support Mgr.
17. Tech & Eng. Support
18. Tech & Analysis Grp Lead
19. Vnd Contact Grp Lead
20. Design Chg Grp Lead
21. Design Drawings
22. Rad Assessment Coord.
23. Field Team Director
24. Rad Data Comm.
25. Computer Operator
26. State Radio Operator
27. State Comp Operator
28. State Dose Assess Dir.
29. NRC
30. ENS Phone
31. HPN Phone
32. RSCL Phone
33. PMR Phone
34. MCL Phone
35. LAN Phone

ATTACHMENT 3

CORPORATE EMERGENCY RESPONSE ORGANIZATION



Note: 24-hour staffed positions.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 27 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

ATTACHMENT 5

LOG ENTRY TOPICS

1. Activation of the EOF and assumption of responsibilities.
2. Event classification and escalation/de-escalation basis.
3. Offsite agency/support organization notifications synopsis of discussion.
4. Protective Action Recommendations provided by Iowa Electric and synopsis of discussion/disposition by local and State officials.
5. Vendor contacts and recommendations provided regarding problems discussed.
6. Significant plant status information.
7. Policy decisions regarding response actions made by the ER&RD and others, as appropriate.
8. Recommendations regarding problem resolution provided personnel in the TSC.
9. NRC communications and synopsis of discussions.
10. Deactivation and reentry decisions.
11. Other information, contacts, etc. pertinent to response actions, offsite agency interface, and management directives or actions which may be important from a historical perspective.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 29 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE DEC 22 1992

ATTACHMENT 7

NRC-HPN COMMUNICATOR CHECKLIST

NOTE

The HPN system is established by the NRC during its standby or initial activation mode of operations after the licensee's TSC/EOF has been activated and is operational. Preliminary information from the licensee (before establishment of the HPN) is provided via the ENS and includes both reactor safety and health physics data. After it has been established, the HPN is the primary means of communicating radiological data (onsite and offsite measurements and dose assessment information) from the licensee to the NRC. The ENS remains the primary means of communicating reactor safety-related information throughout an emergency. Should either or both of the Emergency Communication Subsystems (ENS and HPN) fail, the Operations Center in Bethesda, Maryland, should be informed over normal commercial telephone systems by calling (301) 951-0550 or on one of the following backup numbers: (301) 427-4056, (301) 427-4259, (301) 492-8893.

The following checklist shall be used by the NRC-HPN Communicator as a guide for ensuring that emergency response actions are completed. The column to the left can be used to put either a check when an item is completed or any notation which will be of assistance to the user.

- _____ 1. Report to the EOF. Sign-in on the EOF Staffing Board. Notify the Rad and EOF Manager you are assuming the NRC-HPN Communicator position.
- _____ 2. Ensure the necessary materials are available at your position.
- _____ 3. Communicate (on a regular phone line) with the NRC-HPN Communicator in the TSC to learn what information has already been communicated on the HPN line.
- _____ 4. Obtain the latest Health Physics information (rad data) from the RAC.
- _____ 5. To gain access to the HPN, call the NRC Operations Center in Bethesda, Maryland (on one of the dedicated HPN phones) on one of the following telephone numbers (in the order listed): (301) 951-1212, (301) 951-6000, or (301) 951-0550. These telephone numbers are already provided on the sticker affixed to each HPN telephone.
- _____ 6. When the NRC Operations Center person is on the line, indicate to that person that you are the DAEC/Iowa Electric HPN Communicator and that you would like to be connected to the HPN teleconference bridge.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12 PAGE 30 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

ATTACHMENT 7 (Continued)

NRC-HPN COMMUNICATOR CHECKLIST

- _____ 7. Maintain an open, continuous communication channel with the NRC Operations Center. Communicate information as directed or requested by the NRC or the Rad and EOF Manager.

NOTE

The NRC Event Notification Worksheet (Attachment 9) and the following data should be used as a guideline for information to be communicated:

- a. Protective Measures Status
 - Emergency classification
 - Protective actions recommended by IE
 - Protective actions implemented by offsite authorities.
- b. Offsite Survey Results
 - Whole body dose rates
 - Thyroid dose rates
 - Particulate concentrations
 - Iodine concentrations
- c. Projected Radiation Dose to the Public
 - Duration of the release
 - Projected dose
 - Size of population affected
 - Evacuation time estimate
- d. Meteorology
 - Stability class
 - Wind Speed
 - Wind Direction (from)
 - Precipitation
 - Forecast

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 31 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE DEC 22 1992

ATTACHMENT 7 (Continued)

NRC-HPN COMMUNICATOR CHECKLIST

- e. Health Physics Status
 - Release information
- f. Personnel Status
 - Overexposure
 - Personnel contamination
 - Injuries
 - Accountability
- g. Onsite evacuation status
 - Area
 - Status
 - Time Evacuated
- h. Site Surveys
 - Onsite Team information

- _____ 8. Record on an EOF Log Sheet, or equivalent, any information provided to or received from the NRC.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 32 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

ATTACHMENT 8
NRC-ENS COMMUNICATOR CHECKLIST

NOTE

Preliminary information from the licensee (before establishment of the HPN) is provided via the ENS and includes both reactor safety and health physics data. After it has been established, the HPN is the primary means of communicating radiological data (onsite and offsite measurements and dose assessment information) from the licensee to the NRC. The ENS remains the primary means of communicating reactor safety-related information throughout an emergency. Should either or both of the Emergency Communication Subsystems (ENS and HPN) fail, the Operations Center in Bethesda, Maryland, should be so informed over normal commercial telephone systems by calling (301) 951-0550 or on one of the following backup numbers: (301) 427-4056, (301) 427-4259, (301) 492-8893.

The following checklist shall be used by the NRC-ENS Communicator as a guide in ensuring that emergency response actions are completed. The column to the left can be used to put either a check when an item has been completed or any notation which will be of assistance to the user.

- _____ 1. Sign-in on the EOF Status Board.
- _____ 2. Report to the Rad and EOF Manager that you are assuming the NRC-ENS Communicator position.
- _____ 3. Ensure the necessary materials are available at your position.
- _____ 4. Obtain copies of the "Event Notification Worksheet". (See Attachment 9.)
- _____ 5. Communicate with the NRC-ENS Communicator (via regular phone line) in the TSC to learn what information has already been communicated on the ENS line.
- _____ 6. To gain access to the ENS call the NRC Operations Center in Bethesda, Maryland (on one of the dedicated FTS-2000 ENS phones) on one of the following telephone numbers (in the order listed): (301) 951-0550, (301) 427-4056, (301) 427-4259 or (301) 492-8893. These telephone numbers are already provided on the sticker affixed to each ENS telephone.
- _____ 7. When the NRC Operations Center person is on the line, indicate to that person that you are the DAEC/ Iowa Electric EOF ENS Communicator and that you would like to be connected to the ENS Telconference bridge.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 33 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

ATTACHMENT 8 (Continued)
NRC-ENS COMMUNICATOR CHECKLIST

- _____ 8. Maintain an open, continuous communication channel with the NRC Operations Center. Communicate information as directed or requested by the NRC or the Rad and EOF Manager.

NOTE

The NRC event notification worksheet may be used as a guideline or the following data may be communicated at a minimum:

- a. Event Classification
 - b. Event Description
 - c. Reactor Coolant System leaks
 - 1. Location of leak
 - 2. Leak rate
 - d. List of safety-related equipment not operational
- _____ 8. Record on an EOF Log Sheet, or equivalent, any information provided to or received from the NRC.

CORPORATE PLAN IMPLEMENTING PROCEDURE

No. CPIP - 1.3 Rev. 12

PAGE 34 of 41

ACTIVATION AND OPERATION OF THE EOF

EFFECTIVE DATE: DEC 22 1992

ATTACHMENT 9
TYPICAL

NRC FORM 361 8-88		U.S. NUCLEAR REGULATORY COMMISSION OPERATIONS CENTER			
EVENT NOTIFICATION WORKSHEET					
NOTIFICATION TIME	FACILITY OR ORGANIZATION	UNIT	CALLER'S NAME	CALL BACK # ENS	
				or 1 - - - - -	
EVENT TIME & ZONE	EVENT DATE	1-Hr Non-Emergency 10 CFR 50.72(b)(1)		(iv) Loss of Offsite Comm.	AECS
		(i)(A) TS Required S/D	ASHU	(vi) Fire	AFIR
		(i)(B) TS Deviation	AOEV	(vii) Toxic Gas	ACHE
POWER/MODE BEFORE	POWER/MODE AFTER	(ii) Degraded Condition	ADEG	(viii) Rad Release	ARAD
		(i)(A) Unanalyzed Condition	AUNA	(ix) On-Holding Safe Qs	AMHN
EVENT CLASSIFICATIONS		(iii)(B) Outside Design Basis	AOUT	4-Hr Non-Emergency 10 CFR 50.72(b)(2)	
		(i)(A)(1) Not Covered by OPRs	ACNC	(i) Degraded White S/D	ADAS
GENERAL EMF AGENCY	GEN/AAC	(ii) Earthquake	ANEA	(ii) RPS Actuation (scram)	ARPS
SITE AREA (M) AGENCY	SIT/AAC	(iii) Flood	ANFL	(iii) ESF Actuation	AESF
ALERT	ALZ/AAC	(iii) Hurricane	ANHU	(iii)(A) Safe S/D Capability	AINA
UNUSUAL EVENT	UNU/AAC	(iv) Ice/Hail	ANIC	(iii)(B) RHR Capability	AINB
50.72 NON-EMERGENCY	see next column	(iv) Lightning	ANLI	(iii)(C) Control of Rad Release	AIRC
PHYSICAL SECURITY (TS F1)	DPF1	(v) Tornado	ANTD	(iii)(D) Accident Mitigation	AIND
TRANSPORTATION	WTRA	(vii) On Natural Phenomenon	ANDT	(iv)(A) Air Release > 2X App 6	AAIR
20.402 MATERIAL EXPOSURE	BTFP	(vi) ECCS Discharge to RCS	ACCS	(iv)(B) Lig Release > 2X App 8	ALIQ
OTHER NDAM NLCD NENL NINF NLTR NONR		(vi) Lost ENS	AENS	(v) Offsite Medical	AMED
CODE FROM EIRR OCON		(vii) Loss Emerg. Assessment	AARC	(vi) Offsite Notification	APRE

DESCRIPTION

SAMPLE

Include: Systems affected; actuations & their initiating signals; causes; effect of event on plant; actions taken or planned, etc.

NOTIFICATIONS	YES	NO	WILL BE	ANYTHING UNUSUAL OR NOT UNDERSTOOD?	YES (Explain above)	NO
NRC RESIDENT						
STATE(s)				DO ALL SYSTEMS FUNCTION AS REQUIRED?	YES	NO (Explain above)
LOCAL						
OTHER GOV AGENCIES				MODE OF OPERATION	ESTIMATE FOR	ADDITIONAL INFO
USCIB BRCC RELEASE				UNTIL CORRECTED	RESTART DATE	ON BACK?

CORPORATE PLAN IMPLEMENTING PROCEDURE

No. CPIP - 1.3 Rev. 12

PAGE 35 of 41

ACTIVATION AND OPERATION OF THE EOF

EFFECTIVE DATE: DEC 22 1992

ATTACHMENT 9 (Continued)
TYPICAL

RADIOLOGICAL RELEASES CHECK OR FILL IN APPLICABLE ITEMS (Specific details/explanations should be covered in event description)	
LIQUID RELEASE	GASEOUS RELEASE
MONITORED	UNMONITORED
UNPLANNED RELEASE	PLANNED RELEASE
OFFSITE RELEASE	T.S. EXCEEDED
PERSONNEL EXPOSED OR CONTAMINATED	OFFSITE PROTECTIVE ACTIONS RECOMMENDED
*State release path in description	
Release Rate (Ci/sec)	N.T.S. LIMIT
MOORE GAS	HOO GUIDE
Iodine	Total Activity (Ci)
Particulate	N.T.S. LIMIT
MOORE GAS (excluding tritium & dissolved MOORE gases)	HOO GUIDE
Liquid (tritium)	10 uCi/sec
Total Activity	10 uCi/min
	0.2 Ci/min
	5 Ci
PLANT STACK	CONDENSER/AIR EJECTOR
	MAIN STEAM LINE
	SG BLOWDOWN
	OTHER
RAD MONITOR READINGS	
ALARM SETPOINTS	
N.T.S. LIMIT (if applicable)	
RCS OR SG TUBE LEAKS CHECK OR FILL IN APPLICABLE ITEMS (Specific details/explanations should be covered in event description)	
LOCATION OF THE LEAK (e.g., SG #, valve, pipe, etc.)	
LEAK RATE	UNITS (cm ³ /hr)
	T.S. LIMITS
	SUDDEN OR LONG TERM DEVELOPMENT
LEAK START DATE	TIME
	COOLANT ACTIVITY & UNITS PRIMARY
	SECONDARY
LIST OF SAFETY RELATED EQUIPMENT NOT OPERATIONAL	
EVENT DESCRIPTION (Continued from page)	

SAMPLE

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 36 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

ATTACHMENT 10
TECHNICAL RECORDER CHECKLIST

The following checklist shall be used by the Technical Recorder as a guide to ensure that necessary actions are completed. The blanks at the left can be used to put either a check when the action is completed or any notes which may assist the user.

- _____ 1. Report to the EOF. Sign-in on the EOF Staffing Board. Notify the Rad and EOF Manager that you are assuming the position.
- _____ 2. Establish and maintain communications with:
 - Technical Liaison in the ENC
 - IE Tech Liaison in the Linn County EOC
 - IE Tech Liaison in the Benton County EOC
 - IE Tech Liaison in the State EOC

NOTE

Communications with the above should be accomplished via a five party conference call. Use the "Repertory Dialing" and "Conference Call" Instructions in the Rolmphone Quick Reference Guide. A headset should be used.

- _____ 3. Continually communicate pertinent information and answer questions on event status, including, but not limited to:
 - Emergency Classification and EALs
 - Current plant and equipment status
 - Recommended and approved Protective Actions
 - Evacuation and sheltering status
 - Expected and measured onsite and offsite releases
 - Actions being taken to mitigate the emergency
 - Meteorological conditions
 - Personnel injuries and contaminations and their treatments
- _____ 4. Keep a log of significant events and activities
- _____ 5. Collect communication forms and status reports distributed during the event. These are to be used to answer questions during and after the event.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 37 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 2 8 1992

ATTACHMENT 10 (Continued)
TYPICAL
TECHNICAL RECORDER CHECKLIST

- _____ 6. Develop a written report for distribution to all offsite EOCs within 8 hours of de-escalation. (Per Section 4.5.4)
- _____ 7. Review ACP 1402.3 for other 10 CFR reporting requirements (per Section 4.5.4).

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 38 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

ATTACHMENT 11
NRC ROLES DURING A NUCLEAR POWER PLANT EMERGENCY

Monitoring-Only Role

In this role, NRC response is essentially passive and confined to information acquisition and assessment. The licensee, in conjunction with State and local authorities, has primary responsibility for dealing with the incident. NRC keeps itself apprised of the situation and the status of response actions, based on data supplied by the licensee as well as any data obtained independent of the licensee via a data system, reported by NRC personnel on site, or provided by offsite authorities. NRC also maintains cognizance of offsite conditions and activities related to the incident. Additional ad hoc information may be requested by NRC, as deemed necessary. Data from all sources is collated, verified, analyzed, and evaluated by the NRC so that it may independently estimate the situation and the adequacy of the operational protective measures being taken. NRC in its role as Lead Federal Agency (LFA), serves as the focal point at the Federal level for providing authoritative technical information on the incident related to the onsite situation and licensee offsite activities.

The monitor role is exercised by NRC headquarters and the Director of Site Operations throughout the course of an incident. However, once the Director of Site Operations has arrived and assumed control at the site, the Director of Site Operations becomes the primary contact with the licensee, State and local authorities.

Inform Role

On the basis of the monitoring role, the NRC may find it appropriate to inform affected officials and the public about the status of the emergency. This role would be exercised only when it is clear that responsible parties are not aware of pertinent information or when information is specifically requested by other interested parties (e.g., news media, Congress, White House). Primary interaction with the news media will transfer from the headquarters executive team to the Director of Site Operations when the Director of Site Operations assumes control.

Advisory Role

The NRC role in this case is expanded to include exerting influence on the response process, using information gathered by continued monitoring. However, primary responsibility for coping with the incident still resides with the licensee. NRC gives advisory support to assist in diagnosing the situation, isolating critical problems, and determining what remedial courses of action and additional precautionary measures are indicated. Advice is made available to the licensee, State and local authorities, and to other Federal agencies concerned.

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 39 of 41
ACTIVATION AND OPERATION OF THE EGF	EFFECTIVE DATE: DEC 2 1992

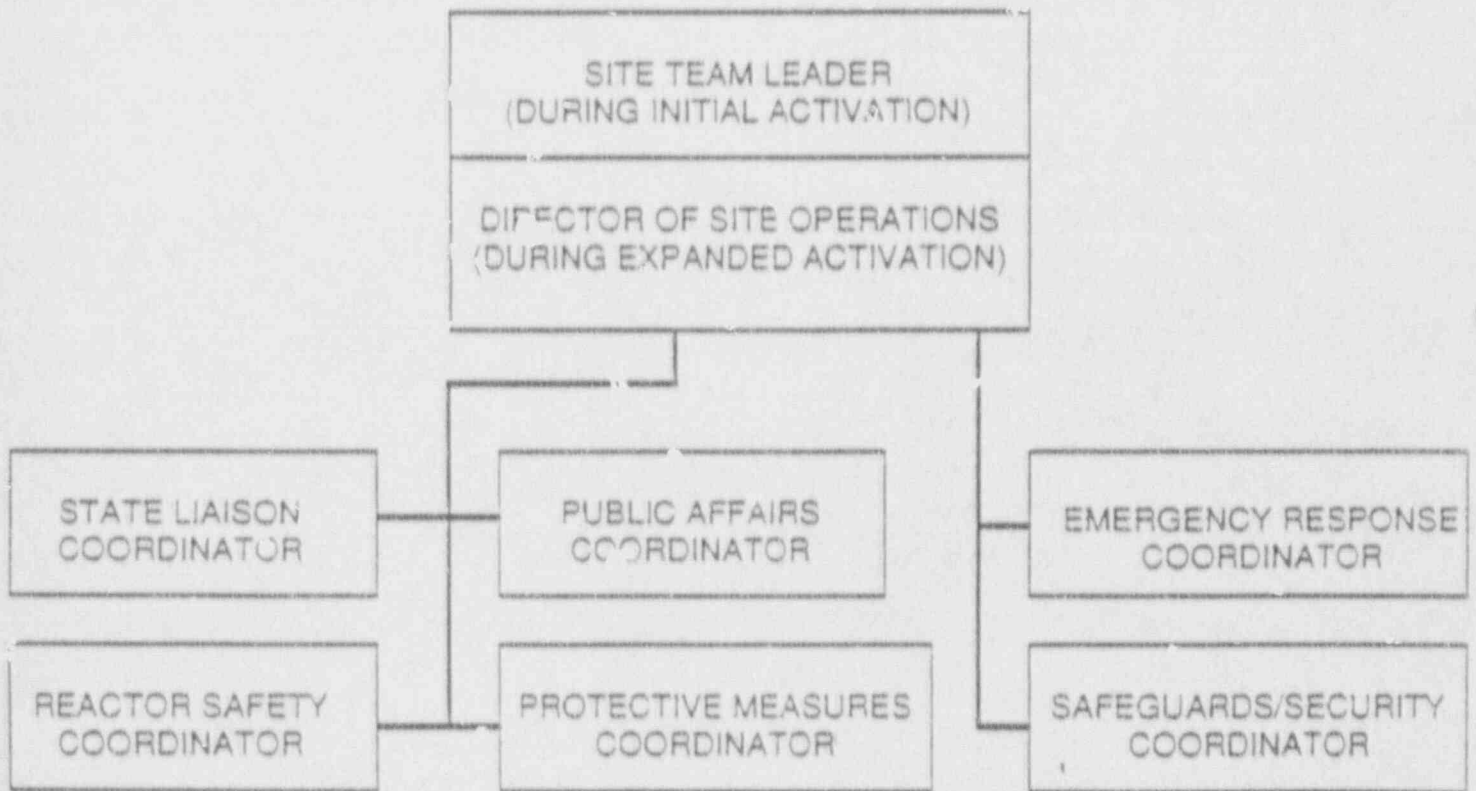
ATTACHMENT 11 (Continued)
NRC ROLES DURING A NUCLEAR POWER PLANT EMERGENCY

In coordination with FEMA, NRC will advise State and local authorities on actions to mitigate the consequences of the incident and for protecting the public. This advice may confirm the licensee's recommendation or provide additional recommendation.

In addition, in selected cases and by request, the NRC may assist the licensee by obtaining onsite and external support relating directly to onsite response needs. In this capacity, NRC may serve as an intermediary for the licensee and various other participants involved.

Limited Direction Role

In addition to monitoring and advisory activities, in some unusual and very rare situations, the NRC could find it necessary to intervene in a limited fashion to direct the licensee's onsite response. It is not expected that the NRC will be required to assume this role, but plans must be made for such a contingency. In such an unlikely event, the NRC would issue formal orders to the licensee to take certain measures and then monitor implementation of the actions ordered. In this role, the licensee continues to make other key operational decisions and to operate and manage the facility with licensee personnel. NRC advice and direction would be channeled to licensee management. Although it is the Director at headquarters that has the authority to issue orders and directives to the licensee, this authority most likely will not be delegated to the Director of Site Operations after one is designated.

ATTACHMENT 12**TYPICAL ORGANIZATION OF THE NRC SITE TEAM
(LEAD DURING EXPANDED ACTIVATION)**

CORPORATE PLAN IMPLEMENTING PROCEDURE	No. CPIP - 1.3 Rev. 12
	PAGE 41 of 41
ACTIVATION AND OPERATION OF THE EOF	EFFECTIVE DATE: DEC 22 1992

ATTACHMENT 12 (Continued)
NRC EXPANDED ACTIVATION SITE TEAM
(BY LOCATION)

Emergency Operations Facility	27*
Technical Support Center	8
Control Room	2
Emergency News Center (JPIC)	3
Federal Response Center (FRC)	2
State Emergency Operations Center	2
State Forward Emergency Operations Center	2
Operations Support Center	1
Radiation Controlled Area	1
Federal Radiological Monitoring and Assessment Center (FRMAC)	7
<hr/>	
Total	55

* Approximately 9 of these people will operate in the EOF working area, with the remainder working out of an area adjacent to the EOF working area.