NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Prairie Island Unit 1	0 5 0 0 0 2 8 2	1 OF 0 2

TITLE (4)

Reactor Trip During Restart

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 5	0 9	8 5	8 5	— 0 1 0	— 0 0	0 6	1 0	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 5	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Arne Hunstad, Staff Engineer	6 1 2 3 8 8 - 1 1 2 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While switching feedwater supplies during startup, a main feedwater pump discharge valve did not open on demand. The reactor tripped on low steam generator level. The valve was opened and restart was completed successfully.

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NRC Form 366A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Prairie Island Unit 1	0 5 0 0 0 2 8 2	8 5	— 0 1 0	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On May 9, 1985 during Unit 1 startup with reactor power at 5%, the reactor tripped on low steam generator level.

During feedwater supply transfer from the auxiliary feedwater supply (BA) to the main feedwater (SJ) the main feedwater pump (P) was started and the pump discharge valve (SHV) open signal given. The valve's motor operator began to turn, and the "not closed" limit switch actuated. Dual valve position lights indicated to the operator that the valve had begun to open. The motor operator had, in fact, tripped on high torque with the valve still fully closed.

When the main feedwater bypass valves (FCV) were opened, steam was being released from the steam generators (SG). The resulting swell, increase, in steam generator level was interpreted as an indication that main feedwater was being supplied, so auxiliary feedwater was terminated. The reactor tripped at 1421 hours on low steam generator level.

The main feedwater pump discharge valve was opened and restart was completed successfully. This event had no effect on public health or safety.

This report will be circulated for training among operations personnel.

Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401
Telephone (612) 330-5500

June 10, 1985

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Reactor Trip During Restart

The Licensee Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72 on May 9, 1985.

Monica Vik
for David Musolf
Manager - Nuclear Support Services

DMM/EFE/dab

c: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
MPCA
Attn: J W Ferman

Attachment

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