

AVERAGE DAILY UNIT POWER LEVEL

Completed by J. P. Ronafalvy

Docket No. 50-311
 Unit Name Salem # 2
 Date June 10, 1985
 Telephone 609-935-6000
 Extension 4455

Month May 1985

Day Average Daily Power Level
 (MWe-NET)

1	<u>505</u>
2	<u>214</u>
3	<u>0</u>
4	<u>71</u>
5	<u>811</u>
6	<u>1073</u>
7	<u>1100</u>
8	<u>1089</u>
9	<u>1087</u>
10	<u>771</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>454</u>
16	<u>1100</u>

Day Average Daily Power Level
 (MWe-NET)

17	<u>1102</u>
18	<u>1099</u>
19	<u>1106</u>
20	<u>1107</u>
21	<u>1100</u>
22	<u>1077</u>
23	<u>1075</u>
24	<u>1070</u>
25	<u>1104</u>
26	<u>1104</u>
27	<u>1104</u>
28	<u>1080</u>
29	<u>1088</u>
30	<u>1099</u>
31	<u>1109</u>

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OPERATING DATA REPORT

Docket No. 50-311
 Date June 10, 1985
 Telephone 935-6000
 Extension 4455

Completed by J. P. Ronafalvy

Operating Status

1. Unit Name	Salem No. 2	Notes
2. Reporting Period	May 1985	
3. Licensed Thermal Power (MWt)	3411	
4. Nameplate Rating (Gross MWe)	1170	
5. Design Electrical Rating (Net MWe)	1115	
6. Maximum Dependable Capacity (Gross MWe)	1149	
7. Maximum Dependable Capacity (Net MWe)	1106	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason	N/A	
9. Power Level to Which Restricted, if any (Net MWe)	N/A	
10. Reasons for Restrictions, if any	N/A	
	<u>This Month</u>	<u>Year to Date</u> <u>Cumulative</u>
11. Hours in Reporting Period	744	3623 31848
12. No. of Hrs. Reactor was Critical	606.0	1054.6 16149.2
13. Reactor Reserve Shutdown Hrs.	0	0 3533.6
14. Hours Generator On-Line	583.1	827.2 15439.3
15. Unit Reserve Shutdown Hours	0	0 0
16. Gross Thermal Energy Generated (MWH)	1867901	2275719 46002755
17. Gross Elec. Energy Generated (MWH)	620020	730140 15007790
18. Net Elec. Energy Generated (MWH)	588633	655527 14173363
19. Unit Service Factor	78.4	22.8 48.5
20. Unit Availability Factor	78.4	22.8 48.5
21. Unit Capacity Factor (using MDC Net)	71.5	16.4 40.2
22. Unit Capacity Factor (using DER Net)	71.0	16.2 39.9
23. Unit Forced Outage Rate	21.6	74.8 41.7
24. Shutdowns scheduled over next 6 months (type, date and duration of each)	N/A	
25. If shutdown at end of Report Period, Estimated Date of Startup:	N/A	
26. Units in Test Status (Prior to Commercial Operation):		
	<u>Forecast</u>	<u>Achieved</u>
Initial Criticality	6/30/80	8/2/80
Initial Electricity	9/1/80	6/3/81
Commercial Operation	9/24/80	10/13/81

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH May 1985

Docket No. 50-311
Unit Name Salem No.2
Date June 10, 1985
Telephone 609-935-6000
Extension 4455

Completed by J.P. Ronafalvy

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
85-046	4-30	F	49.0	A	5	-	CH	PUMPXX	Feedwater Pump Other Exciter
85-048	5-02	F	55.0	A	3	-	HA	RELAYX	Problems
85-068	5-10	F	105.9	A	3	-	IA	CRDRVE	Control Rod Problem

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of
Previous Outage
5-Load Reduction
9-Other

4 Exhibit G
Instructions
for Preparation
of Data
Entry Sheets
for Licensee
Event Report
(LER) File
(NUREG 0161)

5 Exhibit 1
Salem as
Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH MAY 1985

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: June 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2EC-0651	Service Water	Service water replacement of strainer back flush line with stainless steel piping.
2EC-1015	Component Cooling	Provide motor operators for component cooling water pumps suction header valves 2CC17 and 2CC18.
2EC-1242	Service Water	Provide electric space heating in El. 78 Penetration Area Service Water Valve Rooms to prevent freezing of header overpressure controls.
2EC-1564	Fire Protection	Install smoke detectors in areas designated P2C-1, P2C-3, P2C-4, P2C-5, P2F-1, and P2G-1.
2ET-1647	Equipment Qualification	Add temporary temperature monitoring equipment inside Containment at various locations within the Auxiliary and Control Buildings.
2EC-1660	Chilled Water	Install a 4" isolation gate valve in the chilled water supply header immediately upstream of the 4" CW supply line to 2CH62 and a 4" isolation ball valve in the chilled water return header downstream of the 4" CW return from 2CH76.
2EC-1677	Reactor Coolant	Replace reactor coolant narrow range RTD's with environmentally qualified elements.

* Design Change Request

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2EC-1717	Environmental Qualification	Replace existing transmitter with environmentally qualified Rosemount 1153D Series Model (PA-227, PA-7461) (safety injection pump discharge pressure) and PA-211 (BIT pressure) (PT-0942).
2EC-1757	Overhead Annunciator	Modify delay time from 50 milliseconds to 250 milliseconds for first out annunciator panel.
2EC-1789	Various	Environmentally upgrade numerous limitorque valve motor operators. This upgrading is to be accomplished by the replacement of various motor operator parts with parts which are better suited for use in harsh environmental zones.
2EC-1861	21 C/SI Lube Oil Cooler	Replace existing lube oil on No. 21 C/SI Pump with cooler of similar design characteristics but upgraded materials of construction. Upgraded coolers will have outer tubesheets and tubes of titanium material.
2EC-1894	Main Steam	Add a "Y" type strainer downstream of 2MS55 (just before restricting orifice). The "Y" type strainer holes must be smaller than 0.125" to serve as primary protection of the restricting orifice.
2EC-1917	Reactor Coolant	Replace RTD bypass loop valves 23RC 16, 17, 24, 25 & 28 Mark No. FA-17 with Mark No. FA-131.
2EC-1946	Containment Ventilation	Installation of spring assembly in the backdraft damper mechanism of the containment fan coil units 21, 22, 23, 24 and 25.

* Design Change Request

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2EC-1970	Containment Personnel Hatch	Install caps and valving at personnel hatch E. 100 & 130.
2EC-1975	Steam Generator Blowdown and Drains	Cut out and remove valves 21-24GB7 and replace each valve with a dutchman.
2EC-1989	SGFP and Turbine Lube Control Oil	Change the SGFP No. 21 and 22 thrust bearing wear alarm and trip setpoints from 8 and 12 PSI to 13 and 17 PSI respectively.
2ET-2006	Chilled/Service Water	Test application of Garlock "Turbo-Star" cartirdge type mechanical seal in Chiller Condenser Recirc Pump #23.
2EC-2007	Heater Drain Pump/ Miscellaneous Condensate	Install in-line flow meters, check valves local pressure instrumentation and throttling valves on the stuffing box miscellaneous condensate gland injection lines.
2EC-2010	Main Generator and Auxiliaries	This is the removal DCR for the main generator, excitation, seal oil, stator water and the H2 and CO2 systems.
2EC-2072	Main Steam Spring Hangers	Adjust, in a uniform manner, the cold load setting of nine spring hangers located on the #24 Main Steam Header between the reactor containment wall and the mixing bottle, to a new load setting.
2SC-0364	21B & 22B Circulators	Modify condensate, heater drain and circulating water pump motors to allow filling of oil reservoirs.
2SC-0388A	Moisture Separator Reheater	Material change of 1" piping downstream of the RD95 valves (desuperheating supply to the main steam coil drain tanks). The only piping that needs to be changed is between RD95 valves and the SW orifice.

* Design Change Request

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2SC-0420	Service Water Bay #4	Revise motor service water cooling piping to the service water pump motor coolers which will allow quick removal and restoration of piping hangers and associated piping during motor change out.
2SC-0949	Waste Gas	Add two test connections to the Waste Gas System. One test connection between 2WG6 and 2WL99 on vent header and one test connection between 21 & 22WG27 & 21, 22, 23, 24WG33 valves on the H.P. Header.
2SC-1093	Boric Acid System- Batch Tank	Construct a permanent platform around the #2 Unit boric acid batching tank, similar to the one which presently exists on #1 Unit boric acid batching tank, to replace the temporary scaffolding now in place.
2SC-1168	Reactor Coolant	Change reactor coolant flow transmitter from Fisher Porter model #10B2496FB to new model and type specified by Engineering.
2SC-1172A	Steam Generator Feed & Condensate, Bleed Steam & Heater Drain	Provide a suitable replacement for ED-9 and CN47 can-flex valves. The ED-9 valves are to be relocated as close to the condenser as possible. Can-flex valves replaced by "V" ball.
2SC-1269	Hydrogen Monitoring	Install ladder and platform in Unit No. 2 Containment to gain access to 21 & 22 Hydrogen Monitor System sensor assembly to perform maintenance.
2SC-1386	Heater Drain Pump	Provide gland leak off lines from all three heater drain pumps to the Turbine Building flood sumps.

* Design Change Request

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2SC-1580	Main Steam Safety Valves	Remove the lever #17, the forked lever #22 and the spindle nut #12 on all (20) safety valves.

MAJOR PLANT MODIFICATIONS
REPORT MONTH MAY 1985

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: June 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	10CFR 50.59	SAFETY EVALUATION
2EC-0651	The replacement of the Service Water strainer backwash carbon steel line with a stainless steel 316 material will not cause an accident or malfunction not previously analyzed in the Safety Analysis Report. No unreviewed safety or environmental questions are involved.	
2EC-1015	This change will not affect the function of the valves. The change is an enhancement. The added motor operators are seismically (Class I) and environmentally qualified. The description in the FSAR will be modified. No unreviewed safety or environmental questions are involved.	
2EC-1242	This change entails the installation of space heaters in the Service Water Valve Room. The supports are seismically qualified and the work does not involve the safe shutdown of the reactor. No unreviewed safety questions or environmental questions are involved.	
2EC-1564	This change expands the Fire Detection System. No safety related systems are affected. No unreviewed safety or environmental questions are involved.	
2ET-1647	This DCR installs temporary test equipment to monitor ambient temperature. No unreviewed safety or environmental questions are involved.	
2EC-1660	The new isolation valves in the supply and return Chilled Water headers are safety related, nuclear grade, hydrotested, and leak tested prior to installation. They will ensure system pressure integrity and isolation ability when required. These valves will operate only under the following conditions: <ol style="list-style-type: none">1. Unit 2 Chilled Water System is down2. Supply chilled water to the secondary non-safety related area (such as when cutting the 4" Chilled Water header to allow installation of other DCR valves upstream)	

* Design Change Request

3. To provide cross-tie flow between the Units
Chilled Water Systems

The valves will be locked open once Unit 2 Chilled Water System is on line. There is no impact on the safety injection actuated valves 1CH-150 and 1CH-117. The flow diverted from the secondary area from the cross-tie has been verified by calculation to have no impact on the A/C coil operation since flow must be established to satisfy the A/C coil load of both Units first and then 1CH131 is cracked and throttled to provide any excess flow to the secondary areas. No unreviewed safety or environmental questions are involved.

- 2EC-1677 This change involves the replacement of existing equipment. The system response time will be increased by 2.7 seconds. The overtemperature delta T (Table 3.3.2 in the Tech. Specs.) will increase but by an amount less than that allowed in Table 15.1-3 of the FSAR accident analysis. The proposed 5.0 seconds response time will provide a conservative limit relative to the accident analysis of the UFSAR and will, at the same time, provide sufficient latitude to allow the use of presently available, environmentally qualified RTD's. Since there are no modifications to the plant or the plant procedures, the possibility of a new or different accident from any accident previously evaluated is not created. No unreviewed safety or environmental questions are involved.
- 2EC-1717 This change involves the replacement of transmitters which will have the same function as the old ones. No unreviewed safety or environmental questions are involved.
- 2EC-1757 This modification will assure that the first-out panel alarms latch only when accompanied by a valid trip signal. The system is not safety-related and will not affect the safe shutdown of the plant. No unreviewed safety or environmental questions are involved.
- 2EC-1789 The valve motor operator modifications involves the replacement of the existing components with new components which are environmentally qualified. This DCR will not alter any plant process or discharge and will not affect the existing environmental impact. No unreviewed safety or environmental questions are involved.

- 2EC-1861 This DCR involves the installation of Lube Oil Coolers with upgraded materials. The new titanium tubes will have greater resistance to the corrosive affect of the service water. The cooler shell is constructed of 316 stainless steel. The upgraded coolers are designed to the 1980 edition of the ASME Code - Section III, Class 3 - 1981 Summer Addenda on the tube side (contains service water). The shell side is designed to Division I Section VIII of the 1980 edition of the ASME code. The new coolers were purchased from the same design specification as the original. No unreviewed safety or environmental questions are involved.
- 2EC-1894 The installation of the "Y" type strainer upstream of the existing restricting orifice does not create a new safety hazard. This DCR does not alter the function of the system. Engineering Safety Evaluation S-C-0210-MSE-277 addresses a substitute for a Nuclear Class III Seismic Class I "Y" strainer (which is no longer available). No unreviewed safety or environmental questions are involved.
- 2EC-1917 The replacement of the valves do not alter the ability of the piping system to perform as designed. The replacement valve has been evaluated as an acceptable substitute for the existing valve. The new valve meets or exceeds the seismic, nuclear and pressure class of the existing valves. The flowmeter in the loop will ensure that the required flow is not affected after replacing the valves. No unreviewed safety or environmental questions are involved.
- 2EC-1946 The new spring assembly will provide additional force to assist the damper mechanism in closing and is designed so that it will not prevent the damper from opening fully. Failure of the spring will not jeopardize the operation of the damper. The spring assembly is installed external to the ductwork. A failure of the spring will not cause any mechanisms to jam. All present safety evaluations are valid, and no new safety concerns are introduced. No unreviewed safety or environmental questions are involved.

* Design Change Request

- 2EC-1970 All realistic failure modes for this DCR have been considered but are not applicable. This change meets GDC 56 requirements and complies with safety guide #11. The design criteria of Seismic I and Nuclear Class II are met. No unreviewed safety or environmental questions are involved.
- 2EC-1975 Due to the high blowdown rates used on the Salem Units, the No. 21 Blowdown Tank has not been used, all blowdown is being directed to either the No. 22 Condenser or No. 22 Blowdown Tank. By removing the Nos. 21-24 GB7 valves and blanking off the lines upstream of these valves, there will be no further steam leaks in this area. No unreviewed safety or environmental questions are involved.
- 2EC-1989 The intended function of the system will remain the same. This change is in conformance with manufacturer's recommendation. This is a non-safety related system. No unreviewed safety or environmental questions are involved.
- 2EC-2006 This test does not affect any presently performed safety analysis nor does it create any new hazards. The basis for the Tech. Specs. remains unchanged. This test modification will not alter any plant process or discharge and will not affect the existing plant environmental impact. Also, partial implementation by retrofitting each pump separately does not affect the aforementioned. No unreviewed safety or environmental questions are involved.
- 2EC-2007 This modification does not alter any plant process or discharge and will not affect the existing plant impact. Work is to be performed on El. 100 in the Turbine Building. No unreviewed safety or environmental questions are involved.
- 2EC-2010 This modification does not alter any plant process or discharge and will not affect the existing plant impact. No safety related systems are involved. No unreviewed safety or environmental questions are involved.
- 2EC-2072 The adjustment of the No. 24 Main Steam Header Hangers to a new load setting in no way creates a new safety hazard. There are no changes to the Tech. Specs. or the UFSAR involved. No unreviewed safety or environmental questions are involved.

* Design Change Request

- 2SC-0364 The change in the upper and lower bearing oil fill piping will not affect the operation of the motors. This will not involve any change to the FSAR or the Tech. Specs. No unreviewed safety or environmental questions are involved.
- 2SC-0388A The modifications to the quench lines affect systems that are not safety related. They will not alter any plant process or discharge and will not affect the existing plant impact. No unreviewed safety or environmental questions are involved.
- 2SC-0420 The modifications to the Service Water pump motor bearing coolers do not change the function of the coolers. All supports have been redesigned and re-evaluated as necessary. The change will prevent corrosion and/or erosion of the pipe thereby increasing the reliability of the system. This also reduces the possibility of service water leaking into the environment due to a pipe failure. No unreviewed safety or environmental questions are involved.
- 2SC-0949 This DCR addresses drawing changes only. No modification of any control logic or affect of any installed safety related equipment is involved. No unreviewed safety or environmental questions are involved.
- 2SC-1093 This change eliminates a severe safety hazard of utilizing temporary scaffolding (inadequate support) to get up to the level of the hatch or the batching tank. The new platform is seismically qualified. No previous structural safety analyses are adversely affected. No unreviewed safety or environmental questions are involved.
- 2SC-1168 This change involves the replacement of an existing transmitter. All realistic, potential failure modes have been considered but are not applicable. No unreviewed safety or environmental questions are involved.
- 2SC-1172A The valve replacement involves only non-safety related systems. The modification will not alter any plant process or discharge and will not affect the existing plant impact. No unreviewed safety or environmental questions are involved.

* Design Change Request

- 2SC-1269 The telescoping ladders and platforms to be used conform to ANSI standard A92.3-1980. Seismic restraints are provided to negate the effects that a seismic event would have on the platforms while they are in the stored position. The modification will not alter any plant process or discharge and will not affect the existing plant impact. No unreviewed safety or environmental questions are involved.
- 2SC-1386 The addition of gland leak off lines on the Heater Drain Pumps do not affect any presently performed safety analysis, nor does it create any new hazards. The modification will not alter any plant process or discharge and will not affect the existing plant impact. No unreviewed safety or environmental questions are involved.
- 2SC-1580 The removal of the lever, the forked lever and the spindle nut from the affected valves will not affect the intended operation of the valves. The component removal will not result in a violation of Section III since the pressure setpoints will not be affected. The lifting levers on safety valves have been deleted from ASME Section III Code since 1977, since all utilities (including Salem) have an Inservice Inspection Program to test MSSV once per refueling outage. Also, the environment in a nuclear power plant is much cleaner than that of a fossil power plant which precludes oxidation of the nozzle/disc area preventing sticking or galling. Thus, the need for manual lifting levers is obviated. No unreviewed safety or environmental questions are involved.

* Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 2

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
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8505172345

SMD

2

23 TAC PUMP

FAILURE DESCRIPTION: TRIED TO START 23 TAC PUMP AND BREAKER TRIPPED FREE.
NO FLAGS WERE FOUND. PLEASE INVESTIGATE AND REPAIR.

CORRECTIVE ACTION: REMOVED BREAKER SN#0224A6260-007, INSTALLED BREAKER
SN#0224A269-002. ONLY WORK DONE ON THIS WO WAS TO
INTERCHANGE BREAKERS. SEE WO #8505231465 FOR WO ON
FAILED BREAKER.

8505131720

SIC

2

22BF40

FAILURE DESCRIPTION: VALVE HAS A BLOWN DIAPHRAM. PLEASE REPAIR.

CORRECTIVE ACTION: REPLACED DIAPHRAGM OF CONNECTED REGULATOR AND GAUGE
TO VERIFY PROPER STROKE AND CALIBRATION. VERIFIED
PROPER OPERATION.

8505140605

SIC

2

PRESS. SPRAY VALVE

FAILURE DESCRIPTION: THE VALVE IS LEAKING BY CAUSING PRESSURIZER PRESSURE
TO RESPOND SLOWER THAN NORMAL. PLEASE REPAIR.

CORRECTIVE ACTION: CALIBRATED E/P TRANSDUCER. STROKED VALVE FROM
CONTROL TO VERIFY LIMIT INDICATIONS.

8505120213

SIC

2

SHUTDOWN ROD 2SA1

FAILURE DESCRIPTION: WHILE PULLING SHUTDOWN BANK "A" ROD 2SA1 DID NOT
INDICATE MOVEMENT. PLEASE CHECK AND REPAIR.

CORRECTIVE ACTION: FOUND AND OPEN ON WIRE 1 & 2 ON TB5 IN CAB 22AC,
2SA1 ROD.

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
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8505120019

SIC

2

DORIC POINT 101

FAILURE DESCRIPTION: POINT FOR DISCHARGE OF BORIC ACID FILTER IS FAILED
THIS POINT IS ON VITAL HEAT TRACE SURVEILLANCE.
MAINTENANCE DEPARTMENT PLEASE REPAIR.

CORRECTIVE ACTION: FOUND LEADS LIFT ON SATELLITE FOR DORIC TESTING.
RELANDED LEADS AND VERIFIED CORRECT READING.

8505070712

SMD

2

21 WASTE GAS COMP BREAKER

FAILURE DESCRIPTION: BREAKER TRIPPED, SPRINGS ARE DISCHARGED. PLEASE
INVESTIGATE.

CORRECTIVE ACTION: REPLACED MOTOR COMPLETED M3Z.

8505081013

SIC

2

CONTROL ROD

FAILURE DESCRIPTION: THIS ROD IS OUT OF SPEC. WITH THE REST OF THE BANK.
PLEASE CHECK.

CORRECTIVE ACTION: RECALIBRATED SIGNAL CONDITIONING MODULE AND BAILEY
INDICATOR. INDICATION WITHIN LIMITS AFTER
CALIBRATION.

8505011384

SMD

2

21AF119

FAILURE DESCRIPTION: VALVE IS LEAKING THRU, PLEASE REPAIR.

CORRECTIVE ACTION: INSTALLED PIPE CAP ON NIPPLE. LEAK STOPPED.

WO NO DEPT UNIT EQUIPMENT IDENTIFICATION

8504300323

SIC

2

21S/G STM FLOW CH. 2

FAILURE DESCRIPTION: STM FLOW CHANNEL 2 IS READING ERRACTICALLY. PLEASE INVESTIGATE AND REPAIR.

CORRECTIVE ACTION: FOUND INDICATOR OUT OF SPEC. CALIBRATION AND RETURNED TO SERVICE.

8504082288

SIC

2

RAD. MONITOR

FAILURE DESCRIPTION: THE INDICATION FOR THIS CHANNEL IS ERRATIC. PLEASE CHECK.

CORRECTIVE ACTION: CLEANED CONNECTIONS ON REMOTE CONTROLLER SN-215 AND COMPLETED CHANNEL FUNCTIONAL TEST.

0099158426

SMD

2

24 SW PMP

FAILURE DESCRIPTION: 24 SW PMP STRAINER HAS A PACKING LEAK.

CORRECTIVE ACTION: INSTALLED 3 RINGS PACKING 2 RINGS CORTEX.

0099158329

SIC

2

1D4 ROD POSITION IND BEZ

FAILURE DESCRIPTION: 1D4 CONTROL ROD INDICATION APPEARS TO BE DRIFTING.

CORRECTIVE ACTION: REPLACED METER MOVEMENT AND AMPLIFIER IN BEZEL INDICATOR. CALIBRATED INDICATOR AND SIGNAL MODULE. INDICATOR WAS CHECKED THE NEXT DAY NO DRIFTING ALL SAT.

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
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0099157632

SMD

2

25 SW PMP

FAILURE DESCRIPTION: THE PACKING ON THE PMP IS LEAKING.

CORRECTIVE ACTION: REPACKED PUMP AS PER WORK ORDER WO 8505290704
TO REPLACED PACKING SLEEVE.

0099016991

SIC

2

PLANT VENT PROCESS MON

FAILURE DESCRIPTION: HIGH - LOW DEVIATION ALARM FLOW UP. PLEASE
INVESTIGATE AND REPAIR.CORRECTIVE ACTION: PLACED DAUL CONTROLLER FOR MANUAL AND ADJUSTED
SAMPLE FLOW TO 4.6 CFM.

0099015986

SIC

2

COND POLISHER MONITOR

FAILURE DESCRIPTION: ALARM REQUIRED INDICATION FOUND OUT OF SPEC DURING
CHANNEL CALIBRATION PROCEDURE AND CHANNEL LOCKS
UP WHEN ATTEMPTING TO INPUT SIGNAL AT TEST LOCK.
PLEASE INVESTIGATE AND REPAIR.CORRECTIVE ACTION: INSTALLED NEW TEST JACK. PERFORMED CHANNEL CAL
CHECK PROCEDURE 2PD-4.5.063. PERFORMED CHANNEL
CAL PROCEDURE 2PD-4.1.063.

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 2
MAY 1985

SALEM UNIT NO. 2

The period began with the Unit operating at 50% power with adjustments to #22 Steam Generator Feed Pump in progress. On 5/02/85 at 0916 hours the Unit tripped due to loss of excitation to the Generator which resulted from improper wiring of the Loss of Excitation relay circuitry. Synchronization occurred on 5/4/85 at 1615 hours. On 5/8/85 at 1348 hours, power was reduced to approximately 80% to remove a leaking valve and cap a line on the Stator Water Cooling System. The Unit was returned to full power operation at 1500 hours the same day. On 5/10/85 at 1712 hours, the Unit tripped as a result of High Negative Flux Rate. During routine Control Rod operability surveillance testing, Control Rod 2C4 dropped due to a poor electrical connection. The Unit was returned to service at 0305 hours on 5/15/85 following repairs and inspection of all Control Rod Connectors. Full power operation was achieved at 0256 hours on 5/16/85 where it remained for the rest of the period.

REFUELING INFORMATION

COMPLETED BY: J. RonafalvyDOCKET NO.: 50-311UNIT NAME: Salem 2DATE: June 10, 1985TELEPHONE: 609/935-6000EXTENSION: 4455Month May 1985

1. Refueling information has changed from last month:
YES X NO
2. Scheduled date for next refueling: September 6, 1986
3. Scheduled date for restart following refueling: November 16, 1986
4. A) Will Technical Specification changes or other license amendments be required?
YES NO
Not determined to date
- B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
YES NO X
If no, when is it scheduled? August 1986
5. Scheduled date(s) for submitting proposed licensing action:
August 1986 if required
6. Important licensing considerations associated with refueling:
NONE
7. Number of Fuel Assemblies:
A) Incore 193
B) In Spent Fuel Storage 140
8. Present licensed spent fuel storage capacity: 1170
Future spent fuel storage capacity: 1170
9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: March 2003

8-1-7.R4



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

June 10, 1985

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of May 1985 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager -
Salem Operations

JR:sbh

cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosure
8-1-7.R4

IE2A
1/10