NRC Form 366
(9-83)*

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Susquehanna Steam Electric Station - Unit 1	0 5 0 0 0 3 8 7	1 OF 0 1

TITLE (4)
Unanticipated RPS Actuation (IRM 'C').

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 5	1 6	8 5	8 5	0 1 9		0 0	0 6	0 7	8 5		0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9)	5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
D.J. Gandenberger, Power Production Engineer	AREA CODE 7 1 7 5 4 2 - 3 9 1 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

On May 16, 1985 at 1430 during the performance of surveillance testing of Intermediate Range Monitor (IRM) (IG) 'C', a Reactor Protection System (RPS) (JC) actuation occurred. The actuation occurred when IRM 'C' was taken out of bypass by the operator (utility, licensed) at the request of an I&C technician (utility, non-licensed) in the control room. The technician in the control room misinterpreted a phone message from another I&C technician at the IRM 'C' instrument drawer in the relay room and requested the operator to remove IRM 'C' from bypass. The relay room technician had not fully restored IRM 'C' when it was removed from bypass, causing a scram signal. As required by Technical Specification 3.3.1 for refueling, the 'shorting links' had been removed from the RPS circuitry, thus permitting an upscale signal from any IRM to cause a full RPS actuation. (Normal IRM RPS input configuration requires a one-out-of-two-twice trip logic be satisfied to cause a full RPS actuation.) No control rod movement occurred as all rods were fully inserted at the time of the event. The personnel involved were counseled by first-line supervision emphasizing verbatim compliance to procedures, particularly those requiring other than face-to-face communication. Especially underscored was the necessity to be exact when communicating on the phone and not to take any action unless specifically and clearly directed to do so.

IE22
1/1

NRC Form 366
(9-83)

PP&L Pennsylvania Power & Light Company
Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

June 7, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 85-019-00
ER 100450 FILE 841-23
PLAS-086

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 85-019-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that an unanticipated Reactor Protection System actuation occurred during Nuclear Instrument surveillance testing.

H.W. Keiser
Superintendent of Plant-Susquehanna

DJG/pjg

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Mr. R.H. Jacobs
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 52
Shickshinny, PA 18655

IE22
1/1