

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
EDWIN I. HATCH, UNIT I

DOCKET NUMBER (2)

0 5 0 0 0 3 2 1

PAGE (3)

1 OF 2

TITLE (4)
RCIC FAILURE TO DELIVER RATED FLOW

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)													
0	5	1	9	8	5	8	5	0	1	6	0	0	0	6	0	7	8	5	0	5	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)															
1		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)			
POWER LEVEL (10)	0 8 8	20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(e)			
		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.406(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)							
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)							
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)

NAME		TELEPHONE NUMBER	
		AREA CODE	
Steven B. Tipps, Superintendent of Regulatory Compliance		9 1 2	3 6 7 + 7 8 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	B N	A M P G O	8 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)		X NO		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
		X					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At approximately 1220 CDT on 05/19/85, with the plant operating at approximately 2134 MWt (approximately 88% power), and during performance of the monthly "RCIC PUMP OPERABILITY" procedure (HNP-1-3405), plant personnel determined that the RCIC pump (1E51-C001) delivered only 900 PSIG at 400 GPM, when reactor pressure was 960 psig. This event is contrary to the requirements of Tech. Specs. section 4.5.E.1.c.

An immediate investigation showed that the RCIC system's automatic control amplifier control circuit board (1E51-K616) had failed. Subsequent to the investigation, the circuit board was replaced. The RCIC system was then satisfactorily functionally tested and returned to service per the "RCIC PUMP OPERABILITY" procedure (HNP-1-3405) at approximately 1345 CDT on 05/19/85.

This 30 day LER is required by 10CFR 50.73 (a)(2)(i)(B), because this event shows that the plant was operated in a condition prohibited by Tech. Specs. section 3.5.E.1.a.

This event had no actual or potential safety consequences, nor was the health and safety of the public affected.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT I	0 5 0 0 0 3 2 1	8 5	— 0 1 6	— 0 0	0 2	OF 0	2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At approximately 1220 CDT on 05/19/85, with the plant operating at approximately 2134 Mwt (approximately 88% power), and during performance of the monthly "RCIC PUMP OPERABILITY" procedure (HNP-1-3405), plant personnel determined that the RCIC pump (1E51-C001) delivered only 900 PSIG at 400 GPM, when reactor vessel pressure was 960 psig. This event is contrary to the requirements of Tech. Specs. section 4.5.E.1.c, because RCIC did not meet its flow rate requirement of 400 GPM at normal reactor vessel operating pressure.

A seven day LCO was initiated, and immediate further investigation showed that the RCIC system's automatic control amplifier circuit board (1E51-K616) had failed.

Subsequent to the investigation, the RCIC system's automatic control amplifier circuit board (1E51-K616) was replaced. The RCIC system was then satisfactorily functionally tested and returned to service per the "RCIC PUMP OPERABILITY" procedure (HNP-1-3405) at approximately 1345 EDT on 05/19/85.

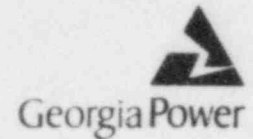
The cause of the amplifier circuit board (1E51-K616) failure is not known. However, the circuit board will be returned to the vendor for analysis and repair.

This 30 day LER is required by 10CFR 50.73 (a)(2)(i)(B), because this event shows that the plant was operated in a condition prohibited by Tech. Specs. section 3.5.E.1.a; i.e., RCIC would not deliver rated flow at normal reactor vessel pressure in its automatic mode.

This event is the consequence of component failure. When automatic control amplifier 1E51-K616 failed, the result was reduced RCIC turbine speed; consequently, RCIC pump flow was reduced to the extent that rated flow could not be delivered at normal reactor pressure vessel operating pressure.

During this event the RCIC system was capable of being controlled manually to deliver the required flow rate at normal reactor vessel pressure. Therefore, no actual or potential safety consequences resulted, nor did this event affect the health and safety of the public. There have been no other past similar events.

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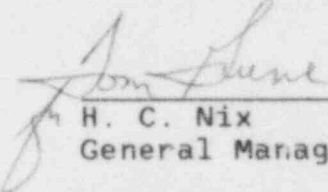
Edwin I. Hatch Nuclear Plant

June 7, 1985
GM-85-559

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-321

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-321/1985-016. This report is required by 10CFR 50.73(a)(2)(i).



H. C. Nix
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