AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
Unit Name Salem # 1
Date June 10, 1985
Telephone 609-935-6000
Extension 4455

Completed by J. P. Ronafalvy

Month May 1985

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	1106	17	1099
2	1084	18	1100
3	1104	19	1105
4	1097	20	1091
5	1104	21	1104
6	1085	22	1098
7	1099	23	1098
8	1100	24	1095
9	1101	25	1104
10	1096	26	1095
11	1093	27	1086
12	1096	28	1103
13	1099	29	1103
14	1103	30	1094
15	1105	31	1095
16	1096		

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OPERATING DATA REPORT

Docket No. 50-272
Date June 10, 1985
Telephone 935-6000
Extension 4455

Completed by J. P. Ronafalvy

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	May 1985	
3.	Licensed Thermal Power (MWt)	3338	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1090	
6.	Maximum Dependable Capacity (Gross MWe)	1124	
7.	Maximum Dependable Capacity (Net MWe)	1079	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	3623	69432
12. No. of Hrs. Reactor was Critical	744	3605.6	39429.1
13. Reactor Reserve Shutdown Hrs.	0	0	3088.4
14. Hours Generator On-Line	744	3602.7	37761.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2483261	11960859	114728856
17. Gross Elec. Energy Generated (MWH)	849490	4093240	38009290
18. Net Elec. Energy Generated (MWH)	816916	3935588	36033570
19. Unit Service Factor	100	99.4	54.4
20. Unit Availability Factor	100	99.4	54.4
21. Unit Capacity Factor (using MDC Net)	101.8	100.7	48.1
22. Unit Capacity Factor (using DER Net)	100.7	99.7	47.6
23. Unit Forced Outage Rate	0	0.6	31.0

24. Shutdowns scheduled over next 6 months (type, date and duration of each)
N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:
N/A

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	9/30/76	12/11/76
Initial Electricity	11/1/76	12/25/76
Commercial Operation	12/20/76	6/30/77

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH May 1985

Completed by J.P. Ronafalvy

Docket No. 50-272
Unit Name Salem No.1
Date June 10,1985
Telephone 609-935-6000
Extension 4455

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence

* There were no outages for the month of May resulting in a load reduction of greater than 20%.

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH May 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: June 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1360	28VDC	Replace existing undervoltage relay with new type (RIS Model ET-1214 XC with welded enclosure).
1EC-1477	Solenoid Valves (X130)	Replace internals of solenoid valves SV0394, 0397, 0399, 0401, 0506, 0518, 0519, 0520, 0521, 0910, 0911, 0912, 0913, 0914, 0915, 1077, 1079, 1081, 1083, 1384 and 1385 with resilient seated valve internals.
1EC-1756	Overhead Annunciator	Modify delay time from 50 milliseconds to 250 milliseconds for first out annunciator panel.
1EC-1766	Post Accident Sampling	Design and install sample lines from the discharge of the RHR pumps to the PASS Liquid Sampling Panel #814. The modification shall be capable of sampling the 11 and 12 RHR trains. Samples shall be conditioned/cooled prior to entering Panel 813.
1EC-1794A	Service Water-Radiation Monitoring	Relocate FCU RMS 1R13!&B to east service water valve room and RMS 1R-13C,D&E to west service water valve room from present location. In vicinity of 1R-13A&B install two new dummy detectors with lead enclosure.
1EC-1823	Circulating Water Main Condenser Cathodic Protection	Add a cathodic protection anode in the center end of the outlet waterboxes of the Unit No. 1 Main Condenser (6 anodes total).

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1834	RVLIS	Perform the field modification as per Westinghouse Procedure No. NSID-EIS-83-16. Scope of this change is under Section 5 of this procedure which includes environmental/seismic qualification changes and human factors modifications.
1EC-1837	Service Water Intake Structure	Raise the ventilation intake penthouse and add structural steel barrier plates to the ventilation exhaust penthouse on the top of the Service Water Intake Structure in order to provide flood and wave run-up protection El. 128.
1EC-1838	Technical Support Center	Relocate the interim TSC located on the second floor of the Clean Facilities Building to the third floor. Also arrange the new furniture in accordance with the Nuclear Site Protection Department layout plan.
1EC-1892	Circulating Water	Replace circulating water pump vacuum breakers valves #11CW15, 12CW15, 13CW15, 11CW115, 12CW115, 13CW115.
1EC-1976	Service Water	Install temporary spool piece for valve 13SW20.
1EC-1999	Main Steam	Replace turbine first stage impulse pressure transmitters PA0195 and PA0216 and mount so as to reduce vibration.

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1ET-2045	Switchyard	Identification of spare conductors and jumpers required for paralling in Switchyard 13.8kv breaker control circuits on a temporary basis to eliminate voltage drop problem.
1SC-0663	Refueling Water Tank	Install station lighting in pipe tunnel for refueling water tanks.
1SC-0787	Condensate Chemical Analysis	Change the hotwell condensate setpoints from 0.5 umhos to 0.3 umhos.
1SC-0839	Steam Generators-Secondary Side	Revise the design of the Salem 1 steam generators moisture separator drains.
1SC-1170A	Station Air Compressors	Install 8" pipe spool down stream of separator.
1SC-1312B	Containment Personnel Hatch	Install caps and valving at personnel hatch El. 100 and 130.
1ST-1368	Reactor Coolant	Verify operability of the reactor head vent system. Identify valve seat leakage if any at various system pressure and temperature conditions. Verify operability of the solenoid valves at rated system pressure and temperature conditions.
1SC-1419	Warehouse #2 Receiving Cage	Increase the size of receiving cage to include a work area for QA inspection, QA hold area and larger work area for receiving material.

MAJOR PLANT MODIFICATIONS
REPORT MONTH MAY 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: June 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1360	The new relay will have a higher setpoint. It is not safety related. The failure of the relay will cause the loss of alarm function if the 28VDC charger feeding the Bus fails. The loss of charger may be verified by reading the 28VDC system voltmeter located on 1RP9. The relay is mounted upright to minimize the possibility of it vibrating out of its socket. There is nothing in the panel which could be damaged by the undervoltage relay during a seismic event and cause an unsafe condition. No unreviewed safety or environmental questions are involved.
1EC-1477	No changes are being made which change the qualification, function or fit of any equipment. No unreviewed safety or environmental questions are involved.
1EC-1756	This modification will assure that the first-out panel alarms latch only when accompanied by a valid trip signal. No unreviewed safety or environmental questions are involved.
1EC-1766	The PASS shielding assembly is located in the Aux. Bldg. and runs between the Unit 1 and 2 Boric Acid Evaporator Rooms. The installation is performed in accordance with the design specifications for the presently installed PASS. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1794A	The Service Water (SW) System pressure boundary will not be jeopardized by this DCR. It only involves the relocation of the existing RMS units and the SW supply line to these units. If the relocated RMS should malfunction due to a false high signal and actuate the SW discharge valves to close, the Control Room personnel will be alerted via the annunciation panel and manually override the control circuitry to operate the valves as required. Also, if the relocated RMS should malfunction due to low or no signal and the SW discharge valves stay open, the Control Room personnel will be alerted via the control annunciation panel indicating that the system requires correction/calibration. If the relocated RMS should fail to function due to flooding, there will be no interruption in the SW fan coil units to continue their safety function. No unreviewed safety or environmental questions are involved.
1EC-1823	This modification does not affect any presently performed safety analysis, nor does it create any new hazards. The basis for the Tech. Specs remains unchanged. This modification will not alter any plant discharge or process and will not affect the existing plant environmental impact. No unreviewed safety or environmental questions are involved.
1EC-1834	All potential, realistic failure modes have been considered but are not applicable. These changes are required to make the RVLIS System fully meet the NRC requirements of NUREG 0737. No unreviewed safety or environmental questions are involved.
1EC-1837	This DCR raises the Service Water (SW) Intake Structure Ventilation Intake Penthouse flood and wave runup protection level from elevation 122' to elevation 128'. Structural barriers will also be installed on the Ventilation Exhaust Penthouse to protect the SW Intake Control Rooms from flooding and inhibit water from running into the exhaust chase down into the pump rooms. The FSAR is being changed to incorporate the revised flood and wave runup level for the structure. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1838	The Clean Facilities Building and the Meterological System are not safety related. The function of the Meterological System will not be altered. No unreviewed safety or environmental questions are involved.
1EC-1892	The replacement of the vacuum breaker valves does not alter any presently performed safety analysis. It also does not create any new hazards. This modification will not alter any plant process or discharge and will not affect the existing plant impact. No unreviewed safety or environmental questions are involved.
1EC-1976	The installation of the spool piece allows the plant to change modes. It temporarily isolates the SW supply to the Turbine Generator area so the Unit can be operated in Mode 1 provided the monthly surveillance test is not required during the use of the temporary installation. The installation can be removed upon completion of repairs to valve 13SW20. No unreviewed safety or environmental questions are involved.
1EC-1999	This modofication involves replacement of malfunctioning pressure transmitters. Functionally, there is no change. No unreviewed safety or environmental questions are involved.
1ET-2045	No safety related systems are involved with this DCR. Therefore, no unreviewed safety or environmental questions are involved.
1SC-0663	The implementation of this DCR will utilize the standard cable control manual procedures. No unreviewed safety or environmental questions are involved.
1SC-0787	The intended function of this system remains unaffected. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1SC-0839	This DCR will provide more effective drainage of water, but will not affect steam generation nor system performance. Unit 2 has had these same modifications installed prior to its commercial operation. This modification will not add or alter effluent release to the environment as specified in the ETS or NPDES permit. No unreviewed safety or environmental questions are involved.
1SC-1170A	The modifications of the drainage lines off the compressors intercoolers and the separators do not create a new safety hazard. The changes made to the FSAR are only made to reflect the "wording" changes and the flow diagram changes. No Tech. Spec. changes are involved. No unreviewed safety or environmental questions are involved.
1SC-1312B	All realistic failure modes have been considered but are not applicable. This DCR meets GDC 56 requirements and complies with safety guide #11. The additional equipment meets the design criteria of Seismic I and Nuclear Class II. No unreviewed safety or environmental questions are involved.
1ST-1368	The system will not be modified in any way as a result of this test. This test is required to comply with NUREG 0737 Item II.B.1. The test involves testing the head vent system operability and leakage (if any). The test procedure contains all necessary precautions. The test will be done in Modes 6, 5, 4, and 3 and will be conducted in accordance with the normal design operation of the system. No unreviewed safety or environmental questions are involved.
1SC-1419	This DCR expands the existing receiving and hold area in Warehouse No. 2 by lengthening the cage wall by 50' to the east. This building and the cage modification are not safety related. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
8505280032	SMD	1	12 BORIC ACID TRANS. PUMP	
			FAILURE DESCRIPTION:	#12 BORIC ACID TRANSFER PUMP TRIPPED OUT, BREAKER TRIPPED ON THERMAL OVERLOAD. ATTEMPTED ONE RECLOSURE. CONTACTS CHATTERED BUT PUMP SHAFT WOULD NOT MOVE. PUMP SHAFT APPEARS FROZEN. PLEASE INVESTIGATE AND REPAIR.
			CORRECTIVE ACTION:	REBUILT PUMP, REPLACED SHAFT, GASKET AND SEALS. PERFORMED ALIGNMENT AS PER M6D AND IMPELLER CLEANING.
8505190190	SMD	1	START INDICATION	
			FAILURE DESCRIPTION:	THERE IS NO START INDICATION FOR 11 CHARGING PUMP. THE BULBS HAVE BEEN CHECKED AND THE BEZEL HAS BEEN SWAPPED WITH AN OPERABLE BEZEL. THE PUMP HAS BEEN VERIFIED RUNNING AND THE 125VDC AND 28VDC POWER SUPPLIES CHECKED. PLEASE REPAIR.
			CORRECTIVE ACTION:	REPLACED DEFECTIVE RELAY.
8505110315	SIC	1	12 WG COMP CC FLOW	
			FAILURE DESCRIPTION:	THE FLOWMETER READS 34 GPM EVEN WHEN THE VALVES ARE CLOSED WITH NO FLOW THROUGH THE LINE.
			CORRECTIVE ACTION:	REPLACED FLOAT ASSEMBLY.
8505110340	SMD	1	SEAL INJECTION LINE	
			FAILURE DESCRIPTION:	THE VALVE HAS A FLANGE LEAK. PLEASE REPAIR.
			CORRECTIVE ACTION:	TIGHTENED DOWN ON PACKING. STOPPED VALVE FROM LEAKING

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
8505060610	SMD	1	PRESSURIZER HEATERS
			FAILURE DESCRIPTION: BREAKER 1EP4X KEEPS TRIPPING. PLEASE INVESTIGATE.
			CORRECTIVE ACTION: TESTED AND RE-INSTALLED NEW BREAKER
8504300111	SMD	1	15SW25
			FAILURE DESCRIPTION: VALVE BLOWS WATER FROM DIAPHRAGM WHENEVER 16SW24 CYCLES.
			CORRECTIVE ACTION: INSTALLED NEW BONNET.
8504300668	SMD	1	PZR HTR DEION 1EP3X
			FAILURE DESCRIPTION: DEION IS TRIPPED AND WILL NOT RESET. MAINTENANCE DEPARTMENT PLEASE REPAIR.
			CORRECTIVE ACTION: REPLACED 1EP3X DEION WITH A NEW DEION.
8504300676	SMD	1	PZR HTR DEION 1EP11X
			FAILURE DESCRIPTION: DEION IS TRIPPED AND WILL NOT RESET. MAINTENANCE DEPARTMENT PLEASE REPAIR.
			CORRECTIVE ACTION: REPLACED 1EP11X DEION WITH A NEW DEION.
8504250555	SIC	1	PZR LEVEL CONTROL CHAN. I
			FAILURE DESCRIPTION: PRESSURIZER LEVEL CONTROL CHANNEL I IS READING APPROX. 3-4% HIGHER THAN THE OTHER TWO CHANNELS. PLEASE INVESTIGATE AND RECALIBRATE AS NECESSARY.
			CORRECTIVE ACTION: ADJUSTED BAILEY INDICATOR APPROXIMATELY 2.5% DIFFERENCE BETWEEN CHANNEL 1 & 3.

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
8504090027				
	SMD	1	12 CHILLER RECIRC.	
			FAILURE DESCRIPTION:	12 CHILLER RECIRC PUMP MECHANICAL SEAL IS LEAKING.
			CORRECTIVE ACTION:	INSTALLED NEW SEAL.
0099164060				
	SIC	1	#13 S/G STM FLOW CH II	
			FAILURE DESCRIPTION:	1FT-533 COMPARATOR OUTPUT IS ERRATIC. PLEASE CHECK.
			CORRECTIVE ACTION:	REPLACED COMPARATOR AO-526 WITH AO-1027. CALIBRATED MODULE PER CH CAL SHEET OF 19 OF 20 OF 1PD-2.2.049.
0099158639				
	SMD	1	11 SW PMP STRAINER	
			FAILURE DESCRIPTION:	THE STRAINER SHEAR PIN IS BROKEN.
			CORRECTIVE ACTION:	REPLACE SHEAR KEY, LOWER SHOES AND STRAINER ELEMENTS.
0099024373				
	SMD	1	12 FHB EXHAUST FAN	
			FAILURE DESCRIPTION:	ROUGHING FILTER HAS A HIGH DIFF. PRESSURE (2). PLEASE CHECK AND CORRECT.
			CORRECTIVE ACTION:	REPLACED ROUGHING FILTERS (20) 40-1339.

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 1
MAY 1985

SALEM NO. 1

The Unit operated at full power for the entire period.

REFUELING INFORMATION

COMPLETED BY: J. Ronafalvy

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: June 10, 1985
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month May 1985

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: February 22, 1986

3. Scheduled date for restart following refueling: May 4,1986

4. A) Will Technical Specification changes or other license amendments be required?
   YES _____ NO _____
   NOT DETERMINED TO DATE _____

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES _____ NO X
   If no, when is it scheduled? January 1986

5. Scheduled date(s) for submitting proposed licensing action:
   January 1986 if required

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 296

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001

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PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

June 10, 1985

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of May 1985 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

JR:sbh
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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