



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 5, 1993

Docket
File

Docket No. 50-245

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: SAFETY EVALUATION OF THE THIRD TEN-YEAR INTERVAL INSERVICE
INSPECTION PROGRAM PLAN FOR MILLSTONE NUCLEAR POWER STATION, UNIT 1
(TAC NO. M79663)

The Northeast Nuclear Energy Company (NNECO) submitted the Third Ten-Year Interval Inservice Inspection (ISI) Program Plan and associated requests for relief for the Millstone Nuclear Power Station, Unit 1 to the NRC by letter dated December 14, 1990. The NRC staff requested additional information to complete the review of the ISI program on September 3, 1991. The information was provided by NNECO in a submittal dated November 5, 1991. Further information was requested and a commitment was developed in telephone conversations with NNECO on January 17 and 20, 1992, and February 25, 1992. As a result of the telephone conversations, NNECO submitted additional information in a letter dated March 30, 1992. Additional telephone conversations occurred on September 17, 1992, concerning NNECO's high energy line break program, and NNECO provided the dates of documents demonstrating its compliance with staff guidance and positions.

The NRC staff, with technical assistance from the Idaho National Engineering Laboratory (INEL), has reviewed and evaluated the Program Plan, additional information provided, and requests for relief from some Section XI requirements that NNECO determined to be impractical to perform at the facility. The Program Plan with associated requests for relief was submitted for review and evaluation of compliance with the requirements of the 1986 Edition of Section XI of the ASME Boiler and Pressure Vessel Code, the Regulations, and plant Technical Specifications.

Based on information submitted, the staff agrees with INEL's conclusions and recommendations except for the conclusion regarding Standard Review Plan Section (SRP) 3.6.2, "Determination of Rupture Locations and Dynamic Effects Associated with the Postulated Rupture of Piping," and staff positions in Branch Technical Position (BTP) MEB 3-1 presented in the Technical Evaluation Report enclosed. Because of the perceived noncompliance by NNECO, INEL recommended that the Millstone Nuclear Power Station, Unit 1, Third Ten-Year ISI Interval Program Plan be found unacceptable. NNECO identified documents dated August 30, 1973, January 29, 1974, March 20, 1974 and August 14, 1975, as demonstrating its compliance with staff guidance and positions. NNECO had

Mr. John F. Opeka

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satisfied the staff by meeting or providing acceptable alternatives to the guidance and technical positions in SRP 3.6.2 and BTP MEB 3-1. Accordingly, the staff has found the Millstone Nuclear Power Station, Unit 1, Third Ten-Year Interval ISI Program Plan acceptable.

The NRC staff has concluded that the Millstone Nuclear Power Station, Unit 1, Third Ten-Year Interval Inservice Inspection Program with the additional information provided and reliefs granted, or granted with conditions, constitute part of the basis for meeting the requirements of 10 CFR 50.55a(g) and the Technical Specifications. We have determined that Section XI Code requirements for examination of some components cited by NNECO are impractical to perform at the Millstone Nuclear Power Station, Unit 1 and we have granted relief from those requirements. The NRC staff has determined that granting relief, pursuant to 10 CFR 50.55a(a)(3)(i) and (g)(6)(i), is authorized by law and will not endanger life, property, or the common defense and security and is otherwise in the public interest giving due consideration to the burden upon NNECO that could result if the ASME Code requirements were imposed on the facility. In cases where the findings could not be made, we have denied the requests. A summary of the requirements and the bases for granting, granting with conditions, or denying relief requests are contained in the enclosed Safety Evaluation and Technical Evaluation Report. Other requests were authorized as acceptable alternative examinations as provided in 10 CFR 50.55a(a)(3).

Sincerely,

Original signed
by

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Safety Evaluation
2. Technical Evaluation Report

cc w/enclosures:
See next page

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Millstone Nuclear Power Station
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