

U.S. NUCLEAR REGULATORY COMMISSION
REGION I

Report No. 50-423/85-19

Docket No. 50-423

License No. CPPR-113 Priority -- Category C

Licensee: Northeast Nuclear Energy Company

P.O. Box 270

Hartford, Connecticut 06141-0270

Facility Name: Millstone Nuclear Power Station, Unit No.3

Inspection At: Waterford, Connecticut

Inspection Conducted: May 7-10, 1985

Inspectors: J. R. White, Senior Radiation Specialist

6/6/85
date

J. A. Cioffi, Radiation Specialist

6/6/85
date

Approved by: M. M. Shanbaky, Chief, PWR Radiation Safety Section

6/10/85
date

Inspection Summary: Inspection on May 7-10, 1985 (Report No. 50-423/85-19)

Areas Inspected: Routine, unannounced safety inspection to review pre-operational status of areas pertaining to radiation protection and radioactive waste management, including organization, personnel training and qualifications, facilities and equipment, and procedure development. The inspection involved 70 hours onsite by two region-based inspectors.

Results: No violations were identified within the areas inspected.

DETAILS

1.0 Persons Contacted

- *W. D. Romberg, Station Superintendent
- *R. Herbert, Station Services Superintendent
- *D. Miller, Startup Manager
- K. Gray, Staff Assistant
- *F. Perry, Assistant Radiation Protection Supervisor
- *B. Granados, Health Physics Supervisor
- M. Pearson, Operations Assistant
- *R. Sachetello, Radiation Protection Supervisor
- M. Brown, Instrument and Control Supervisor
- *R. Enoch, Instrument and Control Group Leader
- P. O'Connell, Staff Assistant
- *R. Lager, Chemistry Supervisor
- *L. Wadeau, Northeast Utilities Staff

*Denotes attendance at the Exit Interview conducted May 13, 1985.

2.0 Purpose

The purpose of this inspection was to review the pre-operational status of the unit in regard to radiation protection program development. Included in this review was examination of organization and management controls, personnel training and qualification, facilities and equipment, and status of procedures for surveys, contamination control, and initial startup surveillance.

3.0 Status of Previously Identified Items

3.1 (Closed) Bulletin (78-BU-07) Protection Afforded by Air-Line Respirators and Supplied-Air Hoods.

The licensee's procedure SHP 4931, Rev. 3, dated April 9, 1985, pertains to issuance and use of air-line respirators and supplied air hoods. The procedure contains instructions that are consistent with the precautionary provisions of the bulletin and adequately addressed all of the concerns that were identified.

3.2 (Open) Bulletin (78-BU-08) Radiation Levels From Fuel Element Transfer Tubes

FSAR 12.3.1.2 implies that Fuel Element Transfer Tube shielding has been enhanced at the Containment wall interface and in the space between the Containment and Fuel Handling Building. An internal memorandum, NES-24078, dated April 16, 1981 also indicates that the shielding had been modified for a design basis of 50 millirem/hour at the shield and 5 millirem per hour at the personnel access hatch;

and referenced drawings 12179-EC-51R for the in-containment modification and 12179-EL-49R for the modification to the fuel element tube between the Containment and Fuel Handling Building. The FSAR further described a security fence in-place to prevent inadvertent access to the space between the Containment and Fuel Handling Buildings.

The security fence was not installed, but is being carried as an open item by the licensee pending the completion of construction in the area. The licensee was able to produce drawing 12179-EV-261A-2, which is depicted in the FSAR, but was unable to produce as-built drawings, or drawings 12179-EC-51R and 12179-EL-49R that described the shielding applied to the tube.

This item remains open pending verification of shielding and FSAR statements by review of associated as-built drawings, and the installation of the security fence between the buildings.

3.3 (Open) 76-CI-03 (Circular) Radiation Exposure in Reactor Cavities.

The licensee has not initiated any action to address the concerns expressed in this circular. Review of the Containment Building and accesses to the areas under the reactor vessel and adjacent to thimble guide tubes indicate that significant action in terms of engineered controls and administrative procedures need to be developed.

4.0 Fuel Receipt Activities

The licensee's actions relative to fuel receipt were examined against the following criteria:

- Operation Procedure 3211A, "New Fuel Assembly and RCCA Receipt and Inspection"
- Rad Waste Procedure RW36006, "Receipt of Radioactive Material"
- Health Physics Procedure HP4905, "Radiological Surveys"
- Special Nuclear Material License Application, dated January 28, 1985
- Supplement to Special Nuclear Material (SNM) License Application, dated March 20, 1985
- Special Nuclear Material License No. 1950, dated April 16, 1985

On May 8, 1985 the inspector witnessed the licensee's efforts relative to receipt of seven containers of new fuel (container nos. 82, 45, 77, 144, 115, 156 and 137). The licensee's staff performed extensive surveys of the truck and contents; and provide escort to the Fuel Handling Building.

Areas were properly posted and personnel restrictions and RWP requirements were established. Continuous coverage was maintained during all fuel inspections and transfer activities.

The personnel providing radiological control were verified to have been trained and qualified in accordance with the SNM license conditions.

Sufficient radiological surveys were taken and evaluated during work performance, including venting of containers prior to opening. All survey results were recorded and documented with the fuel receipt packages.

No unexpected loose surface contamination was found, and dose rates in close proximity to the new fuel were less than 5 millirem per hour. Personnel dosimetry was required for all personnel involved in the fuel receipt activity.

The licensee's performance was determined to be in accord with the conditions of the SNM license.

5.0 Radiation Monitoring System

The Millstone Unit No. 3 Radiation Monitoring System (RMS) is a micro-processor based network of process and area monitors designed to detect, indicate, annunciate and record radioactivity levels associated with operation of the plant.

The as-built system includes, fifteen ventilation monitors with dual channel capability, sixteen liquid process monitors, and thirty-seven area radiation monitors.

Control of all functions, such as recording, display, and computation is centralized in the RMS computer system which consists of two PDP 11/34 minicomputers. The computers are normally linked by synchronous serial interfaces but can be operated independently for diagnostic and software development purposes.

The system is controlled by an operator console in the control room, a color CRT, a keyboard and a printer to produce a picture of the CRT displays. A second-terminal is located in the health physics facility space.

The monitors are linked to the computer system by seven data loops, two for 1E monitors and five for non-1E monitors. Two Category 1E control room cabinets are provided for recording, indication and control of the 1E monitors for the purpose of alarm scanning, data logging, display, data recording and retrieval.

Since the RMS system is an integral part of the plant design, there was concern when the vendor agency, KAMAN Instruments, Inc., became incapable

of supporting the system. However, the licensee's Instrument and Control, and Computer Support organizations were able to acquire significant detail and information concerning the design of the system including hardware and software components; and have amassed a technical capability that appears able to continue with the startup, test, maintenance and repair of the system.

As of May 10, 1985, the status of the system was as depicted in Attachment 1. The operational status of the RMS will continue to be followed in subsequent inspections of this area. (50-423/85-19-01)

The licensee has initiated surveillance procedure development (i.e., for operation, testing and calibration) for RMS units that are expected to be addressed by Technical Specifications. Attachment II depicts the status of these procedures as of May 4, 1985.

RMS devices which are not addressed by Technical Specification are operated, tested and calibrated in accordance with generic procedures developed for each instrument type. Attachment III depicts the status of these procedures as of April 24, 1985.

The status of RMS procedures will continue to be followed in subsequent inspections of this area. (50-423/85-19-02)

While the RMS system and associated procedures are not yet complete, remaining actions have been scheduled to support full system operability by start-up. Sufficient management attention has been directed to this area and system status is being monitored on a weekly basis to assure system readiness.

Health Physics personnel, though having responsibilities to monitor and utilize the RMS, have not been subjected to any training on the capabilities of the system, operation, and how to interrogate the system. A simulated system has been established for reactor operator training, but has not been used for instructing health physics personnel.

This item will remain open pending completion of training for health physics personnel in RMS operation. (85-423/85-19-03)

No violations were identified in this area.

6.0 Status of Start-up Survey Procedure

The licensee's startup procedure was reviewed against Regulatory Guide 1.68, "Initial Test Programs for Water-Cooled Nuclear Power Plants", and Final Safety Analyses Report, Chapter 14, "Initial Startup Test Phase" to establish the status of planning for initial surveys for power ascent.

Procedures for startup surveys have not yet been developed, however, the responsibility for writing startup survey procedures has been assigned to

the corporate Health Physics staff. In addition to using TLDs on phantoms and performing gamma and neutron surveys of the containment penetration areas, licensee representatives stated that they plan to test the response of the GM detectors and ionization chambers in the subatmospheric conditions of the containment during power ascent. The procedure for the startup survey will be reviewed in a future inspection (50-423/85-19-04).

7.0 Radiation Protection Organization and Staffing

The licensee's radiation protection organization and management controls was reviewed against:

- Millstone 3 Final Safety Analysis Report, Chapter 12, "Radiation Protection"
- Millstone 3 Final Safety Analysis Report, Chapter 13, "Conduct of Operations"

The radiation protection organization for Millstone Unit 3, as depicted in Figure 1, parallels the organization already in place for Units 1 and 2. The Station Superintendent has the ultimate responsibility for the health physics program. The Radiological Services Supervisor reports directly to the Station Superintendent and implements the program for all three Millstone units. Reporting to the Radiological Services Supervisor is the Health Physics Supervisor.

The Health Physics Supervisor for the Millstone site manages three Radiation Protection Supervisors, one for each unit, and one Radiation Protection Supervisor-Support, who administers the external dosimetry program, whole body counting program, respiratory protection program and the maintenance and calibration of radiation instrumentation for Units 1 and 2. Discussions with licensee representatives indicated that the Radiation Protection Supervisor-Support will supply support activities for Unit 3 also.

The Radiation protection supervisor of Unit 3 has one assistant radiation protection supervisor and ten technicians to administer the health physics program at Unit 3. Additional contractor support will be acquired as needed for startup.

8.0 Personnel Selection, Training and Qualifications

The training and qualifications for the radiation protection staff were reviewed against:

- Millstone 3 Final Safety Analysis Report, Chapter 12, "Radiation Protection"
- Regulatory Guide 1.8, Revision 1, "Personnel Selection and Training"

- ANSI N18.1 - 1971, "Selection and Training of Nuclear Power Plant Personnel"
- Regulatory Guide 8.8, "Information Relevant to Ensuring that Occupational Radiation Exposures at Nuclear Power Stations Will Be As Low As Is Reasonably Achievable"
- Regulatory Guide 8.27, "Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power Plants."

All radiation protection staff attended a two week health physics course taught at the corporate training center in Berlin, Connecticut. In addition, a Unit 3 systems course was given for the Unit 3 health physics technicians to acquaint them with the systems and layout of the plant.

The Millstone site training facility conducts additional training courses, such as Emergency Monitoring Training. Annual Requalification Training of the health physics personnel is scheduled to take place in June, however, the lesson plans have not yet been written. These lesson plans containing course content, and the results of the requalification of personnel will be reviewed in a subsequent inspection (50-423/85-19-05).

All radiation protection personnel met the qualifications specified in the Final Safety Analysis Report.

9.0 Portable Survey, Sampling and Contamination Monitoring Instruments

The supplies of equipment for Unit 3 were reviewed against commitments made in the Final Safety Analysis Report, Chapter 12, "Health Physics Program", by discussions with the Radiation Protection Supervisor-Unit 3, the Radiation Protection Supervisor-Support, the Assistant Radiation Protection Supervisor, a tour of the instrumentation storage facility, and observation of the supplies of equipment.

The inspector found sufficient supplies of all portable survey, sampling, and contamination monitoring instruments, with the exception of one type of alpha scintillation ratemeter specified in the FSAR. Discussions with licensee personnel indicated that the range specified for the alpha scintillation rate meters (0-2 million DPM) was not typical for alpha counting equipment. There are sufficient supplies of alpha ratemeters in the range of 0 to 500,000 disintegrations per minute on site. Licensee representatives stated that this matter would be reviewed, and resolved prior to start-up. The status of this item will remain open until reviewed in a subsequent inspection (50-423/85-19-06).

The inspector examined the supplies of emergency equipment placed outside of the control room. There was sufficient supplies of calibrated instrumentation, pumps for air sampling, and protective clothing, and a copy of emergency procedures in the locker in the event of an emergency.

10.0 Equipment for Personal Dosimetry

The inspector reviewed the supplies of pocket dosimeters (PICs) against the commitments made in the FSAR. Although there were sufficient supplies of PICs in the ranges 0 to 500 milliroentgens, 0 to 2000 milliroentgens, and 0 to 100 roentgens, the PICs in the ranges 0 to 1000 milliroentgens and 0 to 5000 milliroentgens were not in sufficient numbers with respect to the commitments in the FSAR. Additionally, there were insufficient supplies of pocket dosimeter chargers in the health physics office at Unit 3. The supplies of this equipment will be reviewed in a subsequent inspection (50-423/85-19-07).

11.0 Facilities

The facilities associated with the radiation protection program were reviewed against the plan presented in Millstone Unit 3 Final Safety Analysis Report, Chapter 12, "Health Physics Program".

The licensee's provisions and capabilities were determined from discussions with licensee personnel, and a tour of the health physics and chemistry facilities, and the plant.

The arrangement of the health physics facilities is currently unresolved. Discussion with Unit 3 personnel indicate that the facility, as described in the FSAR, does not have sufficient space for calibration of equipment, personnel and equipment decontamination, storage of equipment and supplies, a counting room sufficient for the requirements of the equipment, facilities for the whole body counter, respiratory protective equipment, and dosimetry issue. Licensee management is aware of the lack of sufficient space at the Unit 3 facility and is considering additional buildings centrally located to the three units for Health Physics activities.

However, in the interim, there are sufficient facilities available to perform whole body counting; and respirator and dosimetry issuance at Units 1 and 2. It is currently the licensee's plan to remove physical barriers between Unit 3 and the rest of the site by start-up to take advantage of facilities at Units 1 and 2. This matter will remain open pending the decision on the use of space and facilities for Health Physics functions of Unit 3 (50-423/85-19-08).

12.0 Status of Health Physics Procedures for Surveys and Contamination Control

The inspector reviewed the Health Physics procedures relative to the criteria contained in:

- 10 CFR 20.101, "Radiation dose standards for individuals in restricted areas"
- 10 CFR 20.201, "Surveys"
- 10 CFR 20.202, "Personnel Monitoring"
- 10 CFR 20.203, "Caution Signs, Labels, Signals, and Controls"
- Final Safety Analysis Report (FSAR), Chapter 12, "Health Physics Program"

The licensee's provisions and capabilities related to the above criteria were determined from discussions with licensee personnel, a review of the procedures for surveys and contamination control, and a tour of the plant.

Unit 3 personnel have generally adopted the current and approved procedures developed by the corporate health physics staff and Units 1 and 2. With few exceptions, these procedures define a program of acceptable radiological control for Unit 3 activities. The licensee continues to make unit specific changes as necessary.

Contamination control for Unit 3 is still being developed. Discussions with licensee representatives indicate that areas within the containment, auxiliary building, fuel handling building, engineered safety features building, and turbine building have been informally evaluated for frisking stations, portal monitors, and dressing facilities. The licensee plans to utilize the same type of contamination control used in Unit 2 (that is, dressing areas and frisking areas located very close to sources of contamination to prevent spreading contamination to otherwise clean areas).

Within the scope of this review, the following concerns were raised with two existing procedures:

- Procedure SHP 4909, Revision 0, "Personnel Monitoring and Decontamination" states in step 8.3.5 that Health Physics personnel or medical personnel could decontaminate the eyes, ears, nose, or throat of contaminated personnel using an isotonic solution and flushing the areas. The inspector determined that health physics personnel were not trained in proper flushing of eyes, ears, nose, and throat, and in the use of isotonic solutions for such situations. Additionally, the procedure for skin decontamination listed eleven steps using increasingly corrosive chemicals. The licensee was unable to provide evidence that such chemicals and treatments were available.
- Procedure SHP 4905, Revision 6, "Radiological Surveys", contained typographical errors in equations used to determine alpha activity. Also the self-absorption factor was not included in the calculation of activity on filter papers.

In the same procedure, in counting filter paper wipes for beta activity, the procedure stated to fold paper over the filter and

count the wipe in that configuration to prevent accidental contamination of the counting equipment. However, no correction was made for the efficiency of counting beta activity with paper over the wipe.

The licensee stated that these procedures were under current revision and some of these deficiencies were already identified. These procedures will be reviewed in a future inspection (50-423/85-19-09).

13.0 Exit Meeting

The inspectors met with licensee representatives (denoted in Paragraph 2) on May 7, 1985, to discuss the scope and findings of the inspection. At no time during this inspection was written material provided to the licensee by the inspectors.

ATTACHMENT 1

Radiation Monitoring System Status

<u>Instrument Number (Mark No.)</u>	<u>Turned Over</u>	<u>Field Tested</u>	<u>Communications Loop Established</u>	<u>Comments</u>
<u>Ventilation/Gas Monitors</u>				
3HVRRE10A/B				3-4 mos. before testing
3HVRRE19A/B				3-4 mos. before testing
3HVRRE11A/B	X			Being Tested
3HVRRE12A/B	X			Test Starts 5/10
3HVRRE13A/B	X			Being Tested
3HVRRE14A/B	X			Test Starts 5/13
3HVRRE15A/B	X			Test Starts 5/13
3HVRRE16A/B	X			Being Tested
3HVRRE17A/B	X	X		Fuel Building P/G
3HVRRE18A/B	X	X		Waste Building P/G
3HVCRE91A/B				
3CMSRE22A/B				
3HVZRE09A/B				
3HVQRE49A/B	X			Parts Missing
3HVCRE16A/B				

ATTACHMENT 1, Continued

Radiation Monitoring System Status

<u>Instrument Number (Mark No.)</u>	<u>Turned Over</u>	<u>Field Tested</u>	<u>Communications Loop Established</u>	<u>Comments</u>
<hr/> Liquid Process Monitors <hr/>				
3CHSRE69	X			
3CCPRE31				
3CNARE47	X	X		Conductivity meter failed
3DASRE50				
3LWSRE70				
3LWCRE65	X			S/W change required
3CNDRE07	X	X		
3SSRRE08				
3SWPRE60A/B	X			Test Starts 5/17
3GWSRE48				
3ARCRE21				
3MSSRE75	X	X		
3MSSRE76	X			Connector failed
3MSSRE77	X	X		
3MSSRE78	X	X		
3MSSRE79	X			

ATTACHMENT 1, Continued

Radiation Monitoring System Status

<u>Instrument Number (Mark No.)</u>	<u>Turned Over</u>	<u>Field Tested</u>	<u>Communications Loop Established</u>	<u>Comments</u>
<u>Area Monitors</u>				
3RMSRE16	X			Start Test/Calibration 5/13
3RMSRE17	X			Start Test/Calibration 5/13
3RMSRE18	X			Start Test/Calibration 5/13
3RMSRE19	X			Start Test/Calibration 5/13
3RMSRE20	X			Start Test/Calibration 5/13
3RMSRE21	X			Start Test/Calibration 5/13
3RMSRE22	X			Start Test/Calibration 5/13
3RMSRE24	X			Start Test/Calibration 5/13
3RMSRE25	X			Start Test/Calibration 5/13
3RMSRE28	X			Fuel Building Spent Fuel Cask Area
3RMSRE29	X	X		Fuel Building Spent Fuel Cask Area
3RMSRE31	X			Start Test/Calibration 5/13
3RMSRE32	X			Start Test/Calibration 5/13
3RMSRE33	X			Start Test/Calibration 5/13
3RMSRE34	X			Start Test/Calibration 5/13
3RMSRE35	X			Start Test/Calibration 5/13
3RMSRE36	X	X		Fuel Pool
3RMSRE37	X			Start Test/Calibration 5/13

ATTACHMENT 1, Continued

Radiation Monitoring System Status

<u>Instrument Number (Mark No.)</u>	<u>Turned Over</u>	<u>Field Tested</u>	<u>Communications Loop Established</u>	<u>Comments</u>
<u>Area Monitors</u>				
3RMSRE04*	X			Start Test/Calibration 5/13
3RMSRE05*	X			Start Test/Calibration 5/13
3RMSRE41*	X			Start Test/Calibration 5/13
3RMSRE42*	X			Start Test/Calibration 5/13
3RMSRE01	X			Start Test/Calibration 5/13
3RMSRE02	X			Start Test/Calibration 5/13
3RMSRE03	X			Start Test/Calibration 5/13
3RMSRE06	X	X		Fuel Building Decon Area
3RMSRE07	X			Start Test/Calibration 5/13
3RMSRE08	X	X		Spent Fuel Bridge
3RMSRE09	X			Start Test/Calibration 5/13
3RMSRE10	X			Start Test/Calibration 5/13
3RMSRE11	X			Start Test/Calibration 5/13
3RMSRE12	X			Start Test/Calibration 5/13
3RMSRE13	X			Start Test/Calibration 5/13
3RMSRE14	X			Start Test/Calibration 5/13
3RMSRE15	X			Start Test/Calibration 5/13
3RMSRE38	X			Start Test/Calibration 5/13
3RMSRE52	X			Start Test/Calibration 5/13

*Category 1E Monitors

ATTACHMENT II

RMS Surveillance Procedure Summary

<u>Procedure Number</u>	<u>Monitor Identification</u>	<u>Status/Comment</u>
3449B01	3HVR-RE19A/B SLCRS Part/Gas Cal. Proc.	Draft
3450F01	3CND-RE07 Waste Neut. Sump Rad. Monitor Cal.	Draft
3450J01	3LWC-RE65 Regen. Evap. Rad. Monitor Cal.	Draft
3450K01	3SSR-RE08 S/G Blowdown Rad. Monitor Cal.	Draft
3449J01	3HVQ-RE49 ESF Part & Gas Rad. Monitor Cal.	In Development
3449H01	3RMS-RE04/05 Cont. HR Inter. 51'4" CTMT. Cal. Proc.	In Development
3449G01	3HVC-RE16A/B Control Bldg. Inlet Cal. Proc.	In Development
3449A11	3HVR-RE10A/B Vent Part/Gas 66'6" Aux Bldg. Oper. Test	In Development
3449B11	3HVR-RE19A/B SLCRS Part/Gas Oper. Test	In Development
3449D11	35WR-RE60A/B Cont. Recirc. Cool Outlet Oper. Test	In Development
3449E11	3RMS-RE41/42 Fuel Drop Monitor 51'4" Oper. Test	In Development
3449G11	3HVC-RE16A/B Control Bldg. Inlet Oper. Test	In Development
3449H11	3RMS-RE04/05 Cont. HR Inter. 51'4" CTMT. Oper. Test	In Development
3450F11	3CND-RE07 Waste Neut. Sump Rad. Monitor Oper. Test	In Development
3450G11	3DAS-RE50 Turb. Weldg. Floor Drains Rad. Monitor Oper. Test	In Development
3450H11	3LWS-RE70 Liquid Waste Rad. Monitor Oper. Test	In Development
3450J11	3LWC-RE65 Regen. Evap. Rad. Monitor Oper. Test	In Development
3450K11	3SSR-RE08 S/G Blowdown Rad. Monitor Oper. Test	In Development
3449A01	3HVR-RE10A/B Vent Part/Gas 66'6" Aux. Bldg. Cal. Proc.	Draft
3449C01	3CMS-RE22A/B CTMT Atmos. Part/Gas Cal. Proc.	In Development
3449C11	3CMS-RE22A/B CTMT Atmos. Part/Gas Oper. Test	Draft
3449D01	3SWP-RE60A/B TM Recirc. Cool Outlet Cal. Proc.	In Development
3449E01	3RMS-RE41/42 Fuel Drop Monitor 51'4" Cal. Proc.	In Development
3449E21	Fuel Pool Storage Criticality Monitor	PORC Approved
3449E31	Fuel Pool Storage Crit. Monitor Oper. Test	PORC Approved
3449F01	3HVZ-RE09A/B Recom. Cub. A/B Exh. 7'6" Cal. Proc.	In Development
3449F11	3HVZ-RE09A/B Recom. Cub. A/B Exh. 7'6" Oper. Test	In Development

ATTACHMENT II, Continued

RMS Surveillance Procedure Summary

<u>Procedure Number</u>	<u>Monitor Identification</u>	<u>Status/Comment</u>
3449J11	3HVQ-RE49 ESF Part & Gas Rad. Monitor Oper. Test	In Development
3450D01	3MSS-RE75, 76, 77, 78 Main Stm. Rel. Line A-D Cal. Proc.	In Development
3450D11	3MSS-RE75, 76, 77, 78 Main Stm. Rel. Line A-D Oper. Test	In Development
3450E01	3MMS-RE79 Turb. Drv. Aux. FDW Pump Dsch. Cal. Proc.	In Development
3450E11	3MMS-RE79 Turb. Drv. Aux. Fdw. Pmp. Dsch. Oper. Test	In Development
3450G0F	3DAS-RE50 Turb. Bldg. Floor Drains Rad. Monitor Cal.	In Development
3450H01	3LWS-RE70 Liquid Waste Rad. Monitor Cal.	In Development

ATTACHMENT III

RMS Generic Procedure Summary

<u>Generic No. (GPIC)</u>	<u>I&C Procedure No.</u>	<u>Monitor Identification</u>	<u>Status/Comment</u>
20.01	3490B01	Kaman Science (KMG-HRN) Radiation Monitor	Draft
20.02	3490002	Kaman Science (KMG-HRH) Radiation Monitor	Draft
20.03	3490B03	Kaman Science (KMPG) Radiation Monitor	In Review
20.04	3490B04	Kaman Science (KML) Radiation Monitor	Draft
20.05	3490B05	Kaman Science (KMG) Radiation Monitor	Draft
20.06	3490106	Kaman Science Hydrogen Vent Monitor	In Review
20.07	3490B07	Kaman Science Hydrogen Recombiner Monitor	In Review
20.08	3490B08	Kaman Science (KMA-HR) Radiation Monitor	In Review
20.09	3490B09	Kaman Science (KMA-MR) Radiation Monitor	In Review
20.10	3490B16	Kaman Science (KMA) Radiation Monitor	Draft
20.11	3490B11	Kaman Science Fuel Failure Monitor	In Review
20.12	3490B12	Kaman Science Service Water Effluent Monitor	In Review

FIGURE 1

Radiation Protection Organization