

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)										DOCKET NUMBER (2)										PAGE (3)									
Millstone Point Unit 2										0 5 0 0 0 3 3 6										1 OF 0 3									
TITLE (4)																													
Containment Local Leak Rate Limits Exceeded																													
EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																		
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER (5)														
															0 5 0 0 0														
0 3	0 5	8 5	8 5		0 0 3		0 1	0 7	1 7	8 5					0 5 0 0 0														
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																										
POWER LEVEL (10)			20.402(b)					20.405(c)					50.73(a)(2)(iv)					73.71(b)											
			20.405(a)(1)(i)					50.36(c)(1)					50.73(a)(2)(v)					73.71(c)											
			20.405(a)(1)(ii)					50.36(c)(2)					50.73(a)(2)(vi)					OTHER (Specify in Abstract below and in Text: NRC Form 366A)											
			20.405(a)(1)(iii)					50.73(a)(2)(j)					50.73(a)(2)(viii)(A)																
			20.405(a)(1)(iv)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(B)																
			20.405(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(ix)																
LICENSEE CONTACT FOR THIS LER (12)																													
NAME										TELEPHONE NUMBER																			
Steve Stadnick, Plant Engineer, X 4427										AREA CODE 2 0 3 4 4 7 - 1 7 9 1																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																													
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC																			
B	CIC	ISV	F1	30	Y	B	WIK	ISV	M1	20	Y																		
B	VIA	ISV	F1	30	Y																								
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR															
YES (If yes, complete EXPECTED SUBMISSION DATE)																													
X NO																													

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines. (16)

During the 1985 refueling outage the Containment Penetrations subject to Type B and C local leak rate testing exceeded the maximum limit of .6La. Reference Technical Specification 3.6.1.2.a. Actual leakage was calculated to be 19.34%/day.

The combined leakage for those penetrations that contribute to secondary containment bypass leakage also exceeded the maximum limit of .017La. Reference Technical Specification 3.6.1.2.b. Actual leakage was .0226%/day.

Several Reactor Building Closed Cooling Water, Hydrogen Purge, Containment Purge, and Containment Sump System valves were significant contributors to the excessive leakage rate. Repairs to these valves were completed and successful leakage testing was performed. The preventative maintenance program for these valves is being reviewed to reduce leakage rates.

8507310395 850717
PDR ADOCK 05000336
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1) Millstone Point Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 3 6 8 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		— 0 0 3	— 0 1		0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During Type "B" and "C" combined containment penetration leakage rate testing in the 1985 refueling outage the AS FOUND leakage rate exceeded the maximum limit of .6La specified in Technical Specification 3.6.1.2.a. where La is .5 percent by weight of containment air per 24 hours. The actual leakage rate was calculated to be 19.34%/day. A calculated number is reported because the leakage thru the hydrogen purge penetration (number 82) exceeded the capability of the test equipment.

The penetrations that bypass the secondary containment (Penetration Number 14) also had a leakage rate that exceeded the maximum limit of .017La specified in Technical Specification 3.6.1.2.b. The actual leakage rate was .0226%/day.

For penetrations whose leakage rate exceeds the capacity of the flow test equipment, normally a pressure decay type of test would be performed. In this type of test the time required for the pressure in the test volume to decay off is measured and the leakage rate is calculated using the volume. For Penetration 82 the test volume could not be pressurized to the test pressure of 60 PSIG. In this instance the leakage rate was calculated assuming the test volume was pressurized to 60 PSIG and the pressure decayed off to 0 PSIG in a one (1) second time period.

The following penetrations/valves were significant contributors to the excessive leakage rate.

<u>PENETRATION #</u>	<u>VALVE #</u>	<u>TYPE/SIZE</u>	<u>MANUFACTURER/MODEL</u>
82	2-EB-91	Butterfly 6"	Fisher / 9212
82	2-EB-92	Butterfly 6"	Fisher / 9212
26/31	2-RB-28.2B	Butterfly 6"	Fisher / 9222
	2-RB-28.3B	Butterfly 10"	Fisher / 9222
	2-RB-28.1A	Butterfly 10"	Fisher / 9222
27/32			
28/33	2-RB-28.1C	Butterfly 10"	Fisher / 9222
39	2-AC-5	Butterfly 48"	Fisher / 9220
40	2-AC-6	Butterfly 48"	Fisher / 9220
14	2-SSP-16.1	Globe 3"	MASONEILAN/38-20721
14	2-SSP-16.2	Globe 3"	MASONEILAN/38-20721

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1): Millstone Point Unit 2	DOCKET NUMBER (2): 0 5 0 0 0 3 3 6	LER NUMBER (6):			PAGE (3):		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	— 0 0 3	— 0 1	0 3	OF	0 3

TEXT: If more space is required, use additional NRC Form 366A's (17)

CAUSES AND CORRECTIVE ACTIONS:

PENETRATION 82 - The outside containment isolation valve (2-EB-92) was not completely closed by its air operator. The lever arm woodruff key slot for the valve shaft was worn. The lever arm was replaced and a new woodruff key installed. The inner isolation valve (2-EB-91) "T"-ring seat required adjustment.

PENETRATION 26/31 valves 2-RB-28.2B and 2-RB-28.3B and PENETRATION 39 valve 2-AC-5 and PENETRATION 40 valve 2-AC-6. The "T"-ring seat in these valves was replaced and adjusted to bring the leakage within specification.

PENETRATION 27/32 valve 2-RB-28.1A and PENETRATION 28/33 valve 2-RB-28.1C. The "T"-ring seat of these valves was adjusted to bring the leakage within specification limits.

PENETRATION 14 valves 2-SSP-16.1 and 2-SSP-16.2. The seating area of these valves was cleaned, the valves were local leak tested and were within specification limits.

ACTION TO PREVENT RECURRENCE:

For penetrations that contain valves with adjustable "T"-ring seats the preventative maintenance program is being reviewed to determine an effective maintenance schedule that will eliminate leakage thru these valves. For the normal containment sump, (Penetration 14), a design change is being processed to install a modified screen over the sump area. This change will eliminate buildup of debris under the valve seats. Implementation of this change will occur during the next refueling outage.

Previous LERS for similar events: 79-034, 80-032, 82-006, 84-005.

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

General Offices • Selden Street, Berlin, Connecticut

P.O. BOX 270
HARTFORD, CONNECTICUT 06141-0270
(203) 666-6911

July 17, 1985
MP-8003

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Reference: Facility Operating License No. DPR-65
Docket No. 50-336
Reportable Occurrence RO 50-336/85-003, Revision 1

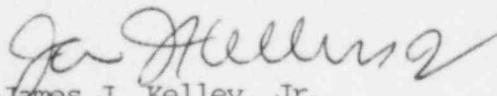
Gentlemen:

This letter forwards the Licensee Event Report 85-003, Revision 1 required to be submitted within thirty (30) days pursuant to paragraph 50.73 (a) (2) (ii), a condition that resulted in a principle safety barrier being degraded.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

FOR: Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

BY: 
James J. Kelley, Jr.
Unit 2 Superintendent
Millstone Nuclear Power Station

WDR/SS:ejl

Attachment: LER 50-336/85-003, Revision 1

cc: Dr. T. E. Murley, Region I

IE22
11