

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Surry Power Station, Unit I										DOCKET NUMBER (2) 0 5 0 0 0 2 8 1 0				PAGE (3) 1 OF 0 3		
TITLE (4) Overpressure Transient																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
													0 5 0 0 0			
0	5	1	2	8	5	0	0	9	0	0	0	6	1	0	8	5
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)														
POWER LEVEL (10) 0 0 0		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)		
		20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)		
		20.405(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vi)				<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 355A)		
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)				SPECIAL REPORT		
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(v)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME										TELEPHONE NUMBER						
R. F. Saunders, Station Manager										8 0 4 3 5 7 1 - 3 1 8 4						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC							
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR		
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO						

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On May 12, 1985, at 0558, the unit I reactor coolant system was solid and at 150°F and 350 psig. Power operated relief valve (, ORV E11S No. RV) PCV-1455C was observed to cycle twice in response to an overpressure condition.

PCV-1145, the normal letdown pressure control valve, failed to properly control pressure at the 350 psig setpoint during solid plant operation. This caused PCV-1455C to lift at approximately 410 psig.

Normal pressure control devices were used to return the primary system pressure to 350 psig. The valve positioner for PCV-1145 was adjusted to maintain letdown pressure.

This event is reportable pursuant to the special reporting requirements of T.S.3.1.G.3 and T.S.6.6.4.g.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1) Surry Power Station, Unit I	DOCKET NUMBER (2) 0 5 0 0 0 2 8 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	— 0 0 9	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

OVERPRESSURE TRANSIENT1. Description of the Event

On May 12, 1985, at 0558, the unit I Reactor Coolant System was solid and at 150°F and 350 psig. Power operated relief valve (PORV EIIS No. RV) PCV-I455C was observed to cycle twice in response to an overpressure condition. Peak primary pressure was approximately 410 psig.

This event occurred when the letdown pressure control valve, PCV-II45, failed to control letdown pressure. The controller for PCV-II45 was operated manually and the pressure transient was terminated using normal pressure control systems.

This event is reportable pursuant to the special reporting requirements of T.S.3.1.G.3 and T.S.6.6.4.g.

2. Probable Consequences

The overpressure mitigating system is intended to mitigate pressure transients when the primary system is less than 350°F and the reactor vessel head is bolted.

PORV PCV-I455C cycled twice to prevent primary pressure from increasing beyond approximately 410 psig.

Since the overpressure mitigating system performed to prevent an overpressure condition and the UFSAR transient analysis shows that only one power operated relief valve, set to open at 435 psig, is needed to prevent exceeding 10CFR50, appendix G limits, an unreviewed safety question was not created and the health and safety of the public were not affected.

3. Cause

PCV-II45, the normal letdown pressure control valve, failed to properly control pressure at the 350 psig setpoint during solid plant operation. This caused PCV-I455C to lift at approximately 410 psig.

A calibration check for PCV-II45 found the positioner out of adjustment.

4. Immediate Corrective Action

The controller for PCV-II45 was placed in manual and charging was isolated. In addition, all pressurizer heaters were secured.

5. Additional Corrective Actions

Normal pressure control devices were used to return the primary system pressure to 350 psig. The valve positioner for PCV-II45 was adjusted to maintain letdown pressure.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Surry Power Station, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 8 0 8 5 - 0 0 9 - 0 0 0 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		OF

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6. Action Taken to Prevent Recurrence

The valve positioner for PCV-II45 was adjusted.

7. Generic Implications

None.



VIRGINIA POWER

Surry Power Station
P. O. Box 315
Surry, Virginia 23883

June 10, 1985

Serial No: 85-014

Docket No: 50-280

License No: DPR-32

U.S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Gentlemen:

Pursuant to Surry Power Station Technical Specifications, Virginia Power hereby submits the following Licensee Event Report for Surry Unit 1.

REPORT NUMBER

85-009-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

R. F. Saunders
Station Manager

Enclosure

cc: Dr. J. Nelson Grace
Regional Administrator
Suite 2900
101 Marietta Street, NW
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