



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TERA

May 27, 1981

Docket No. 50-344



Mr. Bart D. Withers
Vice President Nuclear
Portland General Electric Company
121 S. W. Salmon Street
Portland, Oregon 97204

SUBJECT: ENVIRONMENTAL QUALIFICATION OF SAFETY-RELATED ELECTRICAL EQUIPMENT
RE: TROJAN NUCLEAR PLANT, LICENSE NO. NPF-1

Reference (a) - Order for Modification of License Concerning the Environmental Qualification of Safety-Related Electrical Equipment dated October 24, 1980.

Dear Mr. Withers:

This letter transmits the Safety Evaluation for the Environmental Qualification of Safety-Related Electrical Equipment at your facility. This evaluation was based on your submittals dated March 7, April 25, May 19, and September 5, 1980 and January 13, 1981. Our letter of March 26, 1981, which forwarded our preliminary results, indicated that an item-by-item re-evaluation would be required at a later date. This safety evaluation identifies the specific information required by the staff and the actions, on your part, necessary to comply with Reference (a). We request that you provide the information identified in sections 3 and 4 of this Safety Evaluation to us within 90 days.

This safety evaluation addresses only the safety-related electrical equipment exposed to a harsh environment resulting from an accident (i.e., the IEB 79-Q1B review). Reference (a) requires that all safety-related electrical equipment be qualified by June 30, 1982. The NRC review effort for the equipment in mild environments will be addressed by separate correspondence.

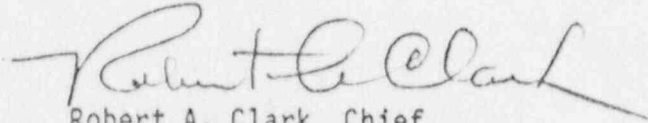
Your response may present alternatives to the staff positions in the evaluation. The staff will consider alternative approaches if these methods are within the DOR Guidelines or NUREG-0588, as appropriate. For example, an acceptable alternative to the NRC staff's temperature criterion used for the service conditions (i.e. Section 3.3) must base that service condition on the FSAR analysis or other NRC approved analysis, provided that the specific analysis, together with reference to the previous NRC acceptance of that analysis, accompanies the 90 day response. In addition, some of the information in the Safety Evaluation may require clarification. Contact your project manager for assistance in these matters.

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It remains your responsibility to assure that the qualification deadline is met for all safety-related electrical equipment. The staff's review of your response to this letter should not delay any action which is required in order to meet the deadline.

Sincerely,

A handwritten signature in dark ink, appearing to read "Robert A. Clark". The signature is fluid and cursive, with a long horizontal stroke extending to the right.

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Enclosures:

1. SER
2. TER

cc w/enclosures:
See next page

Portland General Electric Company

cc: Mr. J. W. Durham, Esquire
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