LOUISIANA POWER & LIGHT / 142 DELARONDE STREET • P.O. BOX 6008
NEW ORLEANS, LOUISIANA 70174-6008 • (504) 366-2345

MIDDLE SOUTH UTILITIES SYSTEM

July 29, 1985

W3P85-1441
A4.05

Director of Nuclear Reactor Regulation
Attention: Mr. G. W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Knighton:

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
INITIAL TEST PROGRAM

Reference: Waterford 3 FSAR, Chapter 14

This submittal is made in accordance with 10 CFR 50.59(b) and the license condition in Section 2.C.10 of the subject license. Reported herewith is a change made to the Waterford 3 Initial Test Program, as described in reference 1, as amended through Amendment No. 36.

The attached marked-up pages 14.2-129 (Amendment 34) and 3.9-109 (Table 3.9-19 Amendment 14) of the Waterford 3 FSAR reflects the change made for test subsection 14.2.12.3.17, Piping Thermal Growth, Vibration, and Shock. A satisfactory evaluation of piping stresses in main steam piping after the revised test will be acceptable instead of an actual instrumented level 4, transient monitoring of the main steam system during turbine trip from 100% power.

The vibration testing consists of a visual inspection of the supports/restraints on the main steam piping after turbine trip from 85% power and a conservative evaluation of the piping stresses using the results of the visual inspection. The evaluation of the results of the visual inspection will also determine if additional transient vibration monitoring of the main steam line is required. An additional transient vibration monitoring of the steam line will be performed if required.

This change to the initial test program does not involve a change in the license technical specifications or an unreviewed safety question. LP&L has conducted and documented the required 10 CFR 50.59 safety evaluation.

A001
1/1

LP&L will provide the FSAR change in accordance with 10 CFR 50.71(e) requirements. The original plus 39 copies of this report transmittal are provided in accordance with 10 CFR 50.59(b).

Very truly yours,

K W Cook

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, D.M. Crutchfield, J.H. Wilson, NRC Resident Inspectors Office, INPO Records Center (J.T. Wheelock), B.W. Churchill, W.M. Stevenson

W3P85-1441

14.2.12.3.17 PIPING THERMAL GROWTH, VIBRATION, AND SHOCK

34

14.2.12.3.17.1 Objectives

To demonstrate that the essential Nuclear Steam Supply System (NSSS) and balance of plant components meet acceptable limits for thermal expansion, vibration, and dynamic response in accordance with design parameters during steady-state and transient conditions.* 15 8

14.2.12.3.17.2 Prerequisites

34

A. Construction activities on the systems to be tested are complete. 18

B. Temporary instrumentation is installed where required. 15

C. Baseline positions and alignment are recorded.

D. Preservice inspection of the snubbers identified in the Technical Specifications (Chapter 16, Section 3/4.7.9) has been completed within 6 months of the start of the individual system preoperational tests. 18

14.2.12.3.17.3 Test Method

34

A. Expansion will be monitored during plant heatups, cooldowns, and operation at various power levels. 8

B. Piping vibration will be monitored during steady-state and transient operation.

C. Demonstrate that the dynamic response during transient operation meets design parameters. For main steam piping it will be demonstrated as per Table 3.9-19. 15

D. On original system heatup and cooldown, verify snubber operability by comparing actual and expected thermal movement at specified temperature intervals. Also verify by observation and/or measurement that adequate swing clearance exists. 18

E. For systems that do not attain design operating temperature, verify by observation and/or calculation that the snubbers will accommodate the predicted thermal movement.

* If additional restraints are installed as a result of the preoperational piping test, the NRC will be advised of the change. In addition, any stress analysis required to verify the change will be maintained on file for NRC review. 34 8

TABLE 3.9-19

VIBRATION TESTING MODES

Flow Modes for Preoperational Vibration Testing

Piping Systems	Steady State	Test Level	Transient	Test Level	Instrumentation Required
Main Steam from Steam Generators to MSIV's	~~100% Power~~ 85% POWER		Turbine trip at ~~100% power~~ 85% POWER *	~~4~~ VISUAL INSPECTION	~~(Will be identified in system test procedure)~~ NONE
	Full flow through atmospheric dump valves, all valves open	1	None		None
Main Steam to Auxiliary Feedwater Pump Turbine	Run at full pump flow	1	AFW turbine trip at full pump flow	1	None
Feedwater and Auxiliary Feedwater	Single AFW Pump Operation for Pumps 2A, 2B, 2C; recirculation	1	Pump start, recirculation mode FW reg valve	1	None
Intake Cooling Water Pumps Discharge Piping	Pump(s) Operating	1	None		None
Component Cooling Water	Pump(s) Operating	1	None		None
Diesel Oil Transfer Pump Discharge Piping	Pump(s) Operating	1	None		None
Steam Generator Blowdown	Flow at normal rate	1	Initiate flow, system cold	1	None
	Flow at maximum rate	1	None		None

* Evaluation of results will determine if additional testing is required.

14

WSPBS-1441