

LICENSEE EVENT REPORT (LER)

Facility Name (1) Byron, Unit 1										Docket Number (2) 0 5 0 0 0 4 5 4				Page (3) 1 of 0 2														
Title (4) FAILURE TO COMPLY WITH TECHNICAL SPECIFICATION 3/4.2.5																												
Event Date (5)			LER Number (6)					Report Date (7)			Other Facilities Involved (8)																	
Month	Day	Year	Year	///	Sequential Number	///	Revision Number	Month	Day	Year	Facility Names		Docket Number(s)															
0	2	2	4	8	5	8	5	0	5	0	0	0	0	6	0	4	8	5										
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)																									
POWER LEVEL (10) 0 4 9			20.402(b)					20.405(c)					50.73(a)(2)(iv)					73.71(b)										
			20.405(a)(1)(i)					50.36(c)(1)					50.73(a)(2)(v)					73.71(c)										
			20.405(a)(1)(ii)					50.36(c)(2)					50.73(a)(2)(vii)					Other (Specify in										
			20.405(a)(1)(iii)					X 50.73(a)(2)(i)					50.73(a)(2)(viii)(A)					Abstract below and										
			20.405(a)(1)(iv)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(B)					in Text)										
			20.405(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(x)															

LICENSEE CONTACT FOR THIS LER (12)																					
Name Roosevelt Groves										Ext. 2606								TELEPHONE NUMBER			
										AREA CODE								8 1 5 2 3 4 - 5 4 4 1			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM		COMPONENT		MANUFAC-TURER		REPORTABLE TO NPRDS				CAUSE	SYSTEM		COMPONENT		MANUFAC-TURER		REPORTABLE TO NPRDS			
D	A	B	/	/	/	/	/	/	/	N											
SUPPLEMENTAL REPORT EXPECTED (14)																					
Yes (If yes, complete EXPECTED SUBMISSION DATE)										X NO								Expected Submission Date (15)			
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																					

On 5-5-85 at 1930 hours, the Unit One Operator discovered that part of the surveillance requirements of Technical Specification 3/4.2.5 were not being addressed. Tave was not documented to be $\leq 591.2^{\circ}\text{F}$ every 12 hours as required since entry into Mode 1.

It was verified that Tave has never exceeded the Tech. Spec. limit and the surveillance requirements for Tave have been permanently added to the daily operating surveillances.

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PDR ADOCK 05000454
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						Page (3)		
		Year	///	Sequential Number	///	Revision Number				
Byron, Unit 1	0 5 0 0 0 4 5 4	8 5	-	0 5 0	-	0 0	0 2	OF	0 2	

TEXT

On May 5, 1985 at 1930 hours, while operating in Mode 1 at 49% power, the Unit Operator noticed the surveillance requirements of Technical Specification 3/4.2.5 were not being met due to a procedure error. Technical Specification 3/4.2.5 requires that DNB related parameters Tav_g and Pr_{zr} pressure, be maintained within the limits of $\leq 591.2^{\circ}\text{F}$ and ≥ 2219 psig, respectively, and that they be verified to be within their limits at least once per 12 hours. It was determined that the step to verify the value of Tav_g had been inadvertently deleted during a revision of the Daily Operating Surveillance. Pressurizer pressure, however, was always verified. This was brought to the attention of the Shift Control Room Supervisor who documented the occurrence.

It was later concluded that the station had been in violation of 3/4.2.5 since 2-24-85, its initial entry into Mode 1.

A control board mounted strip chart recorder continuously records auctioneered high Tav_g. The strip charts from this recorder were reviewed and verified to indicate no instance of recorded Tav_g exceeding 591.2°F .

The strip chart on this recorder additionally is monitored by a reactor operator once per shift who indicates his review by signing the recorder with his initials and time of review. Additionally an annunciator is provided to alarm should any loop Tav_g exceed 590°F . Thus, this event did not significantly affect plant and public safety since the 591.2°F limit was not exceeded. Alarms and automatic reactor trips for plant DNB protection were available to alert the operator and shutdown the plant if DNB limits had been approached.

The immediate corrective action taken was the generation of a temporary procedure change to verify Tav_g below the limit during each shift. This change is also being permanently incorporated in operating procedure 1B05 0.1-1,2,3 (Daily Operating Logs).

Previous similar occurrences: None



Commonwealth Edison
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

June 4, 1985

LTR: BYRON 85-0827

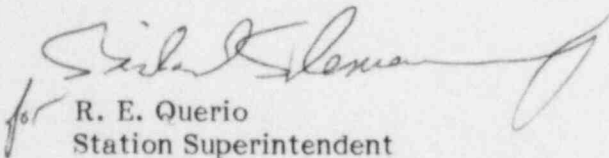
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR 50.73(a)(2)(i) which requires a 30 day written report.

This report is number 85-050-00; Docket No. 50-454.

Very truly yours,



for R. E. Querio
Station Superintendent
Byron Nuclear Power Station

REQ/gt

Enclosure: Licensee Event Report No. 85-050-00

cc: J. G. Keppler, NRC Region III Administrator
J. Hinds, NRC Resident Inspector
INPO Record Center
CECO Distribution List

#3/017

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