

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MAY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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## TABLE OF CONTENTS

- I. INTRODUCTION
- II. MONTHLY REPORT FOR UNIT ONE
  - A. Summary of Operating Experience
  - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
    - 1. Amendments to Facility License or Technical Specifications
    - 2. Facility or Procedure Changes Requiring NRC Approval
    - 3. Tests and Experiments Requiring NRC Approval
    - 4. Corrective Maintenance of Safety Related Equipment
  - C. LICENSEE EVENT REPORTS
  - D. DATA TABULATIONS
    - 1. Operating Data Report
    - 2. Average Daily Unit Power Level
    - 3. Unit Shutdowns and Power Reductions
  - E. UNIQUE REPORTING REQUIREMENTS
    - 1. Main Steam Relief Valve Operations
    - 2. ECCS System Outages
    - 3. Off-Site Dose Calculation Manual Changes
    - 4. Major Changes to Radioactive Waste Treatment System

## I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to an artificial cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982, and commercial power operation was commenced on January 1, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

## II. MONTHLY REPORT FOR UNIT ONE

### A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

May 1-31

May 1, 0000 hours. Reactor critical at 98% power, generator on-line.  
May 1, 0700 hours. Reactor power reduced to 78% for LTS-1100-4, Scram Timing for CRD 22-59.  
May 2, 0700 hours. Reactor power restored to 96%.  
May 6, 1645 hours. Reactor power reduced to 84%, to manipulate control rods.  
May 7, 0700 hours. Reactor power restored to 96%.  
May 10, 2300 hours. Reactor power at 95%.  
May 11, 0200 hours. Reactor power reduced to 76% to manipulate control rods.  
May 11, 2300 hours. Reactor power restored to 94%.  
May 19, 0200 hours. Reactor power reduced to 58% to allow maintenance on heater drains.  
May 20, 1500 hours. Reactor power restored to 89%.  
May 20, 1942 hours. Reactor power reduced to 86% due to Number 3 Control Valve oscillations.  
May 24, 0700 hours. Reactor power at 84%.  
May 24, 0855 hours. Reactor power reduced to 74% due to trip of "B" circulating Water Pump.  
May 24, 1500 hours. Reactor power at 84%.  
May 30, 2352 hours. Reactor power reduced to 78% for surveillance LOS-AA-W1.  
May 31, 0700 hours. Reactor power at 84%.  
May 31, 1945 hours. Reactor manually scrammed following flooding of Lake Screen House when "B" Circulating Water Pump Discharge Line leaked.

The reactor was critical for 739.8 hours during May.

B.  
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PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED  
MAINTENANCE.

1. Amendments to facility license or Technical Specification.

Three amendments were made to the Unit one operating license and Technical Specifications during May.

Amendment 21 modifies the calculational technique used in determining duct heater performance for the standby Gas Treatment System. Amendment 22 changes the channel check requirements to accommodate new instruments installed by Environmental Qualification modifications. Amendment 23 incorporates the requirements of 10 CFR 50.73.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

| WORK REQUEST | COMPONENT  | CAUSE OF MALFUNCTION                                      | RESULTS AND EFFECTS  | CORRECTIVE ACTION<br>ON SAFE OPERATION  |
|--------------|--|---|--|---|
| L41793       | Division I Switch-<br>gear Heat Removal<br>Outside Air Damper. | Faulty Actuator   | Damper would not stroke                                      | Sent actuator to vendor<br>for repair. Reinstalled<br>and verified operability. |
| L45261       | Control Room Zone<br>Mixing Damper "A".                        | Actuator Shaft is not<br>coupled to damper.               | Damper would not operate.                                    | Connected shaft to<br>damper.   |
| L48324       | Ammonia detector "B"<br>for "B" Control Room<br>Ventilation.   | Out-of-calibration.                                       | Detector alarmed and actuated<br>ESF's.                      | Recalibrated detector.  |
| L48347       | Sequence of Events<br>Recorder.                                | Swapped Alarm messages for<br>Chlorine and Ammonia Alarms | Wrong Alarm messages printed.                                | Corrected Alarm<br>messages.  |
| L48478       | Unit One Stand-by<br>Gas Treatment System<br>Recorder.         | Flow Recorder out-of-<br>calibration.                     | Wrong flow indication.                                       | Recalibrated Recorder<br>and adjusted recorder<br>scale.                        |
| L48535       | Scram pilot valve for<br>CRD 22-59.                            | Valve hung up in an inter-<br>mediate position.           | Inadvertantly Scrammed this<br>rod during half-scam testing. | Replaced scram pilot<br>valves.   |
| L48580       | 1A RHR Service<br>Water Pump.                                  | Faulty Motor.   | Pump would not work as designed.                             | Replaced motor with<br>motor from 2B RHR<br>service water pump.                 |
| L48651       | Ammonia detector "B"<br>for "A" Control Room<br>Ventilation.   | Detector out-of-calibration                               | Detector alarmed and actuated<br>ESF's.                      | Recalibrated detector.  |
| L48846       | "A" Rod Block Monitor  | Blown fuse.   | Generated Spurious Rod Blocks.                               | Replaced fuse.  |

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

| WORK REQUEST | COMPONENT                                   | CAUSE OF MALFUNCTION               | RESULTS AND EFFECTS  | CORRECTIVE ACTION<br>ON SAFE OPERATION  |
|--------------|---|------------------------------------|--|---|
| L48918       | "A" Standby Liquid<br>Control Relief Valve. | Valve Leaked through port<br>hole. | Injection of Liquid Control<br>solution would have been<br>slightly slower or otherwise,<br>if required. | Cleaned and adjusted<br>valve. Installed new<br>bellows and lapped the<br>seat. |
| L49054       | HCU Accumulator for<br>CRD 22-23.           | Leaky fill valve.                  | Potential to cause failure to<br>scram this rod if combined with<br>other events.                        | Replaced valve.   |
| L49059       | "A" Rod Block Monitor.                      | Faulty power supply.               | Failed downscale.  | Replaced power supply and<br>fuses, tested.                                     |

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, May 1 through May 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

| <u>Licensee Event Report Number</u> | <u>Date</u> | <u>Title of Occurrence</u>                       |
|-------------------------------------|-------------|--|
| 85-037-00                           | 4-17-85     | Division II ADS Inop.                            |
| 85-038-00                           | 4-12-85     | Ammonia Detector Actuation.                      |
| 85-039-00                           | 4-19-85     | Ammonia Detector Actuation.                      |
| 85-040-00                           | 4-23-85     | Ammonia Detector Failure.                        |
| 85-041-00                           | 5-5-85      | Spurious Auto-start of<br>Emergency Make-up Fan. |
| 85-042-00                           | 5-5-85      | Ammonia/Chlorine Detector<br>Actuations.         |
| 85-043-00                           | 5-8 85      | Chlorine Detector Actuation.                     |



D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORTDOCKET NO. 050-373UNIT LaSalle OneDATE June 10, 1985COMPLETED BY Richard J. RohrerTELEPHONE (815)357-6761

## OPERATING STATUS

1. REPORTING PERIOD: MAY, 1985 GROSS HOURS IN REPORTING PERIOD: 744
  2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
  3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
  4. REASONS FOR RESTRICTION (IF ANY): N/A
- |   | THIS MONTH     | YR TO DATE      | CUMULATIVE      |
|---|----------------|-----------------|-----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL     | <u>739.8</u>   | <u>3042.0</u>   | <u>9323</u>     |
| 6. REACTOR RESERVE SHUTDOWN HOURS           | <u>4.2</u>     | <u>102.9</u>    | <u>1269</u>     |
| 7. HOURS GENERATOR ON LINE                  | <u>739.8</u>   | <u>2969.2</u>   | <u>9025</u>     |
| 8. UNIT RESERVE SHUTDOWN HOURS              | <u>0.0</u>     | <u>0.0</u>      | <u>0.0</u>      |
| 9. GROSS THERMAL ENERGY GENERATED (MWH)     | <u>2184310</u> | <u>8433887</u>  | <u>25257176</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH)      | <u>724455</u>  | <u>27887650</u> | <u>8259408</u>  |
| 11. NET ELEC. ENERGY GENERATED (MWH)        | <u>703230</u>  | <u>2687914</u>  | <u>7882976</u>  |
| 12. REACTOR SERVICE FACTOR                  | <u>99.4%</u>   | <u>83.4%</u>    | <u>75.0%</u>    |
| 13. REACTOR AVAILABILITY FACTOR             | <u>100%</u>    | <u>86.2%</u>    | <u>85.2%</u>    |
| 14. UNIT SERVICE FACTOR                     | <u>99.4%</u>   | <u>81.4%</u>    | <u>72.6%</u>    |
| 15. UNIT AVAILABILITY FACTOR                | <u>99.4%</u>   | <u>81.4%</u>    | <u>72.6%</u>    |
| 16. UNIT CAPACITY FACTOR (USING MDC)        | <u>91.2%</u>   | <u>71.1%</u>    | <u>61.2%</u>    |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>87.7%</u>   | <u>68.4%</u>    | <u>58.8%</u>    |
| 18. UNIT FORCED OUTAGE RATE                 | <u>0.6%</u>    | <u>12.2%</u>    | <u>13.9%</u>    |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)  
Unit one is scheduled for a refueling, maintenance, modification, and surveillance outage beginning September 3, 1985 and lasting 26 weeks.
  20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: June 9, 1985

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373

UNIT: LASALLE ONE

DATE: June 10, 1985

COMPLETED BY: Richard J. Rohrer

TELEPHONE: (815) 357-6761

MONTH: MAY, 1985

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

|                 |                 |
|-----------------|-----------------|
| 1. <u>924</u>   | 17. <u>1043</u> |
| 2. <u>1040</u>  | 18. <u>1032</u> |
| 3. <u>1042</u>  | 19. <u>766</u>  |
| 4. <u>975</u>   | 20. <u>915</u>  |
| 5. <u>875</u>   | 21. <u>879</u>  |
| 6. <u>943</u>   | 22. <u>905</u>  |
| 7. <u>1032</u>  | 23. <u>904</u>  |
| 8. <u>1034</u>  | 24. <u>878</u>  |
| 9. <u>1036</u>  | 25. <u>877</u>  |
| 10. <u>1030</u> | 26. <u>876</u>  |
| 11. <u>926</u>  | 27. <u>873</u>  |
| 12. <u>1030</u> | 28. <u>874</u>  |
| 13. <u>1035</u> | 29. <u>876</u>  |
| 14. <u>1025</u> | 30. <u>883</u>  |
| 15. <u>1020</u> | 31. <u>725</u>  |
| 16. <u>1029</u> |                 |

ATTACHMENT E  
3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373  
UNIT NAME LaSalle One  
DATE JUNE 10, 1985  
COMPLETED BY Richard J. Rohrer  
TELEPHONE (815)357-6761

REPORT MONTH MAY 1985

| NO. | DATE   | TYPE<br>F: FORCED<br>S: SCHEDULED | DURATION<br>(HOURS) | REASON | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS                                    |
|-----|--------|-----------------------------------|---------------------|--------|--|---|
|     |        |                                   |                     |        |  |   |
| 9   | 850519 | S                                 | 0.00                | B      | 1  | Power reduced to allow<br>maintenance of heater<br>drains.        |
| 10  | 850531 | F                                 | 4.25                | A.     | 2  | Manually scrambled<br>following flooding of<br>Lake Screen House. |

## E. UNIQUE REPORTING REQUIREMENTS

### 1. Safety/Relief valve operations for Unit One.

| <u>DATE</u> | <u>VALVES<br/>ACTUATED</u> | <u>NO &amp; TYPE<br/>ACTUATION</u> | <u>PLANT<br/>CONDITION</u> | <u>DESCRIPTION<br/>OF EVENT</u> |
|-------------|----------------------------|------------------------------------|----------------------------|---------------------------------|
| 5-31-85     | 1B21-F013A                 | 1 Manual                           | 620 psig                   | Manually opened                 |
|             | 1B21-F013B                 | 1 Manual                           | 545 psig                   | successively to provide         |
|             | 1B21-F013C                 | 1 Manual                           | 530 psig                   | controlled cool-down of         |
|             | 1B21-F013D                 | 1 Manual                           | 595 psig                   | the reactor following           |
|             | 1B21-F013E                 | 1 Manual                           | 510 psig                   | Main Steam Line Isolation.      |
|             | 1B21-F013F                 | 1 Manual                           | 460 psig                   |                                 |
|             | 1B21-F013G                 | 1 Manual                           | 435 psig                   |                                 |
|             | 1B21-F013H                 | 1 Manual                           | 435 psig                   |                                 |
|             | 1B21-F013K                 | 1 Manual                           | 560 psig                   |                                 |
|             | 1B21-F013L                 | 1 Manual                           | 275 psig                   |                                 |

Valve 1B21-F013L opened at 2330 on May 31, 1985 and closed at 0030 on June 1, 1985. Other valves (1B21-F013 M,N,P,Q,R,S) were opened and will be reported in the Monthly Report for June.

## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

| <u>OUTAGE NO.</u> | <u>EQUIPMENT</u>                                      | <u>PURPOSE OF OUTAGE</u> |
|-------------------|---|--------------------------|
| 0-323-85          | 0 Diesel Generator<br>Fuel Oil Transfer Pump.         | Relocate Cables          |
| 1-399-85          | 1A RHR Service Water<br>Pump.                         | Repair.                  |
| 1-400-85          | 1B RHR Service Water<br>Discharge Valve.              | Prevent Pump<br>Runout.  |
| 1-425-85          | 1B Diesel Generator.                                  | Change Turbo filter.     |
| 1-447-85          | 1A RHR Service Water<br>Process Radiation<br>Monitor. | Repair Seal Leak.        |
| 1-448-85          | High Pressure Core<br>Spray Full-Flow Test<br>Valve.  | Repair Leak.             |
| 1-449-85          | 1A RHR Process Radiation<br>Monitor.                  | Work on Pump Motor.      |

## 3. Off-Site Dose Calculation Manual

Revision 11 was made to the Off-site Dose Calculation Manual, incorporating changes requested by the Nuclear Regulatory Commission.

## 4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MAY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18



## TABLE OF CONTENTS

- I. INTRODUCTION
- II. MONTHLY REPORT FOR UNIT TWO
  - A. Summary of Operating Experience
  - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
    - 1. Amendments to Facility License or Technical Specifications
    - 2. Facility or Procedure Changes Requiring NRC Approval
    - 3. Tests and Experiments Requiring NRC Approval
    - 4. Corrective Maintenance of Safety Related Equipment
  - C. LICENSEE EVENT REPORTS
  - D. DATA TABULATIONS
    - 1. Operating Data Report
    - 2. Average Daily Unit Power Level
    - 3. Unit Shutdowns and Power Reductions
  - E. UNIQUE REPORTING REQUIREMENTS
    - 1. Safety/Relief Valve Operations
    - 2. ECCS System Outages
    - 3. Off-Site Dose Calculation Manual Changes
    - 4. Major Changes to Radioactive Waste Treatment System

1. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to an artificial cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984, and commercial power operation was commenced on June 19, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

MAY 1-31

The reactor was subcritical, and the generator was off-line for the entire month of MAY. Unit Two is in a scheduled outage for maintenance, testing, and modifications.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

Three amendments were made to the Unit two operating license and Technical Specifications during May.

Amendment 9 modifies the calculational technique used in determining duct heater performance for the standby Gas Treatment System. Amendment 10 changes the channel check requirements to accommodate new instruments installed by Environmental Qualification modifications. Amendment 11 incorporates the requirements of 10CFR50.73.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1  
CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

| WORK REQUEST | COMPONENT   | CAUSE OF MALFUNCTION                   | RESULTS AND EFFECTS   | CORRECTIVE ACTION<br>ON SAFE OPERATION |
|--------------|---|--|---|--|
| L40557       | HCU Accumulator for<br>CRD 42-03.                                   | Leaking Scram Pilot valve<br>cap nut.  | Potential to cause failure to<br>scram this rod if combined with<br>other events. | Changed "O-ring".                      |
| L41791       | Main steam line out-<br>board drain valve.                          | Dirty Contacts on limit<br>switch.     | No indication of valve's position.  | Cleaned and adjusted<br>limit switch.  |
| L44542       | "A" RHR Suction Valve.  | Torque switches at wrong<br>setting.   | Valve tripped its thermals when<br>cycled.  | Adjusted Torque and<br>limit switches. |
| L44755       | Steam line downstream<br>drain.                                     | Faulty torque switch.                  | Valve would not close from<br>remote handswitch.                                  | Replaced torque switch.                |
| L46397       | "4B" Standby Liquid<br>Control Explosive<br>Valve Continuity Alarm. | Wired incorrectly and blown<br>fuse.   | Continuity Meter would not<br>function properly.                                  | Corrected wiring and<br>replaced fuse. |
| L47010       | Division I Post-<br>LOCA Oxygen Monitor.                            | Out-of-Calibration.                    | Incorrect reading.  | Recalibrated Monitor.                  |
| L47011       | Division II Post-<br>LOCA Oxygen Monitor.                           | Out-of-calibration.                    | Incorrect Reading.  | Recalibrated Monitor.                  |
| L47273       | Division III Batteries.   | Faulty cell.                           | Degraded battery performance.   | Replaced faulty cell.                  |
| L47530       | Drywell Equipment<br>Drain Inboard<br>Isolation Valve.              | Less-than-desirable spring<br>tension. | Valve failed local leak rate<br>test.   | Increased spring tension.              |
| L47578       | Drywell Equipment<br>Drain Sump Valve.                              | Less-than-desired spring<br>tension.   | Valve failed local leak rate<br>test.   | Increased spring tension.              |

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

| WORK REQUEST | COMPONENT   | CAUSE OF MALFUNCTION                      | RESULTS AND EFFECTS  | CORRECTIVE ACTION<br>ON SAFE OPERATION    |
|--------------|---|---|--|---|
| L47708       | "A" Standby Liquid Control Explosive Valve Continuity Meter.        | Faulty Relay.                             | Meter falsely indicated complete circuit.  | Replaced relay.                           |
| L47773       | RCIC Ambient Temperature Element.                                   | Cables mislabeled and landed incorrectly. | Instrument would not respond.  | Relabeled cables and landed correctly.    |
| L47839       | "B" Primary Containment Chill Water Supply Inboard Isolation Valve. | Loose contacts on torque switch.          | Valve would not open from remote handswitch.   | Tightened torque switch contacts.         |
| L48149       | 125 Volt DC Battery Charger.  | Blown fuses.                              | No Voltage indication in control room.   | Replaced fuses.                           |
| L48329       | Outboard Feed Water Check Valve "A".                                | Actuating cylinder was binding.           | Would not pull valve to closed position.   | Cleaned and lubricated actuator cylinder. |
| L48375       | Scram Discharge Volume Vent Valve.                                  | Air leak on supply to valve actuator.     | Possibility to increase the length of time required to insert rods during a scram if combined with other events. | Cleaned coupling in air line.             |
| L48505       | Average Power Range Monitor "F".                                    | Blown fuse.                               | Half of downscale lights were out.   | Replaced fuse.                            |
| L48628       | Reactor Water Clen-up Inboard Isolation Valve.                      | Broken torque switch.                     | Valve would not completely close with motor operator.  | Replaced torque switch.                   |

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

| WORK REQUEST | COMPONENT  | CAUSE OF MALFUNCTION       | RESULTS AND EFFECTS  | CORRECTIVE ACTION<br>ON SAFE OPERATION                      |
|--------------|--|----------------------------|--|---|
| L48654       | Standby Gas Treatment<br>System Deluge Valves.     | Valves software degraded.  | Valves leaked by, causing char-<br>coal filter to get wet.                 | Replaced software in<br>valves.                             |
| L48745       | Reactor Building<br>Ventilation Exhaust<br>Damper. | Stuck open contactor.      | Damper tripped thermals when<br>cycling open. Burned out the<br>the motor. | Replaced motor, open<br>contactor, and mechanical<br>relay. |
| L49053       | Diesel Generator "2B"                              | Faulty K33 relay.          | Diesel Generator tripped on low<br>oil pressure shortly after<br>starting. | Replaced relay K33.   |
| L49113       | LPCS Minimum Flow<br>Switch.                       | Switch out-of-calibration. | Minimum flow valve did not<br>automatically open when required.            | Recalibrated switch.  |

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, May 1 through May 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

| <u>Licensee Event Report Number</u> | <u>Date</u> | <u>Title of Occurrence</u>                    |
|-------------------------------------|-------------|---|
| 85-019-00                           | 4-12-85     | Primary Containment Group I Isolation Signal. |
| 85-020-00                           | 4-24-85     | Missed Service Water Sample.                  |
| 85-021-00                           | 5-3-85      | RCIC Temperature Leak Detection Miswired.     |
| 85-022-00                           | 5-16-85     | Reactor Scram.                                |



D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

## 1. OPERATING DATA REPORT

DOCKET NO. 050-374UNIT LaSalle TwoDATE June 10, 1985COMPLETED BY Richard J. RohrerTELEPHONE (815)357-6761

## OPERATING STATUS

1. REPORTING PERIOD: May, 1985 GROSS HOURS IN REPORTING PERIOD: 744
  2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
  3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
  4. REASONS FOR RESTRICTION (IF ANY): N/A
- |  | THIS MONTH | YR TO DATE | CUMULATIVE |
|--|------------|------------|------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL    | 0.0        | 1399.8     | 3011.6     |
| 6. REACTOR RESERVE SHUTDOWN HOURS          | 0.0        | 0.0        | 125.3      |
| 7. HOURS GENERATOR ON LINE                 | 0.0        | 1397.3     | 2934.7     |
| 8. UNIT RESERVE SHUTDOWN HOURS             | 0.0        | 0.0        | 0.0        |
| 9. GROSS THERMAL ENERGY GENERATED (MWH)    | 0.0        | 4387385    | 8894977    |
| 10. GROSS ELEC. ENERGY GENERATED (MWH)     | 0.0        | 1460387    | 2945373    |
| 11. NET ELEC. ENERGY GENERATED (MWH)       | -9397      | 1382193    | 2774510    |
| 12. REACTOR SERVICE FACTOR                 | 0.0%       | 38.4%      | 55.5%      |
| 13. REACTOR AVAILABILITY FACTOR            | 0.0%       | 38.4%      | 57.8%      |
| 14. UNIT SERVICE FACTOR                    | 0.0%       | 38.3%      | 54.1%      |
| 15. UNIT AVAILABILITY FACTOR               | 0.0%       | 38.3%      | 54.1%      |
| 16. UNIT CAPACITY FACTOR (USING MDC)       | -1.2%      | 36.6%      | 49.4%      |
| 17. UNIT CAPACITY FACTOR(USING DESIGN MWe) | -1.2%      | 35.2%      | 47.5%      |
| 18. UNIT FORCED OUTAGE RATE                | 0.0%       | 0.0%       | 4.4%       |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
An outage for maintenance and surveillance was begun at 0520 on February 28, 1985.
  20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP June 14, 1985

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374  
 UNIT: LASALLE TWO  
 DATE: June 10, 1985  
 COMPLETED BY: Richard J. Rohrer  
 TELEPHONE: (815) 357-6761  
 MONTH: May 1985

DAY AVERAGE DAILY POWER LEVEL  
 (MWe-Net)

1. -11  
 2. -11  
 3. -11  
 4. -11  
 5. -11  
 6. -11  
 7. -11  
 8. -11  
 9. -11  
 10. -11  
 11. -11  
 12. -12  
 13. -13  
 14. -13  
 15. -13  
 16. -13

DAY AVERAGE DAILY POWER LEVEL  
 (MWe-Net)

17. -14  
 18. -17  
 19. -15  
 20. -15  
 21. -14  
 22. -12  
 23. -12  
 24. -12  
 25. -13  
 26. -14  
 27. -15  
 28. -14  
 29. -14  
 30. -14  
 31. -13

## ATTACHMENT E

## 3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374

UNIT NAME LaSalle TwoDATE JUNE 10, 1985COMPLETED BY Richard J. RohrerTELEPHONE (815)357-6761REPORT MONTH MAY 1985

| NO. | DATE   | TYPE                      | DURATION<br>(HOURS) | REASON | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS                                    |
|-----|--------|---------------------------|---------------------|--------|--|---|
|     |        | F: FORCED<br>S: SCHEDULED |                     |        |  |   |
| 3   | 850228 | S                         | 744.00              | B      | 2  | Maintenance and<br>Surveillance Outage<br>begun 2-28-85 continues |

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

| <u>DATE</u> | <u>VALVES<br/>ACTUATED</u> | <u>NO &amp; TYPE<br/>ACTUATIONS</u> | <u>PLANT<br/>CONDITION</u> | <u>DESCRIPTION<br/>OF EVENT</u>          |
|-------------|----------------------------|-------------------------------------|----------------------------|--|
| 5-23-85     | 2B21-F013D                 | 1 Automatic                         | Cold Shutdown              | Spurious<br>Actuation, cause<br>unknown. |

## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

| <u>OUTAGE NO.</u> | <u>EQUIPMENT</u>                                    | <u>PURPOSE OF OUTAGE</u>                                       |
|-------------------|---|--|
| 2-702-85          | 2E12-F004A  | Minor Maintenance.   |
| 2-706-85          | 2B Diesel Generator<br>Fuel Oil Transfer Valve.     | Assist Mechanical.<br>Maintenance Department.                  |
| 2-707-85          | Low Pressure Core Spray.                            | Check Coupling.  |
| 2-708-85          | 2A Diesel Generator "B"<br>Air Compressor.          | Repair A/C Poppet<br>relief.                                   |
| 2-710-85          | 2B RHR Service Water Pump                           | Remove Motor.  |
| 2-714-85          | Low Pressure Core Spray Water<br>Leg Pump.          | Clean and Lubricate<br>Coupling.                               |
| 2-740-85          | High Pressure Core Spray<br>Spool Piece.            | Remove Spool piece.  |
| 2-747-85          | 2B Diesel Generator Fuel Oil<br>Transfer Valve.     | Repair Leak.   |
| 2-748-85          | 2E12-C300C.   | Repair Motor.  |
| 2-755-85          | 2E12-F050A  | Assist Mechanical Maintenance<br>Department.                   |
| 2-756-85          | 2E12-F050A  | Reapair Binding.   |
| 2-760-85          | 2E12-F064A  | Repack Valve.  |
| 2-761-85          | "A" RHR Pump Breaker.                               | Install Rubber Boots.  |
| 2-763-85          | RCIC Pump.  | Repair Drain Line.   |
| 2-772-85          | "B" RHR Service Water Process<br>Radiation Monitor. | Decontamination.   |
| 2-774-85          | 2B Diesel Generator.                                | Maintenance and Surveillance.                                  |
| 2-785-85          | RHR-Shutdown Cooling.                               | Surveillance.  |
| 2-792-85          | 2A RHR Pump.  | Check Oil Level.   |
| 2-793-85          | 2A RHR Pump.  | Modification to Service<br>Water Process Radiation<br>Monitor. |

| <u>OUTAGE NO.</u> | <u>EQUIPMENT</u>  | <u>PURPOSE OF OUTAGE</u> |
|-------------------|---|--------------------------|
| 2-794-85          | 2A RHR.   | Surveillance.            |
| 2-799-85          | 2D0014  | Troubleshooting.         |
| 2-801-85          | 2A RHR Service Water Process<br>Radiation Monitor.      | Decontamination.         |
| 2-802-85          | 2E22-F038.  | Testing.                 |
| 2-803-85          | "A" LPCI and LPCS Manual<br>Injection Valves.           | Surveillance.            |
| 2-805-85          | High Pressure Core Spray.                               | Isolate System.          |
| 2-806-85          | 2B Diesel Generator.                                    | Repair Immersion Heater. |
| 2-808-85          | High Pressure Core Spray<br>Full-Flow Line Heat Trace.  | Determinate.             |
| 2-810-85          | LPCS Water Leg Pump.                                    | Repair Check Valve.      |
| 2-815-85          | Division I Water Leg Pump.                              | Repair Discharge Valve.  |
| 2-816-85          | 2A RHR Service Water Process<br>Radiation Monitor Skid. | Relocate.                |
| 2-822-85          | High Pressure Core Spray Pump.                          | Check Breaker.           |

\* \* \* 3. Off-Site Dose Calculation Manual

Revision 11 was made to the Off-site Dose Calculation Manual, incorporating changes requested by the Nuclear Regulatory Commission.

4. Radioactive Waste Treatment Systems.

There were no changes to the radioactive waste treatment system during this reporting period.





Commonwealth Edison  
LaSalle County Nuclear Station  
Rural Route #1, Box 220  
Marseilles, Illinois 61341  
Telephone 815/357-6761

June 10, 1985

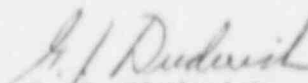
Director, Office of Management Information  
and Program Control  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering  
LaSalle County Nuclear Power Station for the period covering May 1 through May  
31, 1985.

Very truly yours,

  
G. J. Diederich  
Station Manager  
LaSalle County Station

GJD/RJR/crh

Enclosure

xc: J. G. Keppler, NRC, Region III  
NRC Resident Inspector LaSalle  
Gary Wright, Ill. Dept. of Nuclear Safety  
D. P. Galle, CECO  
D. L. Farrar, CECO  
INPO Records Center  
Ron A. Johnson, PIP Coordinator SNED  
J. E. Ellis, GE Resident  
J. M. Nowicki, Asst. Comptroller  
H. E. Bliss, Nuclear Fuel Services Manager  
C. F. Dillon, Senior Financial Coordinator

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11