

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Waterford 3 Steam Electric Station										DOCKET NUMBER (2) 0 5 0 0 0 3 8 2					PAGE (3) 1 OF 0 3										
TITLE (4) Reactor Trip and Reactor Coolant System Leak																									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES N/A				DOCKET NUMBER(S) 0 5 0 0 0												
0	5	1	1	8	5	8	5	0	1	8	0	0	0	6	1	0	8	5	N/A				0 5 0 0 0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																							
1		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)											
POWER LEVEL (10)		0 6 5				20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)			73.71(c)								
		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.405(a)(1)(iii)				<input checked="" type="checkbox"/> 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)															
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)															
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)															
LICENSEE CONTACT FOR THIS LER (12)																									
NAME L. Myers, Operations Superintendent										TELEPHONE NUMBER 5 1 0 4 4 6 4 - 3 1 1 1 8															
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC															
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR									
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

On May 11, 1985 Waterford 3 Steam Electric Station was at 65% reactor power when Operations Personnel calculated an unidentified leakage of 6.1 gallons per minute. An Unusual Event was declared at 0710 hours and at 0945 hours Operations personnel initiated an orderly plant shutdown. Reactor power was gradually reduced until 1543 hours when an uncomplicated reactor trip occurred from approximately 1% power due to a high water level in Steam Generator number 2. At 2143 hours on May 13, 1985 Waterford 3 entered mode 5 (cold shutdown).

The leak originated in the packing glands of pressurizer spray valves RC-301A, B. The packing glands for both valves were modified to improve their reliability.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Waterford 3 Steam Electric Station	0 5 1 0 0 0 3 8 2	8 5	— 0 1 8	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

NARRATIVE

On May 11, 1985 Waterford 3 Steam Electric Station was at 65% reactor power when Operations Personnel, upon observing the frequent automatic makeup of the Volume Control Tank and the alarming of the Containment Sump Level Monitor, initiated procedure OP-903-024, RCS Inventory Balance. At 0545 hours Waterford 3, upon calculating an unidentified leakage of 6.1 gallons per minute, entered Technical Specification Action Statement 3.4.5.2b. At 0710 hours Operations Personnel declared an Unusual Event in accordance with the Emergency Plan Implementing Procedures, and subsequently, at 0945 hours an orderly plant shutdown was commenced. However, at 1543 hours, while at 1% reactor power, Operations Personnel, while manually controlling Steam Generator water levels, overfed Steam Generator number 2 resulting in an uncomplicated reactor trip. Operations Personnel immediately entered Emergency Procedure OP-902-000, Emergency Entry Procedure and OP-902-001, Uncomplicated Reactor Trip Recovery Procedure. A plant cooldown continued, and at 2143 hours on May 13, 1985 Waterford 3 Steam Electric Station entered mode 5 (cold shutdown).

The leak originated in the packing glands of pressurizer spray valves RC-301A, and -301B.

SAFETY CONSEQUENCES AND IMPLICATION

The above events resulted in a leak from the pressurizer spray valves, and an automatic actuation of the Reactor Protective System during startup testing. Since all of the primary system water was contained within the Reactor Containment Building, and since the Reactor Protective System, along with the Control Element Assemblies, functioned as designed, the event in no way placed Waterford 3 in a degraded safety condition.

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		0 1 8	—	0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTION

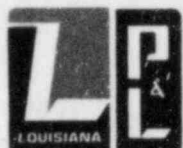
In order to reduce the likelihood of further packing gland leakage, Station Modification 926 was issued to improve the reliability of the packing design by both utilizing a different type of packing, and adding additional rings of packing to the present configuration.

SIMILAR EVENTS

NONE

PLANT CONTACT

L. Myers, Operations Superintendent, 504/464-3118



MIDDLE SOUTH
UTILITIES SYSTEM

LOUISIANA
POWER & LIGHT

142 DELARONDE STREET • P.O. BOX 8008
NEW ORLEANS, LOUISIANA 70174-8008 • (504) 388-2345

June 10, 1985

W3P85-1409
A4.05

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-018-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(i) and 10CFR50.73(a)(2)(iv).

Very truly yours,

R.P. Barkhurst
Plant Manager - Nuclear

RPB:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, D.M. Crutchfield, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), B.W. Churchill,
W.M. Stevenson

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