NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Wolf Creek Generating Station	0 5 0 0 0 4 8 2	1 OF 0 2

TITLE (4)

ESF Actuation - Safety Injection and Main Steamline Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 5	0 6	8 5	8 5	0 2 7	0 0	0 5	3 1	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
3	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services	AREA CODE 3 1 6 3 6 4 - 8 8 3 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 6, 1985, at approximately 1225 CDT, an Engineered Safety Features (ESF) actuation occurred resulting in a Safety Injection (SI) and a Main Steamline Isolation due to a low steamline pressure signal on Steam Generator "A". At the time of the event, the plant was in Hot Standby prior to initial criticality. The Reactor Coolant System average temperature was approximately 530 degrees F and pressure was 2241 psig. The Main Steamline Isolation Valves were closed and Steam Generators "A" and "C" were slowly being fed to adjust level. With no Reactor Coolant System flow through the steam generators, the addition of feedwater cooled the steam generators and consequently decreased steamline pressure to the trip setpoint.

An Unusual Event was declared and terminated. The appropriate federal, state, and local agencies were notified. All ESF equipment responded properly, and the Safety Injection was terminated by approximately 1243 CDT. During the subsequent restoration of the ventilation systems, an additional ESF actuation occurred and is discussed in LER 85-029-00.

There was no threat to the health and safety of the public during this event.

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On May 6, 1985, at approximately 1225 CDT, an Engineered Safety Features (ESF) actuation occurred resulting in a Safety Injection (SI) into the Reactor Coolant System (RCS), and a Main Steamline Isolation. The ESF actuation was due to a low steamline pressure signal on Steam Generator (S/G) "A".

At the time of this event, the plant was in Mode 3, Hot Standby, prior to initial criticality, and hot, no-flow control rod drop testing on Control Bank "B" was in progress. The Reactor Coolant Pumps (RCPs) and Main Steamline Isolation Valves had been secured to support this testing, and the average temperature of the RCS was approximately 530 degrees F. The RCS pressure was approximately 2241 psig. The pressurizer level was being controlled manually at approximately 23 percent and pressure was being controlled automatically. Steam Generators "A" and "C" were being slowly fed via the Main Feedwater Bypass Valves to adjust level. With no significant heat transfer to the Steam Generators from the RCS, the addition of feedwater resulted in cooling the Steam Generators and decreasing the steamline pressure to the trip setpoint for low steamline pressure (615 psig), and the Safety Injection and Main Steamline Isolation signals occurred.

An Unusual Event was declared and subsequently terminated. The appropriate federal, state, and local agencies were notified in accordance with the Emergency Plan Implementing Procedures.

All ESF equipment responded properly. The reactor trip breakers opened, and all rods inserted. The RCS pressure transient was limited to approximately 2340 psig by the Pressurizer Power Operated Relief Valves. Pressurizer level increased to 56 percent during this event, and the lowest temperature in the RCS was approximately 500 degrees F.

The Safety Injection was terminated in accordance with plant procedures by approximately 1243 CDT. During the subsequent restoration of the ventilation systems, a Containment Purge Isolation Signal, a Control Room Ventilation Isolation Signal, and a Fuel Building Ventilation Isolation Signal occurred and is discussed in detail in LER 85-029-00.

The cognitive personnel error which led to this event has been discussed with the operating personnel in shift briefings and crew meetings stressing operator awareness of plant status, and particularly early recognition of developing trends. In addition, since the plant conditions allowing no RCS flow through the Steam Generators (minimum or no decay heat present) will only reoccur after subsequent refuelings, the information contained in this report has been added to a Post-Refueling Action File, maintained by Training, which contains items to be stressed to the operators during the Requalification Training presented near the time of refueling.

At no time during this event did conditions develop that may have posed a threat to the health and safety of the public.

KANSAS GAS AND ELECTRIC COMPANY

GLENN L KOESTER
VICE PRESIDENT NUCLEAR

May 31, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-145
Re: Docket No. STN 50-482
Subj: Licensee Event Report 85-027-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a) (2) (iv) concerning an Engineered Safety Feature actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

Glenn L Koester

Glenn L. Koester
Vice President - Nuclear

GLK:dab

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

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