

Docket

JAN 9 1974

DOCKET NO: 50-155

LICENSEE: Consumers Power Company

FACILITY: Big Rock Point

MINUTES OF MEETING WITH CONSUMERS POWER COMPANY OF MICHIGAN REGARDING
THE ADDITION OF AUTO-DEPRESSURIZATION CAPABILITY TO THE BIG ROCK POINT PLANT

Representatives of Consumers Power Company and the AEC (see attachment) met in Bethesda on December 5, 1973, to discuss seismic design, pipe and vessel code, instrumentation, redundancy, and reliability requirements for the reactor auto-depressurization system proposed for Big Rock Point. Reference was made to previous Consumer Power Company submittals:

1. Special Report No. 15 - "Preliminary Description of Reactor Depressurization System", July 19, 1973, submitted to AEC by CPGC letter dated July 24, 1973.
2. "Big Rock Point Loss of Coolant Analysis with Automatic Depressurization and NPS Demonstration Fuel" submitted to AEC by letter dated September 22, 1972.

It was emphasized by CPGC that the system is designed so that no single failure will actuate or prevent actuation of the system and that a two-minute delay after receipt of the auto-depressurization actuation signal is necessary to allow personnel evacuation of the containment structure before the reactor vessel is depressurized to containment.

A quench tank containing cold water about equal to the primary coolant water volume is provided to quench steam released unintentionally from one relief valve. During auto-depressurization, quenching would occur for about 40 seconds after which the primary system steam would no longer be condensed and would pass through the heated quench tank water to the containment structure. Accidental opening of one relief valve, with failure to isolate, could activate the entire blowdown system (3 or 4 relief valves of approximately 800,000 lbs/hr flow capacity each at 1350 psi) when low level steam drum and reactor vessel signals are received coincidentally since auto-depressurization is activated due to loss of water by level signals from the steam drum and reactor vessel. The need for further evaluation of manual operation and isolation capability was indicated.

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Seismic design criteria for the proposed modifications are based on review of earthquakes that have been recorded for the area and statistical considerations. The new T connection for the relief valve header to be installed in the 12-inch steam line will not be hydrostatically tested but will be 100% radiographed. The quench tank, although not required during auto-depressurization, is designed not to fail during accident conditions to avoid the damage potential resulting from the release of the tank water. The significant change discussed during the meeting involved the decision to use a small quench tank, as discussed above, instead of the spent fuel pool water with the result that containment pressurization is greater than originally planned using spent fuel water-steam suppression but less than DBA LOCA pressurization that has been previously evaluated.

Component delivery schedules, according to CPGO, may not be consistent with completion of the modification to conform with the Interim Acceptance ECCS Criteria by July 1974.

A final design report is expected during January 1974.

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James J. Shea
Operating Reactors Branch #2
Directorate of Licensing

Enclosure.
List of Attendees

cc w/enclosure:
M. Sewell, CPGO
AEC PDA
L Reading
RP Reading
E. G. Case
A. Giambusso
R. S. Boyd
RP/TR Assistant Directors
RP/TR Branch Chiefs
T. J. Carter
J. M. Hendrie
J. J. Shea
J. Scinto
D. Nelson
ACRS (16)
K. Kapur

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R. Pollard	D. Bernreuter	172/74	X7403 JJShea:sh	1/2/74	1/8/74

LIST OF ATTENDEES

<u>Name</u>	<u>Affiliation</u>
D. DeMoor	Consumers Power Company
F. Macri	Consumers Power Company
J. Rang	Consumers Power Company
R. Sewell	Consumers Power Company
J. Henry	McPherson Associates Inc.
D. Wilder	Suntac Nuclear
F. Mueller	Suntac Nuclear
J. Shea	AEC - Licensing
K. Kapur	AEC - Licensing
R. Pollard	AEC - Licensing
D. Bernreuter	AEC - Licensing

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