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MAINE YANKEE ATOMIC POWER COMPANY

Box 450, RFD 2
Wiscasset, Maine 04578

January 30, 1980

United States Nuclear Regulatory Commission
Directorate of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, PA 19406

Reference: License No. DPR-36 (Docket No. 50-309)

Dear Sir:

Subject: Maine Yankee Reportable Occurrence #80-001/01L-0

Pursuant to the requirements of Technical Specification 5.9.1.6.h, the Maine Yankee Atomic Power Company hereby submits the following report.

In the course of reviewing the steam line break analysis in response to the concerns raised by the NRC with regard to automation of the auxiliary feedwater system, it was discovered that an incorrect assumption was utilized in the zero or low-power steam line break analysis. The analysis had assumed that at low power levels, when feedwater is being controlled by the feedwater regulating valve bypass valves, that these bypass valves would remain "as-is" throughout the transient. In reality, as a result of the low steam generator pressure reactor trip signal, an override signal causes the feedwater bypass valves to ramp to an 80% open (5% full feedwater flow) position. A reanalysis of this event shows that this opening of the feedwater bypass valves results in an unacceptably high rate of feedwater addition to the affected steam generator causing a rapid cooldown of the reactor coolant system and a return-to-power transient. A return to power following a steam line break is not permitted under the current design basis for Maine Yankee.

The plant was in the refueling mode at the time this error was discovered and therefore no immediate corrective action was necessary. Modifications necessary to correct this error shall be implemented prior to plant startup.

Very truly yours,

MAINE YANKEE ATOMIC POWER COMPANY

Edwin C. Wood
Edwin C. Wood
Plant Manager

ECW/kmf

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