



Commonwealth Edison
LaSalle County Nuclear Station
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Marseilles, Illinois 61341
Telephone 815/357-6761

December 10, 1992

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for November 1992.

Very truly yours,

for G. J. Diederich
Station Manager
LaSalle County Station

GJD'MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

NOVEMBER 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor sub-critical, Generator off-line, refuel outage (L1R05) in progress.
30	2400	Reactor sub-critical, Generator off-line, refuel outage (L1R05) in progress.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-012-00	11/02/92	Missed fire watch for inoperable fire detection zone 1-15.
92-013-00	11/14/92	Automatic start of the '1A' Diesel Generator due to instrument spike during performance of a Reactor high pressure response time test.
92-014-00	11/11/92	Violation of a high radiation area boundry.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L04716	Residual Heat Removal Manual Discharge Valve 1E12-F098C	Leakage past bonnet gasket.	None.	Replaced bonnet gasket.
L17916	Containment Monitoring System Drywell Air Temperature Indicator 1TI-CM058	Indication sticking at 200°F.	Inaccurate temperature indication.	Replaced indicator.
L18375	Residual Heat Removal Shutdown Down Cooling Manual Stop Valve 1E12-F090B	Dual indication when valve was closed.	Inaccurate indication.	Adjusted limit switches.
L18552	Diesel Fuel Oil Transfer Pump 0D001P	Pump oiler leaking.	None.	Replaced oiler.
L18559	Reactor Vessel Low Water Level 3 'A' Logic	Level Transmitter 1B21-N403A.	Greater than 1% shift during Transmitter Calibration.	Replaced transmitter.
L18706	Main Stream Line High Flow Switch 1E31-N011A	Excessive electrical noise in switch.	None.	Replaced switch.
L18775	Reactor Building Exhaust Isolation Damper 1VR05YB	Damper binding when cycled.	Redundant damper available.	Added additional supports to damper structure.
L18776	Reactor Building Ventilation Exhaust Isolation Damper 1VR05YB	Air hoses leaking.	None.	Replaced air hoses.

(No SOR Failures this month.)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L18777	Reactor Building Ventilation Supply Isolation Damper 1VR04YA	Air hose leaking.	None.	Replaced air hose.
L18869	Diesel Generator Scavenging Air Pressure Indicator OPI-DG086	Broken sensing line.	Improper pressure indication.	Replaced union, reconnected tubing.
L18905	High Pressure Core Spray Pump 1E22-C001	Pump breaker failed to insert fully into cubicle.	None.	Replaced breaker.
L18909	High Pressure Core Spray Diesel Generator Crankcase Pressure Indicator 1E22-R546	Pressure indicator snubber.	Improper indication.	Replaced snubber.
L19033	High Pressure Core Spray Diesel Generator Overspeed Trip	Overspeed trip relay failed to operate.	None.	Replaced relay.
L19039	Control Rod Drive Hydraulic Control Unit 38-31	Accumulator leaking.	None.	Replaced accumulator O-ring.
L19043	Control Rod Drive Hydraulic Control Unit 42-11	Nitrogen leaking from drain cap.	None.	Replaced drain cap.

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L19071	Control Rod Drive Hydraulic Control Unit 18-51	Nitrogen charging valve leaking.	None.	Replaced valve cartridge.
L19097	Control Rod Drive Hydraulic Control Unit 18-35	Nitrogen charging valve leaking.	None.	Replaced valve cartridge.
L19098	Control Rod Drive Hydraulic Control Unit 26-07	Nitrogen charging valve leaking.	None.	Replaced valve cartridge.

(No SOR Failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE December 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD:	November 1992		
GROSS HOURS IN REPORTING PERIOD:	720		
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323		
MAX DEPENDABLE CAPACITY (MWe-Net):	1,036		
DESIGN ELECTRICAL RATING (MWe-Net):	1,078		
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):	N/A		
4. REASON FOR RESTRICTION (IF ANY):			
	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	0.0	6,568.3	53,924.8
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	0.0	6,529.3	52,907.6
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWh _t)	0	19,903,179	155,060,158
10. ELECTRICAL ENERGY GENERATED (MWh _e -Gross)	0	6,702,577	51,792,349
11. ELECTRICAL ENERGY GENERATED (MWh _e -Net)	-8,026	6,461,252	49,656,397
12. REACTOR SERVICE FACTOR (%)	0.0	81.7	69.0
13. REACTOR AVAILABILITY FACTOR (%)	0.0	81.7	71.1
14. UNIT SERVICE FACTOR (%)	0.0	81.2	67.7
15. UNIT AVAILABILITY FACTOR (%)	0.0	81.2	67.7
16. UNIT CAPACITY FACTOR (USING MDC) (%)	-1.1	77.6	61.3
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	-1.0	74.5	58.9
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	1.4	6.9
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:	01/08/93		

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE December 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: November 1992

DAY	POWER	DAY	POWER
1	-11	17	-11
2	-11	18	-13
3	-11	19	-12
4	-11	20	-11
5	-11	21	-11
6	-11	22	-11
7	-11	23	-11
8	-11	24	-11
9	-11	25	-11
10	-11	26	-11
11	-11	27	-11
12	-11	28	-11
13	-11	29	-11
14	-12	30	-11
15	-11	31	
16	-11		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
4	921003	S	720.0	C	1	Refuel outage (L1R05)

SUMMARY OF OPERATION:

The unit entered a scheduled refueling outage on 10/03/92. Scheduled return to service is 01/08/93.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations
(None.)
2. ECCS System Outages
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual
(None.)
4. Major changes to Radioactive Waste Treatment Systems.
(None.)
5. Indications of Failed Fuel Elements.
(None.)

(Unit 1)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
(U-0)		
0597 0632 0844	ODG01K	Replace air start, tubing, general maintenance.
0831 0845	ODG01K	Installation of quick disconnects on the lube oil circulation pump.
0833	ODG01K	Inspection.
0848	ODG01K	Cooling water strainer inspection cable repair.
0859	ODG01K	Heat exchanger maintenance.
(U-1)		
0676	1DG01K	General diesel generator maintenance.
0709	1DG01K	Relay inspection.
0751	1DG01K	Cooling water strainer inspection.
1650	1DG01K	Inspection.
1655	1DG01K	Modify mounting of the lube oil circulation pump motor.
1658	1DG049A	Troubleshoot and repair check valve.
1659	1DG01K	Administrative control.
1678	1DG035	Troubleshoot and repair motor operated valve.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

NOVEMBER 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1130 Mwe.
	0400	Reduced power level to 990 Mwe due to system load.
	1300	Increased power level to 1135 Mwe.
2	0030	Reduced power level to 940 Mwe due to system load.
	1300	Increased power level to 1135 Mwe.
4	0030	Reduced power level to 980 Mwe due to system load.
	1200	Increased power level to 1135 Mwe.
8	0400	Reduced power level to 945 Mwe due to system load.
	1100	Increased power level to 1135 Mwe.
9	0100	Reduced power level to 995 Mwe due to system load.
	1000	Increased power level to 1135 Mwe.
10	2330	Reduced power level to 890 Mwe due to system load.
11	1100	Increased power level to 1135 Mwe.
12	0230	Reduced power level to 850 Mwe due to system load.
	1100	Increased power level to 1130 Mwe.
13	0300	Reduced power level to 900 Mwe due to system load.
	1100	Increased power level to 1130 Mwe.
	2330	Reduced power level to 850 Mwe to place the Turbine Driven Reactor Feed Pump out of service for maintenance.
14	1000	Increased power level to 1030 Mwe.
16	0123	Reactor scram due to loss of the station service/ instrument air system.
18	0450	Reactor critical.
	2023	Generator on-line at 80 Mwe.
19	0700	Increased power level to 200 Mwe.
	1700	Increased power level to 750 Mwe.
	2100	Increased power level to 1050 Mwe.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2) (CONTINUED)

<u>Day</u>	<u>Time</u>	<u>Event</u>
21	0030	Reduced power level to 815 Mwe for rod set.
	0300	Reduced power level to 630 Mwe for rod set.
	1600	Increased power level to 1130 Mwe.
23	0100	Reduced power level to 995 Mwe due to system load.
	0700	Increased power level to 1130 Mwe.
25	1300	Reduced power level to 1100 Mwe due to system load.
	1800	Increased power level to 1130 Mwe.
26	0230	Reduced power level to 850 Mwe due to system load.
	1400	Increased power level to 1125 Mwe.
27	0100	Reduced power level to 850 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
28	0100	Reduced power level to 850 Mwe due to system load.
	1100	Increased power level to 1135 Mwe.
29	0300	Reduced power level to 895 Mwe due to system load.
	1700	Increased power level to 1135 Mwe.
	2300	Reduced power level to 980 Mwe due to system load.
30	0300	Reduced power level to 910 Mwe due to system load.
	1000	Increased power level to 1135 Mwe.
	2400	Reactor critical, Generator on-line at 1140 Mwe.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
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92-016-00	11/16/92	Reactor scram due to loss of Service Air.
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E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.
(See Table 2)
2. Average Daily Unit Power Level.
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.
(See Table 4)

C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L17682	Control Rod Drive Hydraulic Control Unit 18-43	Directional control valve 122 sticking open.	Double notching of control rod.	Replaced directional control valve
L18120	Local Power Range Monitor 48-09B	LPRM card.	LPRM failed 1/2 scram received.	Cleaned card connector.
L18793	Average Power Range Monitor A	Power supply voltage oscillating.	None.	Replaced power supply.

(See attached SOR Failure Report.)

SOR dp SWITCH FAILURE DATA SHEET

Equipment Piece Number: 2E31-N012BB

Model Number: 103-AS-B203-NX-JJTX6

Serial Number: 85-1-2436

Application: Residual Heat Removal Shutdown Cooling line break switch (alarm)

Date and Time of Discovery: 11/10/92 1745 hours

Reactor Mode: 1 (Run) Power Level: 100%

Calibration Tolerance: 167.4 - 169.4 "WC

Nominal Setpoint: 168.4 "WC

Action Limits: < 163.0 or >173.8 "WC

Reject Limits: < 158.8 or >178.0 "WC

Technical Specification

Limits: N/A "WC

As Found Setpoint: N/A "WC

Date and Time of Return to Service: N/A _____ hours

Model Number of Replacement Switch: N/A _____

Serial Number of Replacement Switch: N/A _____

DVR Number: 1-2-92-080

Cause: Switch was found out of calibration during monthly functional test.
Switch failure was attributed to a torn diaphragm.

Corrective Action: The switch was taken out of service pending removal
during refuel outage L2R05 per modification M01-2-88-060.

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE December 10, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD:	November 1992		
GROSS HOURS IN REPORTING PERIOD:	720		
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323		
MAX DEPENDABLE CAPACITY (MWe-Net):	1,036		
DESIGN ELECTRICAL RATING (MWe-Net):	1,078		
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):	N/A		
4. REASON FOR RESTRICTION (IF ANY):			
	THIS MONTH	YEAR TO DATE	CUMULATIVE
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5. REACTOR CRITICAL TIME (HOURS)	692.6	5,333.7	50,269.5
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	653.0	5,107.8	49,312.4
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh _t)	2,034,654	15,378,284	147,620,683
10. ELECTRICAL ENERGY GENERATED (MWh _e -Gross)	689,387	5,194,685	49,084,435
11. ELECTRICAL ENERGY GENERATED (MWh _e -Net)	670,277	4,987,675	47,119,179
12. REACTOR SERVICE FACTOR (%)	96.2	66.3	70.6
13. REACTOR AVAILABILITY FACTOR (%)	96.2	66.3	73.0
14. UNIT SERVICE FACTOR (%)	90.7	63.5	69.3
15. UNIT AVAILABILITY FACTOR (%)	90.7	63.5	69.3
16. UNIT CAPACITY FACTOR (USING MDC) (%)	89.9	59.9	63.9
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	86.4	57.5	61.4
18. UNIT FORCED OUTAGE FACTOR (%)	9.3	9.5	12.8
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE December 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: November 1992

DAY	POWER	DAY	POWER
1	1,063	17	-11
2	1,056	18	6
3	1,099	19	499
4	1,062	20	987
5	1,100	21	990
6	1,101	22	1,095
7	1,103	23	1,078
8	1,083	24	1,097
9	1,073	25	1,088
10	1,089	26	997
11	1,038	27	1,033
12	1,037	28	1,034
13	1,048	29	1,002
14	967	30	1,054
15	1,008	31	
16	53		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
9	921116	F	67.0	G	3	Reactor scram due to a transient caused by loss of station air.

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system load and maintenance activities. The unit experienced a forced outage on 11/16/92, the unit was returned to service on 11/18/92.

F. . UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

<u>Date</u>	<u>Valves Actuated</u>	<u>No & Type Actuations</u>	<u>Plant Condition</u>	<u>Description of Event</u>
11/16/92	2B21-F013U	Automatic	1	Scram (LER#92-016-00)

2. ECCS System Outages
(See Table 5.)

3. Changes to the Off-Site Dose Calculation Manual.
(None.)

4. Major changes to Radioactive Waste Treatment Systems.
(None.)

5. Indications of Failed Fuel Elements.
(None.)

(Unit 2)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
2218	2E51-D003	Repair steam trap.
2220	2E22-C003	Coupling lubrication.
2228	2E22-S001	Install temporary pressure gauge.
2229	2E51-C004 2E51-C005	Replace brushes.