



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

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October 18, 1979

Honorable Joseph M. Hendrie
Chairman
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: TWO-HUNDRED-THIRTY-FOURTH MEETING OF THE ADVISORY COMMITTEE ON
REACTOR SAFEGUARDS, OCTOBER 4-6, 1979 IN WASHINGTON, DC

Dear Dr. Hendrie:

The Advisory Committee on Reactor Safeguards held its 234th meeting in Washington, DC on October 4-6, 1979. Items considered at this meeting are summarized briefly below:

The Committee continued its evaluation of the March 28, 1979 accident at Three Mile Island Nuclear Station Unit 2 (TMI-2) and the implications of this accident. The Committee members also exchanged preliminary views regarding the effectiveness of the nuclear regulatory process.

The Committee responded to an inquiry from Rep. Morris K. Udall regarding the development of a hybrid standard nuclear plant design. A copy of the resulting letter has been provided to you.

The Committee responded to an inquiry from Commissioner Gilinsky regarding control of xenon emissions which may occur during nuclear plant accidents. A copy of the resulting letter has been provided to you.

The Committee discussed a request from Commissioner Bradford to identify NRC regulations that the Committee believes should be changed, and to comment on the adequacy of the process for implementation of Committee recommendations in this area. This matter has been scheduled for further consideration during the 235th ACRS Meeting.

The Committee provided comments to you regarding the scope and pace of the NRC's Systematic Evaluation Program.

The Committee provided the Executive Director for Operations with its comments regarding the proposed analysis of systems interactions at the Indian Point Nuclear Generating Station Unit 3. A copy of this memorandum has been provided to you.

The Committee discussed procedures for developing its annual report to Congress on the NRC's Reactor Safety Research Program, and the setting of priorities for the projects in this program. This matter will require additional work and has been scheduled for further discussion.

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The Committee was briefed by members of the NRC Staff regarding the sequence of events during the September 25, 1979 transient at the North Anna Power Station Unit 1.

The Committee discussed with members of the NRC Staff the current actions and future plans as a follow-on to the analyses of the TMI-2 accident.

Since the last report of ACRS activities, the following subcommittee meetings have been held:

Reliability and Probabilistic Assessment, in Los Angeles, CA on September 11-12, 1979

Fluid Dynamics, in Los Angeles, CA on September 13-14, 1979

Waste Management, in Washington, DC on September 18-19, 1979

TMI-2 Accident Bulletins and Orders, in Washington, DC on October 2, 1979

TMI-2 Implications Re. Nuclear Power Plant Design, in Washington, DC on October 3, 1979.

The following items are tentatively scheduled for consideration during the 235th ACRS Meeting, November 8-10, 1979:

Review of Regulatory Processes and Procedures -- effectiveness of program

Westinghouse NSSS Using Ice Condenser Containments -- TMI-2 accident implications

Diablo Canyon Nuclear Power Plant Unit 2 -- TMI-2 accident implications

Resolution of ACRS Generic Items -- evaluation of status

NRC Staff Task Action Plans -- status of resolution

Westinghouse NSSS/Salem Nuclear Generating Station Unit 1 -- Steam Line Break Implications

Regulatory Guide 1.97, Instrumentation for LWRs to Assess Accident conditions -- proposed revision

NUREG 0600, Investigation of TMI Accident -- TMI-2 accident implications

ATWS -- status of resolution

Honorable Joseph M. Hendrie

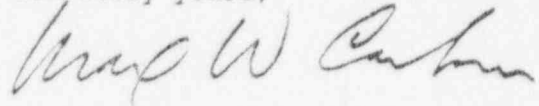
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NRC Rules and Regulations -- proposed changes

Wolf Creek Generating Station -- Seismic Design Basis

Sincerely yours,

A handwritten signature in cursive script, reading "Max W. Carbon". The signature is written in dark ink and is positioned to the right of the typed name.

Max W. Carbon
Chairman