

WESTINGHOUSE CLASS 3 (Non-Proprietary)



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A PDR

WCAP-13559

OPERATIONAL ASSESSMENT FOR
AP600

AP600 DOC.# GW GLR 101

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Westinghouse Electric Corporation
Energy Systems Business Unit
Nuclear And Advanced Technology Division
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| GW GLR 001 | 0 | Dec. 15, 1992 | ASSIGNED TO: |
| ALTERNATE DOCUMENT NUMBER: WCAP-13559 DESIGN AGENT ORGANIZATION: PROJECT: TITLE: OPERATIONAL ASSESSMENT FOR AP600 WORK BREAKDOWN #: This section incorporates the following design changes DCP #/Rev.: | | | ATTACHMENTS |

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| AP600 RESPONSIBLE MANAGER N. J. Liparulo | <i>N. J. Liparulo</i> 12/14/92 |

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IMAGE EVALUATION
TEST TARGET (MT-3)

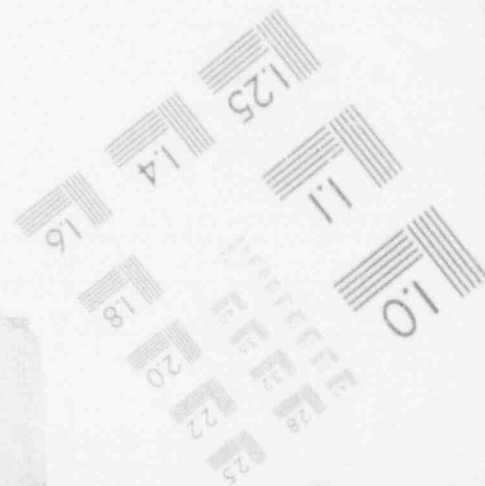
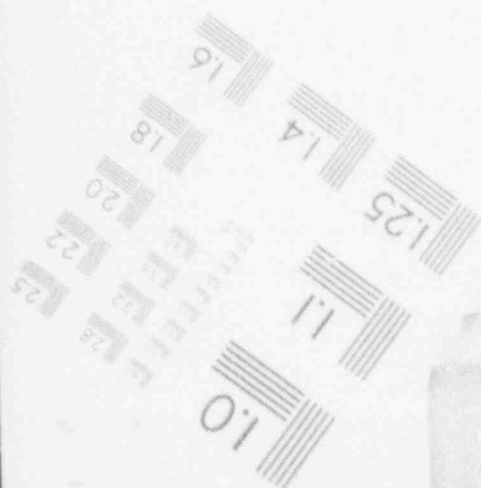
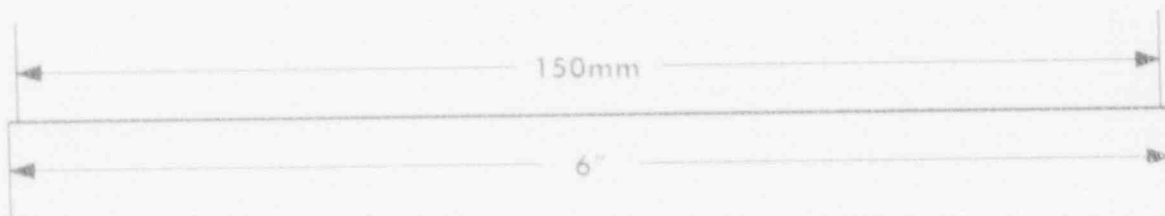
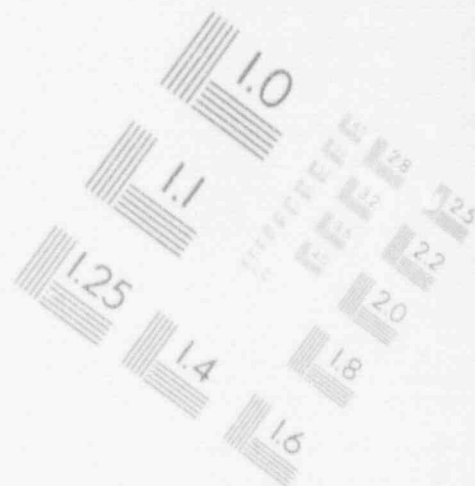
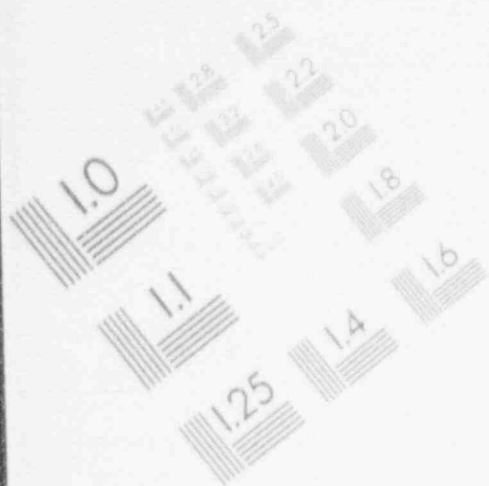
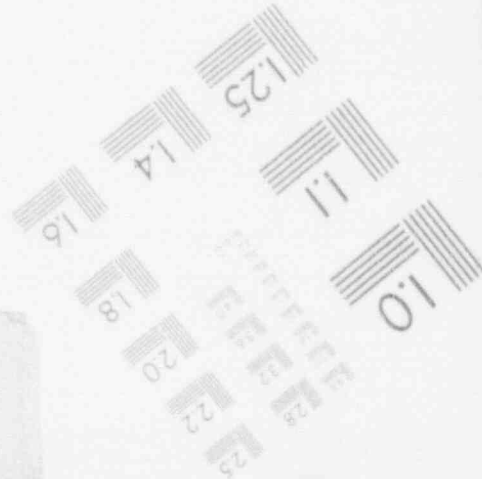
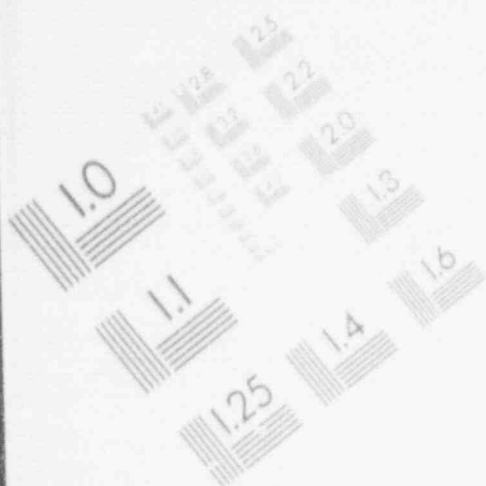
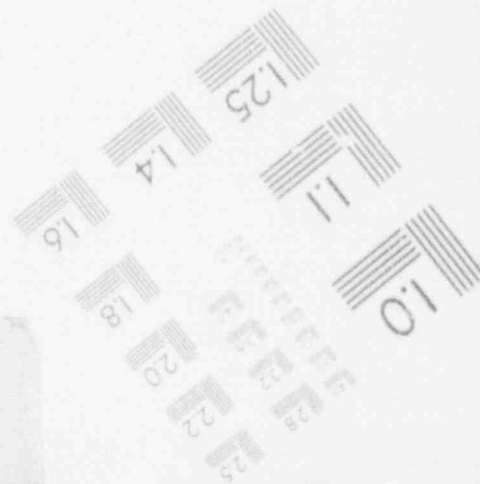
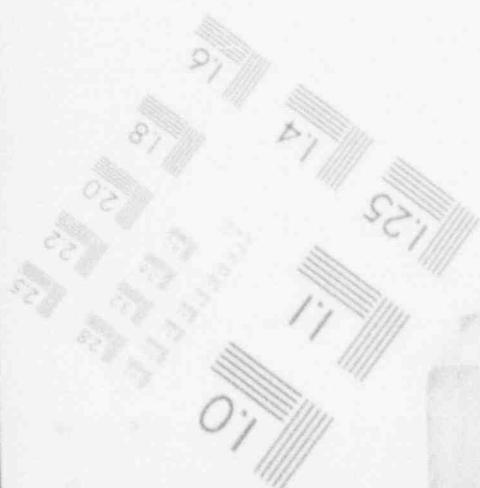
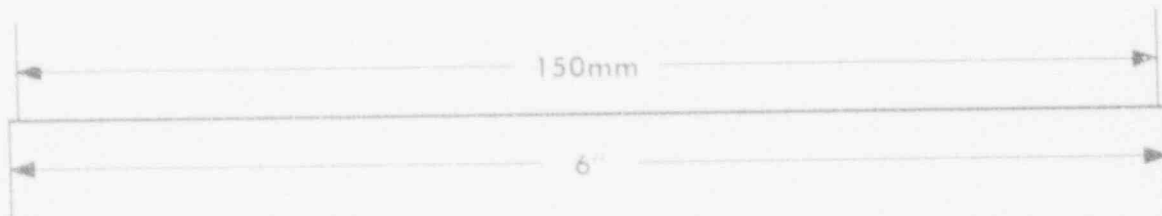
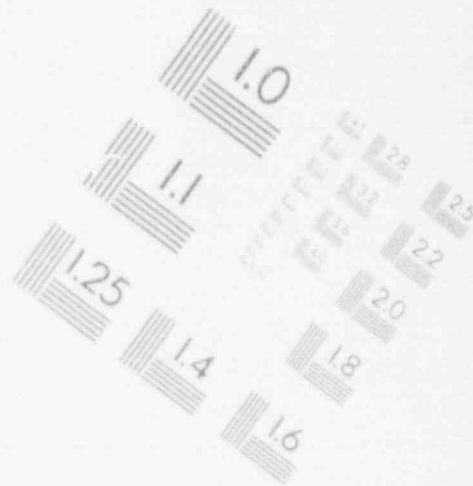
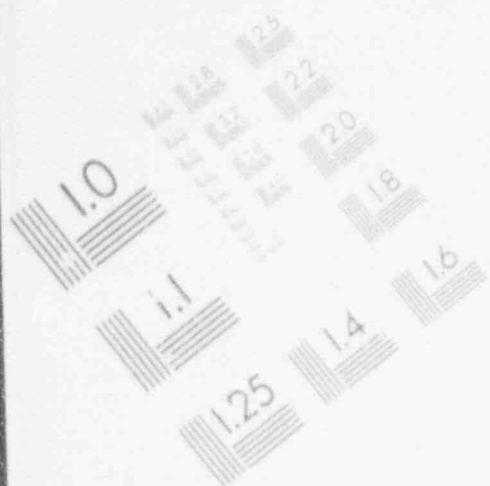


IMAGE EVALUATION
TEST TARGET (MT-3)



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IMAGE EVALUATION
TEST TARGET (MT-3)



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DELIVERED DATA

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WCAP-13559

WESTINGHOUSE CLASS 3

OPERATIONAL ASSESSMENT

FOR

AP600

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December 1992

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TABLE OF CONTENTS

| | |
|---------------------|-----|
| TABLE OF CONTENTS | i |
| INTRODUCTION | ii |
| I. E. BULLETINS | 1 |
| GENERIC LETTERS | 9 |
| I.E. CIRCULARS | 42 |
| INFORMATION NOTICES | 46 |
| AEOD REPORTS | 125 |

INTRODUCTION

The NRC Commissioners have requested in their SRMs dated July 31, 1989, February 15, 1991 and March 5, 1991, that new plant designs provide a discussion of how operational experience has been incorporated into the design process. This report addresses the manner in which operational experience has been incorporated into the design of the AP600.

PROCESS

Generic Letters, IE Bulletins, Circulars, Information Notices and AEOD Reports for the time period January 1, 1980 to December 31, 1991 were reviewed to determine applicability to the AP600 design certification.

The dispositions of individual documents can be broken down into the following categories:

- Not Applicable to the AP600 design
(e.g. BWR only, B & W facilities only, not applicable to commercial reactors)
- Not Applicable to the design certification due to other reasons. For example:
 - Procurement issue - These issues were determined not to be applicable to the design certification phase.
 - Administrative communications - These items were documents that provide administrative information (e.g. scheduling of meetings, forwarding other documents, etc)
 - Procedural issue - These items were concerned with either lack of proper procedures or failure to comply with procedures.
 - Plant specific/isolated event - These issues were concluded to be isolated events at specific plants with no generic conclusions or impacts.
 - Maintenance/Surveillance issues - These issues were concluded to be related to maintenance or surveillance of a system or component.
- Applicable to the AP600 design certification in which case the proper SSAR Section reference is given

I.E. BULLETINS

I.E. BULLETIN

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 80-01 | Operability of ADS Valve Pneumatic Supply (1/80) | Not Applicable BWR only |
| 80-02 | Inadequate Quality Assurance for Nuclear Supplied Equipment (1/80) | Not Applicable BWR only |
| 80-03 | Loss of Charcoal From Standard Type II, 2 Inch, Tray Adsorber Cells (2/80) | Not Applicable Procurement issue |
| 80-04 | Analysis of a PWR Main Steam Line Break With Continued Feedwater Addition (3/80) | SSAR Section 15.1.5 |
| 80-05 | Vacuum Condition Resulting in Damage to Chemical Volume Control System (CVCS) Holdup Tanks (3/80) | Not Applicable AP600 design has no hold-up tanks in the CVCS |
| 80-06 | Engineered Safety Feature (ESF) Reset Controls (3/80) | SSAR Section 7.3.1.1.1 |
| 80-07 | BWR Jet Pump Assembly Failure (4/80) | Not Applicable BWR only |
| 80-07 S1 | BWR Jet Pump Assembly Failure (5/80) | Not Applicable BWR only |
| 80-08 | Examination of Containment Liner Penetration Welds (4/80) | Not Applicable AP600 design has no containment liner |
| 80-09 | Hydrometer Actuator Deficiencies (4/80) | Not Applicable Procurement issue |
| 80-10 | Contamination of Nonradioactive System and Resulting Potential for Unmonitored, Uncontrolled Release of Radioactivity to Environment (5/80) | Not Applicable Part of COL Surveillance issue |
| 80-11 | Masonry Wall Design (5/80) | Not Applicable AP600 design has no safety-related masonry walls |
| 80-12 | Decay Heat Removal Operability (5/80) | SSAR Section 7.4.1 |

I.E. BULLETIN

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 80-13 | Cracking in Core Spray Spargers (5/80) | Not Applicable BWR only |
| 80-14 | Degradation of BWR Scram Discharge Volume Capability (6/80) | Not Applicable BWR only |
| 80-15 | Possible Loss of Emergency Notification System (ENS) with Loss of Offsite Power (6/80) | Not Applicable Part of COL |
| 80-16 | Potential Misapplication of Rosemount Inc. Models 1151 and 1152 Pressure Transmitters with Either "A" or "D" Output Codes (6/80) | Not Applicable Procurement/Maintenance issue |
| 80-17 | Failure of 76 to 185 Control Rods to Fully Insert During a Scram at a BWR (7/80) | Not Applicable BWR only |
| 80-17 S1 | Failure of 76 to 185 Control Rods to Fully Insert During a Scram at a BWR (7/80) | Not Applicable BWR only |
| 80-17 S2 | Failure of 76 to 185 Control Rods to Fully Insert During a Scram at a BWR (7/80) | Not Applicable BWR only |
| 80-17 S3 | Failure of 76 to 185 Control Rods to Fully Insert During a Scram at a BWR (8/80) | Not Applicable BWR only |
| 80-17 S4 | Failure of 76 to 185 Control Rods to Fully Insert During a Scram at a BWR (12/80) | Not Applicable BWR only |
| 80-17 S5 | Failure of 76 to 185 Control Rods to Fully Insert During a Scram at a BWR (2/81) | Not Applicable BWR only |
| 80-18 | Maintenance of Adequate Minimum Flow Through Centrifugal Charging Pump Following Secondary Side High Energy Line Rupture (7/80) | Not Applicable AP600 design has no charging pumps as part of SI system |
| 80-19 | Failures of Mercury-Wetted Relays in Reactor Protective Systems of Operating Nuclear Power Plants Designed by Combustion Engineering (7/80) | Not Applicable Procurement issue |
| 80-19 S1 | Failures of Mercury-Wetted Relays in Reactor Protective Systems of Operating Nuclear Power Plants Designed by Combustion Engineering (7/80) | Not Applicable Procurement issue |

I.E. BULLETIN

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 80-20 | Failures of Westinghouse Type W-2 Spring Return to Neutral Control Switches (7/80) | Not Applicable Procurement issue |
| 80-21 | Valve Yokes Supplied by Malcolm Foundry Company, Inc. (11/80) | Not Applicable Procurement issue |
| 80-22 | Automation Industries, Model 200-520-008 Sealed-Source Connectors (9/80) | Not applicable to commercial nuclear power plants |
| 80-23 | Failures of Solenoid Valve Manufactured by Valcor Engineering Corporation (11/80) | Not Applicable Procurement issue |
| 80-24 | Prevention of Damage Due to Water Leakage Inside Containment (11/80) | SSAR Section 3.4.1.2.2.1 |
| 80-25 | Operating Problems with Target Rock Safety - Relief Valves at BWRs (12/80) | Not Applicable BWR only |
| 81-01 | Failure of Mechanical Snubbers (1/82) | Not Applicable Procurement issue |
| 81-02 | Failure of Gate Type Valves to Close Against Differential Pressure (4/81) | Not Applicable Procurement issue |
| 81-03 | Flow Blockage of Cooling Water to Safety System Components by Corbicula SP. and Mytilus SP. (4/81) | Not Applicable Site specific issue |
| 82-01 | Alteration of Radiographs of Welds in Piping Subassemblies (3/82) | Not Applicable Administrative communication |
| 82-01 R1 | Alteration of Radiographs of Welds in Piping Subassemblies (5/82) | Not Applicable Administrative communication |
| 82-01 S1 | Alteration of Radiographs of Welds in Piping Subassemblies (8/82) | Not Applicable Administrative communication |
| 82-02 | Degradation of Threaded Fasteners in Reactor Coolant Pressure Boundary of PWR Plants (6/82) | Not Applicable Maintenance issue |
| 82-03 | Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants (10/82) | Not Applicable BWR only |

I.E. BULLETIN

| NUMBER | TITLE | COMMENT |
|----------|---|-------------------------------------|
| 82-03 R1 | Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants (10/82) | Not Applicable BWR only |
| 82-04 | Deficiencies in Primary Containment Electrical Penetration Assemblies (12/80) | Not Applicable Procurement issue |
| 83-01 | Failure of Reactor Trip Breakers (Westinghouse DB-50) to Open on Automatic Trip Signal (2/83) | Not Applicable Maintenance issue |
| 83-02 | Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants (3/83) | Not Applicable BWR only |
| 83-03 | Check Valve Failures In Raw Water Cooling Systems of Diesel Generators (3/83) | Not Applicable Procurement issue |
| 83-04 | Failure of the Under-Voltage Trip Function of Reactor Trip Breakers (3/83) | Not Applicable BWR only |
| 83-05 | ASME Nuclear Code Pumps and Spare Parts Manufactured by the Hayward Tyler Pump Company (5/83) | Not Applicable Procurement issue |
| 83-06 | Nonconforming materials Supplied by Tube-Line Corporation Facilities at Long Island City, New York; Houston, Texas; and Carol Stream, Illinois (7/83) | Not Applicable Procurement issue |
| 83-07 | Apparently Fraudulent Products Sold by Ray Miller, Inc. (7/83) | Not Applicable Procurement issue |
| 83-07 S1 | Apparently Fraudulent Products Sold by Ray Miller, Inc. (10/83) | Not Applicable Procurement issue |
| 83-07 S2 | Apparently Fraudulent Products Sold by Ray Miller, Inc. (12/83) | Not Applicable Procurement issue |
| 83-08 | Electrical Circuit Breakers with an Undervoltage Trip Feature in Use in Safety-Related Applications Other Than the Reactor Trip System (12/83) | Not Applicable Maintenance issue |

I.E. BULLETIN

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 84-01 | Cracks in Boiling Water Reactor Mark 1 Containment Vent Headers (2/84) | Not Applicable BWR only |
| 84-02 | Failures of General Electric Type HFA Relays in Use in Class 1E Safety Systems (3/84) | Not Applicable Procurement issue |
| 84-03 | Refueling Cavity Water Seal (8/84) | Not Applicable AP600 design does not have this type of seal |
| 85-01 | Steam Binding of Auxiliary Feedwater Pumps (10/85) | AP600 has no safety-related AFW or similar safety-related system |
| 85-02 | Undervoltage Trip Attachments of Westinghouse DB-50 Type Reactor Trip Breakers (11/85) | SSAR Section 7.1.2.2.4, Chapter 16, Section 3.3.1.6 Surveillance issue |
| 85-03 | Motor-Operated Valve Common Mode Failures During Plant Transients Due to Improper Switch Settings (11/85) | See Generic Letter 89-10 |
| 85-03 S1 | Motor-Operated Valve Common Mode Failures During Plant Transients Due to Improper Switch Settings (4/88) | See Generic Letter 89-10 |
| 86-01 | Minimum Flow Logic Problems that Could Disable RHR Pumps (5/86) | Not Applicable BWR only |
| 86-02 | Static "O" Ring Differential Pressure Switches (7/86) | Not Applicable Procurement issue |
| 86-03 | Potential Failure Of Multiple ECCS Pumps Due to Single Failure of Air-Operated Valve in Minimum Flow Recirculation Line (10/86) | Not Applicable AP600 design has no valves in miniflow lines |
| 86-04 | Defective Teletherapy Timer That May Not Terminate Treatment Dose (10/86) | Not applicable to commercial power plants |
| 87-01 | Thinning of Pipe Walls in Nuclear Power Plants (7/87) | Surveillance issue Part of COL SSAR Sections 5.4.3.4, 10.3.6 |
| 87-02 | Fastener Testing to Determine Conformance with Applicable Material Specifications (11/87) | Not Applicable Procurement issue |

I.E. BULLETIN

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 88-01 | Defects in Westinghouse Circuit Breakers (2/88) | Not Applicable Procurement issue |
| 88-02 | Rapidly Propagating Fatigue Cracks In Steam Generator Tubes (2/88) | Not Applicable AP600 design has stainless steel support plates |
| 88-03 | Inadequate Latch Engagement in HFA Type Latching Relays Manufactured By General Electric Company (3/88) | Not Applicable Procurement issue |
| 88-04 | Potential Safety-Related Pump Loss (5/88) | Not Applicable AP600 design has no safety-related pumps |
| 88-05 | Nonconforming Materials Supplied By Piping Supplies, Inc., at Folsom, NJ and West Jersey Manufacturing Company at Williamstown, New Jersey (5/88) | Not Applicable Procurement issue |
| 88-05 S1 | Nonconforming Materials Supplied By Piping Supplies, Inc., at Folsom, NJ and West Jersey Manufacturing Company at Williamstown, New Jersey (6/88) | Not Applicable Procurement issue |
| 88-05 S2 | Nonconforming Materials Supplied By Piping Supplies, Inc., at Folsom, NJ and West Jersey Manufacturing Company at Williamstown, New Jersey (8/88) | Not Applicable Procurement issue |
| 88-06 | Action To Be Taken For the Transportation of Model No. Spec. 2-T Radiographic Exposure Device (6/88) | Not applicable to commercial power plants |
| 88-07 | Power Oscillations in Boiling Water Reactors (BWRs) (6/88) | Not Applicable BWR only |
| 88-07 S1 | Power Oscillations in Boiling Water Reactors (BWRs) (12/88) | Not Applicable BWR only |
| 88-08 | Thermal Stresses in Piping Connected to Reactor Coolant Systems (6/88) | SSAR Section 5.4.10.1 |
| 88-08 S1 | Thermal Stresses in Piping Connected to Reactor Coolant Systems (6/88) | SSAR Section 5.4.10.1 |
| 88-08 S2 | Thermal Stresses in Piping Connected to Reactor Coolant Systems (8/88) | SSAR Section 5.4.10.1 |

I.E. BULLETIN

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 88-09 | Thimble Tube Thinning In Westinghouse Reactors (7/88) | Not Applicable AP600 does not have this design |
| 88-10 | Nonconforming Molded-Case Circuit Breakers (11/88) | Not Applicable Procurement issue |
| 88-11 | Pressurizer Surge Line Thermal Stratification (12/88) | SSAR Section 5.4.10.1 |
| 89-01 | Failure of Westinghouse Steam Generator Tube Mechanical Plugs (5/89) | Not Applicable Procurement issue |
| 89-01 S1 | Failure of Westinghouse Steam Generator Tube Mechanical Plugs (11/90) | Not Applicable Procurement issue |
| 89-01 S2 | Failure of Westinghouse Steam Generator Tube Mechanical Plugs (6/91) | Not Applicable Procurement issue |
| 89-01 S1 | Failure of Westinghouse Steam Generator Tube Mechanical Plugs (9/91) | Not Applicable Procurement issue |
| 89-02 | Stress Corrosion Cracking of High Hardness Type 410 Stainless Steel Internal Preloaded Bolting in Anchor Darling Model S350W Swing Check Valves or Valve of Similar Design (7/89) | Not Applicable Procurement issue |
| 89-03 | Potential Loss of Required Shutdown Margin During Refueling Operations (11/89) | Not Applicable Procedural issue |
| 90-01 | Loss of Fill Oil in Transmitters Manufactured By Rosemount (3/90) | Not Applicable BWR only |
| 90-02 | Loss of Thermal Margin Caused By Channel Box Bow (3/90) | Not Applicable BWR only |
| 91-01 | Reporting Loss of Criticality Safety Controls (10/91) | Not applicable to commercial nuclear power plants |

GENERIC LETTERS

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 80-001 | NUREG-0630, "Cladding, Swelling and Rupture Models for LOCA Analysis" (1/80) | Not Applicable Administrative Communication |
| 80-002 | Quality Assurance Requirements Regarding Diesel Generator Fuel Oil (1/80) | Not Applicable SSAR Section 1.9 |
| 80-003 | BWR Control Rod Failures (1/80) | Not Applicable BWR Only |
| 80-004 | IEB 80-01 Operability of ADS Valve Pneumatic Supply (1/80) | See Bulletin 80-01 |
| 80-005 | IEB 79-01B Environmental Qualification of Class 1E Equipment | Backfit EQ design issue SSAR Sections 3.11, 1.9 |
| 80-006 | Issuance of NUREG-0313, Rev. 1, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping" | Not Applicable BWR Only |
| 80-007 | Information Regarding the Program for Environmental Qualification of Safety Related Electrical Equipment (1/80) | Not Issued |
| 80-008 | IEB 80-02 Inadequate Quality Assurance for Nuclear Supplied Equipment (1/80) | Not Applicable BWR Only |
| 80-009 | Low Level Radioactive Waste Disposal (1/80) | SSAR Section 11.4 |
| 80-010 | Issuance of NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment" | Backfit EQ design issue SSAR Sections 3.11, 1.9 |
| 80-011 | IEB 80-03 Loss of Charcoal from Standard Type II, 2 Inch, Tray Adsorber Cells (2/80) | See Bulletin 80-03 |
| 80-012 | IEB 80-04 Analysis of a PWR Main Steam Line Break with Continued Feedwater Addition (2/80) | See Bulletin 80-04 |
| 80-013 | Qualification of Safety-Related Electrical Equipment (2/80) | Backfit EQ design issue SSAR Sections 3.11, 1.9 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 80-014 | LWR Primary Coolant System Pressure Isolation Valves (2/80) | SSAR Section 1.9 |
| 80-015 | Request for Additional Management and Technical Resources Information (2/80) | Not Applicable Administrative Communication |
| 80-016 | IEB 79-01B Environmental Qualification of Class 1E Equipment (2/80) | Backfit EQ design issue SSAR Sections 3.11, 1.9 |
| 80-017 | Modifications to BWR Control Rod Drive Systems (3/80) | Not Applicable BWR Only |
| 80-018 | Crystal River 3 Reactor Trip from Approximately 100% Full Power (3/80) | Not Applicable B & W Facilities Only |
| 80-019 | Resolution of Enhanced Fission Gas Release Concern (3/80) | No Action Required |
| 80-020 | Action Required from OL Applicants of NSSS Designs by W and CE Resulting from NRC B&O Task Force Review of TMI-2 Accident (3/80) | AP600 design does not require a safety-grade AFW system |
| 80-021 | IEB 80-05 Vacuum Condition Resulting in Damage to Chemical Volume Control System Holdup Tanks (3/80) | See Bulletin 80-05 |
| 80-022 | Transmittal of NUREG-0654 "Criteria for Preparation and Evaluation of Radiological Emergency Response Team | Not Applicable Part of COL SSAR Section 13.3 |
| 80-023 | Change of Submittal Date for Evaluation Time Estimates (3/80) | Not Applicable Administrative Communication |
| 80-024 | NRC Nuclear Data Link (3/80) | Not Applicable Administrative Communication |
| 80-025 | IEB 80-06 Engineering Safety Features (ESF) Reset Controls (3/80) | See Bulletin 80-06 |
| 80-026 | Qualifications of Reactor Operators (3/80) | Not Applicable Part of COL |
| 80-027 | IEB 80-07 BWR Jet Pump Assembly Failure (4/80) | Not Applicable BWR Only |
| 80-028 | IEB 80-08 Examination of Containment Liner Penetration Welds (4/80) | See Bulletin 80-08 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 80-029 | Modifications to BWR Control Rod Drive Systems (4/80) | Not Applicable BWR Only |
| 80-030 | Clarification of the Term "Operable" as it applies to Single Failure Criterion for Safety Systems Required by TS (4/80) | Definition found in SSAR Section 16.1 |
| 80-031 | IEB 80-09 Hydramotor Actuator Deficiencies (4/80) | See Bulletin 80-09 |
| 80-032 | Information Request on Category I Masonry Walls Employed by Plants under CP and OL review (4/80) | Not Applicable AP600 design has no safety-related masonry walls |
| 80-033 | Actions Required from OL applicants of B&W Designed NSSS Resulting from NRC B&O Task Force Review of TMI-2 Accident (4/80) | Not Applicable B & W Facilities Only |
| 80-034 | Clarification of NRC Requirements for Emergency Response Facilities at Each Site (4/80) | SSAR Section 18.11.2 |
| 80-035 | Effect of a DC Power Supply Failure on ECCS Performance (4/80) | Not Applicable GE BWR Facilities Only |
| 80-036 | IEB 80-10 Containment of a Non-Radioactive System and Resulting Potential for Unmonitored, Uncontrolled Release to Environment (5/80) | See Bulletin 80-10 |
| 80-037 | Five additional TMI-2 Related Requirements to Operating Reactors (5/80) | SSAR Section 1.9.3 |
| 80-038 | Summary of Certain Non-power Reactor Physical Protection Requirements (5/80) | Not applicable to commercial nuclear power plants |
| 80-039 | IEB 80-11 Masonry Wall Design (5/80) | See Bulletin 80-11 |
| 80-040 | Transmittal of NUREG-0645 and NUREG-0626 (5/80) | Not Applicable GE Facilities Only |
| 80-041 | Summary of Meetings Held on April 22-23, 1980 with Representatives of the Mark I Owners Group (5/80) | Not Applicable Mark I Containment facilities Only |
| 80-042 | IEB 80-12 Decay Heat Removal System Operability (5/80) | See Bulletin 80-12 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 80-043 | IEB 80-13 Cracking in Core Spray Spargers (5/80) | Not Applicable BWR Only |
| 80-044 | Reorganization of Functions and Assignments Within ONRR/SSPB (5/80) | Not applicable to commercial nuclear power plants |
| 80-045 | Fire Protection Rule (5/80) | SSAR Section 9.5.1 |
| 80-046 | Generic Technical Activity A-12 Fracture Toughness (5/80) | USI A-12 SSAR Section 1.9.4.1 |
| 80-047 | Additional Guidance on "Potential for Low Fracture Toughness and Laminar Testing on PWR Steam Generator and Reactor Coolant Pump Supports" NUREG-0577 (5/80) | USI A-12 SSAR Section 1.9.4.1 |
| 80-048 | Revision to 05/19/80 Letter on Fire Protection (5/80) | SSAR Section 9.5.1 Generic Letter 80-45 |
| 80-049 | Nuclear Safeguards Problems (6/80) | Not Applicable Administrative Communication |
| 80-050 | Generic Activity A-10 BWR Cracks (6/80) | Not Applicable BWR only |
| 80-051 | Temporary On-Site Storage of Low-Level Radioactive Waste (6/80) | Not Applicable Part of COL |
| 80-052 | Five Additional TMI-2 Related Requirements -Errata Sheets to 5/7/80 Letter (6/80) | Not Applicable Administrative Communication |
| 80-053 | Decay Heat Removal Capability (6/80) | SSAR Section 16.3.5 |
| 80-054 | IEB 80-14 Degradation of Scram Discharge Volume Capability (6/80) | Not Applicable BWR only |
| 80-055 | IEB 80-15 Possible Loss of Hotline with Loss of Offsite Power (6/80) | See Bulletin 80-15 |
| 80-056 | NRC Memorandum and Order on Equipment Qualification (6/80) | SSAR Sections 3.11, Appendix 3D |
| 80-057 | Further Commission Guidance for Power Reactor Operating Licenses NUREG-0660 and NUREG-0694 (6/80) | SSAR Section 1.9.3 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|--|--|
| 80-058 | IEB 80-16 Potential Misapplication of Rosemount 1151/1152 Pressure Transmitters with "A" or "D" Output Codes (6/80) | See Bulletin 80-16 |
| 80-059 | Transmittal of Federal Register Notice RE Regional Meetings to Discuss Environmental Qualification of Electrical Equipment (6/80) | Not Applicable Administrative Communication |
| 80-060 | Request for Information Regarding Evacuation Times (7/80) | Not Applicable Administrative Communication |
| 80-061 | TMI-2 Lessons Learned (7/80) | SSAR Section 1.9.3 |
| 80-062 | TMI-2 Lessons Learned (7/80) | Not Applicable BWR only |
| 80-063 | IEB 80-17 Failure of Control Rod to Insert During a Scram at a BWR (7/80) | See Bulletin 80-17 |
| 80-064 | Scram Discharge Volume Design (7/80) | Not Applicable BWR only |
| 80-065 | Request for Estimated Construction Completion and Fuel Load Schedules (7/80) | Not Applicable Administrative Communication |
| 80-066 | IEB 80-17 Supplement 1 Failure of Control Rods to Insert During a Scram at a BWR (7/80) | Not Applicable BWR only |
| 80-067 | Scram Discharge Volume (7/80) | Not Applicable BWR only |
| 80-068 | IEB 80-17 Supplement 2 Failures Revealed by Testing Subsequent to Failure of Control Rods to Insert During a Scram at a BWR (7/80) | Not Applicable BWR only |
| 80-069 | IEB 80-18 Maintenance of Adequate Minimum Flow Through Centrifugal Charging Pumps Following Secondary Side HELB (7/80) | See Bulletin 80-18 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|--|--|
| 80-070 | IEB 80-19 Failure of Mercury Wetted Matrix Relays in RPS of Operating Nuclear Power Plants Designed by GE (7/80) | Not Applicable GE plants only |
| 80-071 | IEB 80-20 Failures of Westinghouse Type W-2 Spring Return to Neutral Control Switches (7/80) | See Bulletin 80-20 |
| 80-072 | Interim Criteria for Shift Staffing (7/80) | SSAR Section 1.9.3, 18.7 |
| 80-073 | Functional Criteria for Emergency Response Facilities NUREG-0696 (8/80) | Not Applicable Administrative communication |
| 80-074 | Notice of Forthcoming Meeting with the Representatives of EPRI to Discuss Program for Resolution of USI A-12 Fracture Toughness (8/1/80) | Not Applicable Administrative communication |
| 80-075 | Lesson Learned Tech Specs (8/80) | Not Applicable Administrative Communication |
| 80-076 | Notice of Forthcoming Meeting with GE to Discuss Proposed BWR Feedwater Nozzle Leakage Detection System (8/80) | Not Applicable BWR only |
| 80-077 | Refueling Water Level (8/80) | SSAR Section 16.3.9 |
| 80-078 | Mark I Containment Long Term Program (8/80) | Not Applicable BWR only |
| 80-079 | IEB 80-17 Supplement 3 Failure Revealed by Testing Subsequent to Failure of Control Rods to Insert During a Scram at a BWR (8/80) | Not Applicable BWR only |
| 80-080 | Preliminary Clarification of TMI Action Plan Requirements (9/80) | SSAR Section 1.9.3 |
| 80-081 | Preliminary Clarification of the TMI Action Plan Requirements Addendum to 9/5/80 Letter (9/80) | SSAR Section 1.9.3 |
| 80-082 | IEB 79-01B Supplement 2 Environmental Qualification of Class 1E Equipment (9/80) | Backfit EQ design issue SSAR Sections 3.11, 1.9 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 80-083 | Environmental Qualification of Safety Related Equipment (10/80) | Backfit EQ design issue SSAR Section 3.11, 1.9 |
| 80-084 | BWR Scram System (10/80) | Not Applicable BWR only |
| 80-085 | Implementation of Guidance from USI A-12 "Potential for Low Fracture Toughness and Lamellar Tearing on Component Supports (10/80) | USI A-12 SSAR Section 1.9.4.1 |
| 80-086 | Notice of Meeting to Discuss Final Resolution of USI A-12 (10/80) | Not Applicable Administrative communication |
| 80-087 | Notice of Meeting to Discuss Status of EPRI Proposed Resolution of the USI A-12 Fracture Toughness Issue (10/80) | Not Applicable Administrative communication |
| 80-088 | Seismic Qualification of Auxiliary Feedwater Systems (10/80) | AP600 SUFW System does not require seismic qualification SSAR Section 10.4.9 |
| 80-089 | IEB 79-01B Supplement 3 Environment Qualification of Class 1 E Equipment (10/80) | Backfit EQ design issue SSAR Sections 3.11, 1.9 |
| 80-090 | Post - TMI Requirements, NUREG-0737 (10/80) | SSAR Section 1.9.3 |
| 80-091 | ODYN Code Calculation (11/80) | Not Applicable BWR only |
| 80-092 | IEB 80-21 Valve Yokes Supplied By Malcom Foundry Company, Inc. (11/80) | Not Applicable Procurement issue |
| 80-093 | Emergency Preparedness (11/80) | Not applicable to commercial nuclear power plants |
| 80-094 | Emergency Plan (11/80) | Not Applicable Part of COL |
| 80-095 | Generic Activity A-10 (11/80) | Not Applicable BWR only |
| 80-096 | Fire Protection (11/80) | SSAR Section 9.5.1 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 80-097 | IEB 80-23 Failures of Solenoid Valves Manufactured by Valcor Engineering Corp. (11/80) | Not Applicable Procurement issue |
| 80-098 | IEB 80-24 Prevention of Damage Due to Water Leakage Inside Containment (11/80) | See Bulletin 80-24 |
| 80-099 | Technical Specification Revisions for Snubber Surveillance (11/80) | SSAR Section 5.2.4 Part of COL |
| 80-100 | Appendix R to 10 CFR Part 50 Regarding Fire Protection Federal Register Notice (11/80) | SSAR Section 9.5.1 |
| 80-101 | Inservice Inspection Programs (11/80) | Not Applicable Administrative communication |
| 80-102 | NRC Memorandum and Order of May 23, 1980 (11/80) | Backfit EQ design issue SSAR Sections 3.11, 1.9 |
| 80-103 | Fire Protection - Revised Federal Register Notice (11/80) | Not Applicable Administrative communication |
| 80-104 | Orders on Environmental Qualification of Safety Related Electrical Equipment (11/80) | Not Applicable Administrative communication |
| 80-105 | Implementation of Guidance for USI A-12 Potential for Low Fracture Toughness and Lamellar Tearing on Component Supports (11/80) | USI A-12 SSAR Section 1.9.4.1 |
| 80-106 | Report on ECCS Cladding Models, NUREG-0630 (11/80) | Not Applicable Administrative communication |
| 80-107 | BWR Scram Discharge System (12/80) | Not Applicable BWR only |
| 80-108 | Emergency Planning (12/80) | Not Applicable Part of COL |
| 80-109 | Guidelines for SEP and Soil Structure Interaction Reviews (12/80) | Not Applicable Part of COL |
| 80-110 | Periodic Updating of FSARs (12/80) | Not Applicable Administrative communication |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 80-111 | IEB 80-17 Supplement 4 Failure of Control Rods to Insert During a Scram at a BWR (12/80) | Not Applicable BWR only |
| 80-112 | IEB 80-25 Operating Problems with Target Rock Safety Relief Valves (12/80) | Procurement issue not design issue |
| 80-113 | Control of Heavy Loads (12/80) | USI-36 SSAR Section 1.9 |
| 81-01 | Qualification of Inspection, Examination, and Testing of Audit Personnel (5/81) | Not Applicable Regulatory Guide 1.58 was withdrawn SSAR Section 1.9 |
| 81-02 | Analysis, Conclusions, and Recommendations Concerning Operator Licensing (1/81) | Not Applicable Part of COL |
| 81-03 | Implementation of NUREG-0313, Rev. 1, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping (Generic Task A-42) (2/81) | Not Applicable BWR only |
| 81-04 | Emergency Procedures and Training for Station Blackout Events (2/81) | USI A-44 SSAR Section 1.9 |
| 81-05 | Information Regarding the Program for Environmental Qualification of Safety-Related Electrical Equipment (1/81) | USI A-24 AP600 EQ SSAR Sections 3.11, 1.9 |
| 81-06 | Periodic Updating of Final Safety Analysis Reports (FSARs) (2/81) | Not Applicable Administrative communication |
| 81-07 | Control of Heavy Loads (2/81) | USI A-36 SSAR Section 1.9 |
| 81-08 | ODYN Code (1/81) | Not Applicable GE BWR only |
| 81-09 | BWR Scram Discharge System (1/81) | Not Applicable BWR only |
| 81-10 | Post-TMI Requirements for the Emergency Operations Facility (2/81) | SSAR Chapter 18 Part of COL |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 81-11 | NUREG-0619, "BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking" (Corrections) (2/81) | Not Applicable BWR only |
| 81-12 | Fire Protection Rule (2/81) | SSAR Section 9.5.1 |
| 81-13 | SER for GEXL Correlation for 8 X 8 R Fuel Reload Application per the Appendix D Submittals of the General Electric Topical Report, NEDE-24011-P-A, dated 2/28/79 and 12/14/79 (5/81) | Not Applicable BWR only |
| 81-14 | Seismic Qualification of Auxiliary Feedwater Systems (2/81) | AP600 SUFW system does not require seismic qualification SSAR Section 10.4.9 |
| 81-15 | Environmental Qualification of Class 1E Electrical Equipment; Clarification of Staff Handling of Proprietary Information (3/81) | Not Applicable Administrative communication |
| 81-16 | NUREG-0737, Item I.C.I, Abnormal Transient Operator Guidelines (ATOG) (6/81) | Not Applicable B & W facilities only |
| 81-17 | Functional Criteria for Emergency Response Facilities (3/81) | SSAR Chapter 18 Part of COL |
| 81-18 | BWR Scram Discharge System; Clarification of Diverse Instrumentation Requirements (3/81) | Not Applicable BWR only |
| 81-19 | Thermal Shock to Reactor Pressure Vessels (4/81) | USI A-49 SSAR Section 1.9 |
| 81-20 | Safety Concerns Associated with Pipe Breaks in the BWR Scram System (4/81) | Not Applicable BWR only |
| 81-21 | Natural Circulation Cooldown (5/81) | Not Applicable Procedural/Training issue |
| 81-22 | Engineering Evaluation of the H.B. Robinson Reactor Coolant System Leak on January 29, 1981 (5/81) | Not Applicable No action defined |
| 81-23 | Institute of Nuclear Power Operations (INPO) Evaluation Reports (6/81) | Not Applicable Administrative communication |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 81-23A | Institute of Nuclear Power Operations (INPO) Evaluation Reports (7/81) | Not Applicable Administrative communication |
| 81-24 | Multi-Plant Issue B-56 Control Rods Fail to Fully Insert (6/81) | Not Applicable GE BWR only |
| 81-25 | Change in Implementing Schedule for Submission and Evaluation of Upgraded Emergency Plans (6/81) | Not Applicable Administrative communication |
| 81-26 | Licensing Requirements for Pending Construction Permit and Manufacturing License Application (7/81) | SSAR Section 1.9 |
| 81-27 | Privacy and Proprietary Material in Emergency Plans ((7/81) | Not Applicable Administrative communication |
| 81-28 | Steam Generator Overfill (7/81) | USI A-47 SSAR Section 1.9 |
| 81-29 | Simulator Examinations (8/81) | Not Applicable Administrative communication |
| 81-30 | Safety Concerns Associated with Pipe Breaks in the BWR Scram System (7/81) | Not Applicable BWR only |
| 81-31 | Small Break LOCA Confirmatory Integral Systems Experiments for B&W Designed Plants (no date) | Not issued |
| 81-32 | NUREG-0737, Item II.K.3.44 - Evaluation of Anticipated Transients Combined with Single Failure (8/81) | Not Applicable BWR only |
| 81-33 | Technical Specification for Station Batteries Multiple Action (no date) | Not issued |
| 81-34 | Safety Concerns Associated with Pipe Breaks in the BWR Scram System (8/81) | Not Applicable BWR only |
| 81-35 | Safety Concerns Associated with Pipe Breaks in the BWR Scram System (8/81) | Not Applicable BWR only |
| 81-36 | Revised Schedule for Completion of TMI Action Plan Item II.D.1 Relief and Safety Valve Testing (9/81) | Not Applicable Administrative communication |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 81-37 | ODYN Code Reanalysis Requirements (no date) | Not issued |
| 81-38 | Storage of Low-Level Radioactive Waste at Power Reactor Sites (11/81) | Not Applicable Part of COL |
| 81-39 | NRC Volume Reduction Policy (11/81) | SSAR Section 11.4 |
| 81-40 | Qualification of Reactor Operators - License Examinations | Part of COL |
| 82-01 | New Application Survey (1/82) | Not Applicable Administrative communication |
| 82-02 | Nuclear Power Plant Staff Working Hours (2/82) | Part of COL |
| 82-03 | High Burnup MAPLHGR Limits (3/82) | Not Applicable GE BWR only |
| 82-04 | Use of Institute of Power Operations (INPO) SEE-IN Program (no date) | Not Applicable Administrative communication |
| 82-05 | Post-TMI Requirements (3/82) | Not Applicable Administrative communication |
| 82-06 | RTD Response Time Determination (no date) | Not issued |
| 82-07 | Transmittal of NUREG-0909 Relative to Ginna Tube Rupture (4/82) | Not Applicable BWR transmittal only |
| 82-08 | Transmittal of NUREG-0909 Relative to Ginna Tube Rupture (4/82) | Not Applicable Administrative communication PWR transmittal only |
| 82-09 | Environmental Qualification of Safety-Related Electrical Equipment (4/82) | AP600 Equipment Qualification SSAR Sections 1.9, 3.11 |
| 82-10 | Post-TMI Requirements (5/80) | Not Applicable Administrative communication |
| 82-11 | Transmittal of NUREG-0916 Relative to the Restart of R.E.Ginna Nuclear Power Plant (6/82) | Not Applicable Administrative communication |
| 82-12 | Nuclear Power Plant Staff Working Hours (6/82) | Not Applicable Part of COL |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 82-13 | Reactor Operator and Senior Reactor Operator Examinations (6/82) | Not Applicable Part of COL |
| 82-14 | Submittal of Documents to the Nuclear Regulatory Commission (8/82) | Not Applicable Administrative communication |
| 82-15 | Nuclear Plant Staff Working Hours (no date) | Not issued |
| 82-16 | NUREG-0737 Technical Specifications (9/82) | SSAR Chapter 16 |
| 82-17 | Inconsistency Between Requirements of 10 CFR 50.54(t) and Standard Technical Specifications for Performing Audits of Emergency Preparedness Programs (10/82) | Not Applicable Not T.S. Requirement |
| 82-18 | Reactor Operator and Senior Reactor Operator Examinations (10/82) | Not Applicable Part of COL |
| 82-19 | Submittal of Copies of Documents to the NRC (10/82) | Not applicable to commercial nuclear power plants |
| 82-20 | Guidance for Implementing Standard Review Plan Rule (10/82) | SSAR Section 1.9.2 |
| 82-21 | Technical Specifications for Fire Protection Audits (10/82) | Not Applicable Not T.S. Requirement |
| 82-22 | Congressional Request for Information Concerning Steam Generator Tube Integrity (10/82) | USI A-3 SSAR Section 1.9.4 |
| 82-23 | Inconsistency Between Requirements of 10CFR 73.40(d) and Standard Technical Specifications for Performing Audits of Safeguards Contingency Plans (Security Plan) (10/82) | Not Applicable Not T.S. Requirement |
| 82-24 | Safety Relief Valve Quencher Loads: Evaluation for BWR Mark II and III Containments (11/82) | Not Applicable BWR only |
| 82-25 | Integrated IAEA Exercise for Physical Inventory at LWRs (11/82) | Not Applicable Administrative communication |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 82-26 | NUREG-0744 Revision 1; Pressure Vessel Material Fracture Toughness (11/82) | USI A-11 SSAR Section 1.9.4 |
| 82-27 | Transmittal of NUREG-0763 and NUREG-0783 (11/82) | Not Applicable BWR only |
| 82-28 | Inadequate Core Cooling Instrumentation System (12/82) | SSAR Section 1.9.3 |
| 82-29 | NUREG/CR-2980 (no date) | Not issued |
| 82-30 | Filings Related to 10 CFR Part 50 Production and Utilization Facilities (12/82) | Not applicable to commercial nuclear power plants |
| 82-31 | NUREG-0737 Technical Specifications (no date) | Not issued |
| 82-32 | Potential Steam Generator Related Generic Requirements (12/82) | USI A-3 SSAR Section 1.9.4 |
| 82-33 | Supplement 1 to NUREG-0737 - Requirements for Emergency Response Capability (12/82) | SSAR Section 1.9.3 |
| 82-34 | SECY 82-111 Modification of Vacuum Breakers on Mark I Containment (no date) | Not issued |
| 82-35 | No Title (no date) | Not issued |
| 82-36 | Assignment of Authority to Regions for Nonpower Reactor Amendments and Licensing Activities (no date) | Not issued |
| 82-37 | NUREG-0737 Technical Specifications (no date) | Not issued |
| 82-38 | Meeting to Discuss Recent Developments for Operating Licensing Examinations (12/82) | Not Applicable Administrative communication |
| 82-39 | Problems With the Submittals of 10 CFR 73.21 Safeguards Information for Licensing Review (12/82) | Not Applicable Administrative communication |
| 83-01 | Operating Licensing Examination Site Visit (1/83) | Not Applicable Administrative communication |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 83-02 | NUREG-0737 Technical Specifications (1/83) | Not Applicable BWR only |
| 83-03 | No Title (no date) | Not issued |
| 83-04 | Regional Workshops Regarding Supplement 1 to NUREG-0737, Requirements for Emergency Response Capability (2/83) | Not Applicable Administrative communication |
| 83-05 | Safety Evaluation of "Emergency Procedure Guideline, Revision 2," NEDO-24934 (2/83) | Not Applicable BWR only |
| 83-06 | Certificates and Revised Format for Reactor Operator and Senior Reactor Operator Licenses (1/83) | Not Applicable Part of COL |
| 83-07 | The Nuclear Waste Policy Act of 1982 (2/83) | Not Applicable Administrative communication |
| 83-08 | Modification of Vacuum Breakers on Mark I Containments (2/83) | Not Applicable GE BWR only |
| 83-09 | Review of Combustion Engineering Owners Group Emergency Procedures Guidelines Program (2/83) | Not Applicable CE facilities only |
| 83-10a | Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps (CE Applicants) (2/83) | Not Applicable CE facilities only |
| 83-10b | Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps (CE licensees) (2/83) | Not Applicable CE facilities only |
| 83-10c | Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps (Westinghouse applicants) (2/83) | SSAR Section 7.3.1.1.3.3 |
| 83-10d | Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps (Westinghouse licensees) (2/83) | SSAR Section 7.3.1.1.3.3 |
| 83-10e | Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps (B&W applicants) (2/83) | Not Applicable B & W facilities only |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|--|--|
| 83-10f | Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps (B&W licensees) (2/83) | Not Applicable B & W facilities only |
| 83-11 | Licensee Qualification for Performing Safety Analysis in Support of Licensing Actions (2/83) | Not Applicable Administrative communication |
| 83-12 | Issuance of NRC Form 398 Personal Qualifications Statement - Licensee (2/83) | Not Applicable Part of COL |
| 83-12A | Issuance of NRC Form 398 Personal Qualifications Statement - Licensee (7/83) | Not applicable to commercial nuclear power plants |
| 83-13 | Clarification of Surveillance Requirements for HEPA Filters and Charcoal Adsorber Units in Standard Technical Specifications (STS) on ESF Cleanup Systems (3/83) | Not Applicable AP600 design has no safety-related ventilation systems |
| 83-14 | Definition of "Key Maintenance Personnel" (3/83) | Not Applicable Administrative communication |
| 83-15 | Implementation of Regulatory Guide 1.150, "Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations, Revision 1" (3/83) | R.G. 1.150 SSAR Appendix 1A |
| 83-16 | Transmittal of NUREG-0977 relative to the ATWS Events at Salem Generating Station Unit 1 (3/83) | See Generic Letter 83-28 |
| 83-16A | Transmittal of NUREG-0977 relative to the ATWS Events at Salem Generating Station Unit 1 (3/83) | See Generic Letter 83-28 |
| 83-17 | Integrity of the Requalification Examinations for Renewal of Reactor Operator and Senior Reactor Operator Licenses (4/83) | Not Applicable Part of COL |
| 83-18 | NRC Staff Review of the BWR Owners Group (BWROG) Control Room Survey Program (4/83) | Not Applicable BWR only |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 83-19 | New Procedure for Providing Public Notice Concerning Issuance of Amendments to Operating Licenses (5/83) | Not Applicable Administrative communication |
| 83-20 | Integrated Schedule for Implementation of Plant Modifications (5/83) | Not Applicable Administrative communication |
| 83-21 | Clarification of Access Control Procedures for Law Enforcement Visits (5/83) | Not Applicable Administrative communication |
| 83-22 | Safety Evaluation of "Emergency Response Guidelines" (6/83) | Not Applicable Part of COL |
| 83-23 | Safety Evaluation of "Emergency Procedure Guidelines" (7/83) | Not Applicable CE facilities only |
| 83-24 | TMI Task Action Plan Item I.G.1, "Special Low Power Testing and Training," Recommendations for BWRs (6/83) | Not Applicable BWR only |
| 83-25 | No Title (no date) | Not issued |
| 83-26 | Clarification of Surveillance Requirements for Diesel Fuel Impurity Tests (7/83) | Not applicable Regulatory Guide 1.137 SSAR Sections 1.9 |
| 83-27 | Surveillance Intervals in Standard Technical Specifications (7/83) | SSAR Chapter 16, Section 3.0 |
| 83-28 | Required Actions Based on Generic Implications of Salem ATWS Events | SSAR Section 7.1.2.2.4 |
| 83-29 | No Title (no date) | Not issued |
| 83-30 | Deletion of Standard Technical Specification Surveillance Requirement 4.8.1.1.2.d.6 for Diesel Generator Testing (7/83) | Not applicable Regulatory Guide 1.108 SSAR Section 1.9 |
| 83-31 | Safety Evaluation of "Abnormal Transient Operating Guidelines" (9/83) | Not Applicable B & W facilities only |
| 83-32 | NRC Staff Recommendations Regarding Operator Action for Reactor Trip and ATWS (12/83) | SSAR Section 18.8.2.1.2 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 83-33 | NRC Positions on Certain Requirements of Appendix R to 10 CFR Part 50 (10/83) | SSAR Section 9.5.1 |
| 83-34 | No Title (no date) | Not issued |
| 83-35 | Clarification of TMI Action Plan Item II.K.3.31 (11/83) | SSAR Section 15.6.5.4B.1.0 |
| 83-36 | NUREG-0737 Technical Specifications (11/83) | Not Applicable BWR only |
| 83-37 | NUREG-0737 Technical Specifications (11/83) | SSAR Chapter 16 |
| 83-38 | NUREG-0965, "NRC Inventory of Dams" (10/83) | Not Applicable Administrative communication |
| 83-39 | Voluntary Survey of Licensed Operators (12/83) | Not Applicable Administrative communication |
| 83-40 | Operator Licensing Examination (12/83) | Not Applicable Part of COL |
| 83-41 | Fast Cold Starts of Diesel Generators (12/83) | SSAR Section 16.2 |
| 83-42 | Clarification to Generic Letter 81-07 Regarding Responses to NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants" (12/83) | USI A-36 SSAR Section 1.9.4 |
| 83-43 | Reporting Requirements of 10 CFR 50.72, 10 CFR 50.73, and Standard Technical Specifications (STS) (12/83) | Not Applicable Administrative communication |
| 83-44 | Availability of NUREG-1021, "Operator Licensing Examiner Standards" (12/83) | Not Applicable Administrative communication |
| 84-01 | NRC Use of the Terms, "Important to Safety" and "Safety Related" (1/84) | SSAR Chapter 17 |
| 84-02 | Notice of Meeting Regarding Facility Staffing (1/84) | Not Applicable Administrative communication |
| 84-03 | A Prioritization of Generic Safety Issues (1/84) | Not Applicable Administrative communication |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 84-04 | Safety Evaluation of Westinghouse Topical Reports on Elimination of Postulated Pipe Breaks in PWR Primary Main Loops (2/84) | USI A-2 SSAR Section 1.9.4 |
| 84-05 | Change to NUREG-1021, "Operator Licensing Examiner Standards (4/84) | Not Applicable Administrative communication |
| 84-06 | Operator and Senior Operator License Examination Criteria for Passing Grade (4/84) | Not Applicable to commercial nuclear power plants |
| 84-07 | Procedural Guidance for Pipe Replacement at BWRs (3/84) | Not Applicable BWR only |
| 84-08 | Interim Procedure for NRC Management of Plant-Specific Backfitting (4/84) | Not Applicable Administrative communication |
| 84-09 | Recombiner Capability Requirements of 10 CFR 50.44 (c)(3)(ii) (5/84) | SSAR Section 6.2.4 |
| 84-10 | Administration of Operating Tests Prior to Initial Criticality (10 CFR 55.25) (5/84) | Not Applicable Part of COL |
| 84-11 | Inspections of BWR Stainless Steel Piping (5/84) | Not Applicable BWR only |
| 84-12 | Compliance with 10 CFR Part 61 and Implementation of the Radiological Effluent Technical Specifications (RETS) and Attendant Process Control Program (PCP) (4/84) | SSAR Chapter 16, Section 5.7 |
| 84-13 | Technical Specification for Snubbers (5/84) | Inservice Testing Program |
| 84-14 | Replacement and Requalification Training Program (5/84) | Not Applicable Training issue |
| 84-15 | Proposed Staff Actions to Improve and Maintain Diesel Generator Reliability (7/84) | SSAR Section 16.2 |
| 84-16 | Adequacy of On-Shift Operating Experience for Near Term Operating License Applicants (6/84) | SSAR Chapter 16, Section 5.2 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 84-17 | Annual Meeting to Discuss Recent Developments Regarding Operator Training, Qualifications and Examinations (7/84) | Not Applicable Part of COL |
| 84-18 | Filing of Applications for Licenses and Amendments (7/84) | Not Applicable Administrative communication |
| 84-19 | Availability of Supplement 1 to NUREG-0933, "A Prioritization of Generic Safety Issues" (8/84) | Not Applicable Administrative communication |
| 84-20 | Scheduling Guidance for Licensee Submittals of Reloads that Involve Unreviewed Safety Questions (8/84) | Not Applicable Administrative communication |
| 84-21 | Long Term Low Power Operation in PWRs (10/84) | Not Applicable Administrative communication |
| 84-22 | No Title (no date) | Not issued |
| 84-23 | Reactor Vessel Water Level Instrumentation in BWRs (10/84) | Not applicable BWR only |
| 84-24 | Certification of Compliance to 10 CFR 50.49 Environmental Qualification of Electrical Equipment Important to Safety for Nuclear Power Plants (12/84) | SSAR Sections 7.4, 3.11, Appendix 3D |
| 85-01 | Fire Protection Policy Steering Committee Report (1/85) | Not Applicable Administrative communication |
| 85-02 | Staff Recommended Actions Stemming from NRC Integrated Program for the Resolution of Unresolved Safety Issues Regarding Steam Generator Tube Integrity (4/85) | USI A-3 SSAR Section 1.9.4 |
| 85-03 | Clarification of Equivalent Control Capacity for Standby Liquid Control Systems (1/85) | Not applicable BWR only |
| 85-04 | Operator Licensing Examinations (1/85) | Not Applicable Part of COL |
| 85-05 | Inadvertent Boron Dilution Events (1/85) | SSAR Section 15.4.6 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 85-06 | Quality Assurance Guidance for ATWS Equipment That is Not Safety-Related (4/85) | USI A-9 SSAR Section 1.9.4 |
| 85-07 | Implementation of Integrated Schedules for Plant Modifications (5/85) | Not applicable Administrative communication |
| 85-08 | 10 CFR 20.408 Termination Reports - Format (5/85) | Not applicable Administrative communication |
| 85-09 | Technical Specifications for Generic Letter 83-28, Item 4.3 (5/85) | SSAR Chapter 16, Section 3.3.1 |
| 85-10 | Technical Specifications for Generic Letter 83-28, Item 4.3 and 4.4 (5/85) | Not applicable B & W facilities only |
| 85-11 | Completion of Phase II of "Heavy Loads at Nuclear Power Plants" NUREG-0612 (6/85) | USI A-36 SSAR Section 1.9.4 |
| 85-12 | Implementation of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" (6/85) | SSAR Section 7.3.1.1.3.3 |
| 85-13 | Transmittal of NUREG-1154 Regarding the Davis-Besse Loss of Main and Auxiliary Feedwater Event (8/85) | Not Applicable Administrative communication |
| 85-14 | Commercial Storage at Power Reactor Sites of Low-Level Radioactive Waste Not Generated by the Utility (8/85) | Not Applicable Part of COL |
| 85-15 | Information Relating to the Deadlines for Compliance with 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants" (8/85) | Not Applicable Administrative communication |
| 85-16 | High Boron Concentrations (8/85) | Not Applicable AP600 design has no BIT |
| 85-17 | Availability of Supplements 2 and 3 to NUREG-0933, "A Prioritization of Generic Safety Issues" (8/85) | Not Applicable Administrative communication |
| 85-18 | Operating Licensing Examinations (9/85) | Not Applicable Part of COL |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|--|--|
| 85-19 | Reporting Requirements on Primary Coolant Iodine Spikes (9/85) | Not Applicable Part of COL |
| 85-20 | Resolution of Generic Issue 69: High Pressure Injection/Make-Up Nozzle Cracking in Babcock and Wilcox Plants (11/85) | Not Applicable B & W facilities only |
| 85-21 | No Title (no date) | Not issued |
| 85-22 | Potential for Loss of Post-LOCA Recirculation Capability Due to Insulation Debris Blockage (12/85) | USI A-43 SSAR Section 1.9.4 |
| 86-01 | Safety Concerns Associated with Pipe Breaks in the BWR Scram System (1/86) | Not Applicable BWR only |
| 86-02 | Technical Resolution of Generic Issue B-19 Thermal Hydraulic Stability (1/86) | Not Applicable BWR only |
| 86-03 | Applications for License Amendments (2/86) | Not Applicable Administrative communication |
| 86-04 | Policy Statement on Engineering Expertise on Shift (2/86) | SSAR Section 18.7 |
| 86-05 | Implementation of TMI Action Plan Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" | Not Applicable B & W facilities only |
| 86-06 | Implementation of TMI Action Plan Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" | Not Applicable CE facilities only |
| 86-07 | Transmittal of NUREG-1190 Regarding the San Onofre Unit 1 Loss of Power and Water Hammer Event (3/86) | Not Applicable Plant specific issue |
| 86-08 | Availability of Supplement 4 to NUREG-0933, "A Prioritization of Generic Safety Issues" (3/86) | Not Applicable Administrative communication |
| 86-09 | Technical Resolution of Generic Issue B-59: (N-1) Loop Operation in BWRs and PWRs (3/86) | Not Applicable Not in AP600 design |
| 86-10 | Implementation of Fire Protection Requirements (4/86) | SSAR Section 9.5.1 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 86-11 | Distribution of Products Irradiated in Research Reactors (6/85) | Not applicable to commercial nuclear power plants |
| 86-12 | Criteria for Unique Purpose Exemption from Conversion from the Use of HEU Fuel (7/86) | Not applicable to commercial nuclear power plants |
| 86-13 | Potential Inconsistency Between Plant Safety Analyses and Technical Specifications (7/86) | Not Applicable CE and B&W facilities only |
| 86-14 | Operating Licensing Examinations (8/86) | Not Applicable Part of COL |
| 86-15 | Information Relating to the Compliance with 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants" (8/85) | SSAR Section 3.11, Appendix 3D |
| 86-16 | Westinghouse ECCS Evaluation Models (10/85) | Not Applicable SSAR Chapter 15 |
| 86-17 | Availability of NUREG-1169, "Technical Findings Related to Generic Issue C-8; Boiling Water Reactor Main Steam Isolation Valve Leakage and Leakage Treatment Methods" (10/86) | Not applicable BWR only |
| 87-01 | Public Availability of the NRC Operator Licensing Examination Question Book (1/87) | Not applicable Administrative communication |
| 87-02 | Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors, Unresolved Safety Issue (USI) A-46 (2/87) | USI A-46 SSAR Section 1.9.4 |
| 87-02 S1 | Supplemental Safety Evaluation Report No. 2 (SSER No. 2) on SQUG Generic Implementation Procedure, Revision 2, as corrected on February 14, 1992 (GIP-2) (5/87) | USI A-46 SSAR Section 1.9.4 |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 87-03 | Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors, Unresolved Safety Issue (USI) A-46 (2/87) | USI A-46 SSAR Section 1.9.4 |
| 87-04 | Temporary Exemption from Provisions of the FBI Criminal History Rule for Temporary Workers (3/87) | Not Applicable Administrative communication |
| 87-05 | Request for Additional Information-Assessment of Licensee Measures to Mitigate and/or Identify Potential Degradation of Mark I Drywells (3/87) | Not Applicable BWR only |
| 87-06 | Periodic Verification of Leak Tight Integrity of Pressure Isolation Valves (3/87) | SSAR Chapter 16, LCO 3.4.8 and 3.4.9 |
| 87-07 | Information Transmittal of Final Rulemaking for Revisions to Operator Licensing - 10 CFR 55 and Conforming Amendments (3/87) | Not Applicable Administrative communication |
| 87-08 | Implementation of 10 CFR 73.55 Miscellaneous Amendments and Search Requirements (5/87) | Not Applicable Administrative communication |
| 87-09 | Sections 3.0 and 4.0 of the Standard Technical Specifications (STS) on the Applicability of Limiting Conditions for Operation and Surveillance Requirements (6/87) | SSAR Chapter 16, Section 3.0 |
| 87-10 | Implementation of 10 CFR 73.57, Requirements for FBI Criminal History Checks (6/87) | Not Applicable Administrative communication |
| 87-11 | Relaxation in Arbitrary Intermediate Pipe Rupture Requirements (6/87) | SSAR Section 3.6.2 |
| 87-12 | Loss of Residual Heat Removal (RHR) While the Reactor Coolant System (RCS) is Partially Filled (7/87) | SSAR Section 1.9.5.1 |
| 87-13 | Integrity of Requalification Examinations at Non-power Reactors (7/87) | Not applicable to commercial nuclear power plants |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 87-14 | Operator Licensing Examinations (8/87) | Not Applicable Part of COL |
| 87-15 | Policy Statement on Deferred Plants (11/87) | Not Applicable Deferred facilities only |
| 87-16 | Transmittal of NUREG-1262, "Answers to Questions at Public Meeting Regarding Implementation of 10 CFR Part 55 on Operators Licenses" (11/87) | Not Applicable Administrative communication |
| 88-01 | NRC Position on Intragranular Stress Corrosion Cracking (IGSCC) in BWR Austenitic Stainless Steel Piping (1/88) | Not Applicable BWR only |
| 88-01 S1 | NRC Position on Intragranular Stress Corrosion Cracking (IGSCC) in BWR Austenitic Stainless Steel Piping (2/88) | Not Applicable BWR only |
| 88-02 | Integrated Safety Assessment Program II (ISAP II) (1/88) | Not Applicable Administrative communication |
| 88-03 | Resolution of Generic Safety Issue 93, "Steam Binding of Auxiliary Feedwater Pumps" (2/88) | AP600 has no safety-related AFW or similar safety-related system |
| 88-04 | Distribution of Gems Irradiated in Research Reactors (2/88) | Not applicable to commercial nuclear power plants |
| 88-05 | Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components in PWR Plants (3/88) | Not Applicable Part of COL |
| 88-06 | Removal of Organizational Charts from Technical Specification Administrative Control Requirements (3/88) | Not Applicable Administrative communication |
| 88-07 | Modified Enforcement Policy Relating to 10 CFR 50.49, "Environmental Qualification of Electrical Equipment Important to Safety for Nuclear Power Plants" (4/88) | SSAR Sections 1.9, 3.11 |
| 88-08 | Mail Sent or Delivered to the Office of Nuclear Reactor Regulation (5/88) | Not Applicable Administrative communication |
| 88-09 | Pilot Testing of Fundamental Examination (5/88) | Not Applicable BWR only |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 88-10 | Purchase of GSA Approved Security Containers (7/88) | Not Applicable Procurement issue |
| 88-11 | NRC Position on Radiation Embrittlement of Reactor Vessel Materials and its Impact on Plant Operations (7/88) | USI A-47, F. G. 1.99 SSAR Sections 1.9.4, Appendix 1A, respectively |
| 88-12 | Removal of Fire Protection Requirements from Technical Specifications (8/88) | SSAR Chapter 16, Section 5.7 |
| 88-13 | Operator Licensing Examinations (8/88) | Not Applicable Administrative communication |
| 88-14 | Instrument Air Supply System Problems Affecting Safety-Related Equipment (8/88) | SSAR Section 9.3.1 |
| 88-15 | Electric Power Systems - Inadequate Control Over Design Process (9/88) | SSAR Section 8.3.1.1.2.1 |
| 88-16 | Removal of Cycle-Specific Parameter Limits From Technical Specifications (10/88) | SSAR Chapter 16, Section 5.9 |
| 88-17 | Loss of Decay Heat Removal 10 CFR 50.54(f) (10/88) | SSAR Section 1.9.5.1 |
| 88-18 | Plant Record Storage on Optical Disk (10/88) | Not Applicable Part of COL |
| 88-19 | Use of Deadly Force by Licensee Guards to Prevent Theft of Special Nuclear Material (10/88) | Not Applicable Procedural issue |
| 88-20 | Individual Plant Examination for Severe Accident Vulnerabilities - 10 CFR 50.54(f) (11/88) | AP600 Probabilistic Risk Assessment Report |
| 88-20 S1 | Initiation of Individual Plant Examination for Severe Accident Vulnerabilities - 10 CFR 50.54(f) (11/88) | AP600 Probabilistic Risk Assessment Report |
| 88-20 S2 | Accident Management Strategies for Consideration in the Individual Plant Examination Process (4/90) | AP600 Probabilistic Risk Assessment Report |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 88-20 S3 | Completion of Containment Performance Improvement Program and Forwarding of Insights for Use in the Individual Plant Examination for Severe Accident Vulnerabilities (7/90) | AP600 Probabilistic Risk Assessment Report |
| 88-20 S4 | Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities - 10 CFR 50.54(f) (6/91) | AP600 Probabilistic Risk Assessment Report |
| 89-01 | Implementation of Programmatic Controls for Radiological Effluent Technical Specifications (RETS) in the Administrative Controls Section of the Technical Specifications and the Relocation of Procedural Details of RETS to the Offsite Dose Calculation Manual (ODCM) or to the Process Control Program (PCP) (1/89) | SSAR Chapter 16, Section 5.7 |
| 89-02 | Actions to Improve the Detection of Counterfeit and Fraudulently Marked Products (3/89) | Not Applicable Procurement issue |
| 89-03 | Operator Licensing National Examination Schedule (3/89) | Not Applicable Administrative communication |
| 89-04 | Guidance on Developing Acceptance Inservice Testing Programs (4/89) | SSAR Sections 5.2.4, 6.6 |
| 89-05 | Pilot Testing of the Fundamentals Examination (4/89) | Not Applicable Administrative communication |
| 89-06 | Task Action plan Item I.D 2 - Safety Parameter Display System - 10 CFR 50.54(f) (4/89) | SSAR Section 1.9.3 |
| 89-07 | Power Reactors Safeguards Contingency Planning for Service Vehicle Bombs (4/89) | Not Applicable Procedural issue |
| 89-08 | Erosion/Corrosion Induced Pipe Wall Thinning (5/89) | Surveillance issue Part of COL SSAR Sections 5.4.3.4, 10.3.6 |
| 89-09 | ASME Section III Component Replacements (5/89) | Not Applicable Procurement issue |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 89-10 | Safety-Related Motor Operated Valve Testing and Surveillance - 10 CFR 50.54(f) (6/89) | Not Applicable Procurement/Surveillance issue SSAR Section 3.9.6.2 |
| 89-10 S1 | Results of the Public Workshops (6/90) | Not Applicable Procurement/Surveillance issue SSAR Section 3.9.6.2 |
| 89-10 S2 | Availability of Program Descriptions (8/90) | Not Applicable Procurement/Surveillance issue SSAR Section 3.9.6.2 |
| 89-10 S3 | Consideration of the Results of NRC Sponsored Tests of Motor Operated Valves (10/90) | Not Applicable Procurement/Surveillance issue SSAR Section 3.9.6.2 |
| 89-10 S4 | Consideration of Valve Mispositioning in BWRs (2/92) | Not Applicable BWR only |
| 89-11 | Resolution of Generic Issue 101 "Boiling Water Reactor Water Level Redundancy (6/89) | Not Applicable BWR only |
| 89-12 | Operator Licensing Examinations (7/89) | Not Applicable Part of COL |
| 89-13 | Service Water System Problems Affecting Safety-Related Equipment (7/89) | SSAR Section 16.2 |
| 89-13 S1 | Service Water System Problems Affecting Safety-Related Equipment (4/90) | SSAR Section 16.2 |
| 89-14 | Line Item Improvements in Technical Specifications - Removal of the 3.25 Limit on Extending Surveillance Intervals (8/89) | SSAR Chapter 16, Section 3.0 |
| 89-15 | Emergency Response Data System (8/89) | Not Applicable Part of COL |
| 89-16 | Installation of a Hardened Wetwell Vent (9/89) | Not Applicable BWR only |
| 89-17 | Planned Administrative Changes to the NRC Operator Licensing Written Examination Process (9/89) | Not Applicable Part of COL |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 89-18 | Resolution of Unresolved Safety Issue A-17, "Systems Interactions in Nuclear Power Plants (9/89) | USI A-17 SSAR Section 1.9.4 |
| 89-19 | Request for Action Related to Resolution of Unresolved Safety Issue A-47 "Safety Implication of Control Systems in LWR Nuclear Power Plants" Pursuant to 10 CFR 50.54(f) (9/89) | USI A-27 SSAR Section 1.9.4 |
| 89-20 | Protected Area Long-Term Housekeeping (9/89) | Not applicable to commercial nuclear power plants |
| 89-21 | Request for Information Concerning Status of Implementation on Unresolved Safety Issue (USI) Requirements (10/89) | Not Applicable Administrative communication |
| 89-22 | Potential for Increased Roof Loads and Plant Area Flood Runoff Depth at Licensed Nuclear Power Plants Due to Recent Change in Probable Maximum Precipitation Criteria Developed by the National Weather Service (10/89) | SSAR Chapter 2 |
| 89-23 | NRC Response to Questions Pertaining to Implementation of 10 CFR Part 26 (10/89) | Not Applicable Administrative communication |
| 90-01 | Request for Voluntary Participation in NRC Regulatory Impact Survey (1/90) | Not Applicable Administrative communication |
| 90-02 | Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications (2/90) | SSAR Chapter 16, Section 4.2 |
| 90-03 | Relaxation of NRC Position in Generic Letter 83-28, Item 2.2 Part 2, "Vendor Interface for Safety Related Components (3/90) | Not Applicable Procurement issue |
| 90-03 S1 | Relaxation of NRC Position in Generic Letter 83-28, Item 2.2 Part 2, "Vendor Interface for Safety Related Components (5/90) | Not Applicable Procurement issue |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 90-04 | Request for Information on the Status of Licensee Implementation of Generic Safety Issues Resolved with Imposition of Requirements or Corrective Action (4/90) | Not Applicable Administrative communication |
| 90-05 | Guidance for Performing Temporary Non-Code Repair of ASME Code Class 1, 2 and 3 Piping (6/90) | Not Applicable Maintenance issue |
| 90-06 | Resolution of Generic Issue 70, "Power-Operated Relief Valve and Block Valve Reliability," and Generic Issue 94, "Additional Low Pressure Temperature Overpressure Protection for Light Water Reactors," Pursuant to 10 CFR 50.54(f) (6/90) | USI A-26 SSAR Section 1.9.4 |
| 90-07 | Operator Licensing National Examination Schedule (8/90) | Not Applicable Administrative communication |
| 90-08 | Simulation Facility Exemptions (8/90) | Not Applicable Administrative communication |
| 90-09 | Alternative Requirements for Snubber Visual Inspection Intervals and Corrective Actions (12/90) | SSAR Sections 5.2.4, 6.6 |
| 91-01 | Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from Technical Specifications (1/91) | SSAR Chapter 16 |
| 91-02 | Reporting Mishaps Involving LLW Forms Prepared for Disposal (12/90) | Not Applicable Administrative communication |
| 91-03 | Reporting of safeguards Events (3/91) | Not Applicable Administrative communication |
| 91-04 | Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle (4/91) | SSAR Chapter 16, Sections 3.1-3.9 |
| 91-05 | License Commercial-Grade Procurement and Dedication Programs (4/91) | Not Applicable Procurement issue |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 91-06 | Resolution of Generic Issue A-30, "Adequacy of Safety-Related DC Power Supplies," Pursuant to 10 CFR 50.54(f) (4/91) | SSAR Section 8.3.2 |
| 91-07 | GI 23, "Reactor Coolant Pump Seal Failure" and Its Possible Effect on Station Blackout (5/91) | Not Applicable SSAR Section 5.1.3.3 |
| 91-08 | Removal of Component Lists from Technical Specifications (5/91) | SSAR Chapter 16 |
| 91-09 | Modification of Surveillance Interval for the Electrical Protective Assemblies in Power Supplies for the Reactor Protection System (6/91) | Not Applicable BWR only |
| 91-10 | Explosive Searches at Protected Areas Portals (7/91) | Not Applicable to commercial nuclear power plants |
| 91-11 | Resolution of Generic Issues 48, "LCOs for Class 1E Vital Instrument Buses," and 49, "Interlocks and LCOs for Class 1E Tie Breakers" Pursuant to 10 CFR 50.54(f) (7/91) | Not Applicable SSAR Section 8.3.2.1.1.1 |
| 91-12 | Operator Licensing National Examination Schedule (8/91) | Not Applicable Administrative communication |
| 91-13 | Request for Information Related to the Resolution of Generic Issue 130, "Essential Service Water System Failures at Multi-Unit Sites," Pursuant to 10 CFR 50.54(f) (9/91) | SSAR Section 16.2 |
| 91-14 | Emergency Telecommunications (9/91) | Not Applicable Part of COL |
| 91-15 | Operating Experience Feedback Report, Solenoid-Operated Valve Problems at U.S. Reactors (9/91) | Not Applicable Procurement/Maintenance issue |
| 91-16 | Licensed Operator and Other Nuclear Facility Personnel Fitness for Duty (10/91) | Not Applicable Administrative communication |

GENERIC LETTERS

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 91-17 | Generic Safety Issue 29, "Bolting Degradation or Failure in Nuclear Power Plants" (10/91) | Not Applicable Procurement/Maintenance issue |
| 91-18 | Information to Licensees Regarding Two NRC Inspection Manual Sections on Resolution of Degraded and Nonconforming Conditions and on Operability (11/91) | Not Applicable Administrative communication |
| 91-19 | Information to Addressees Regarding New Telephone Numbers for NRC Offices Located in One White Flint North (12/91) | Not Applicable Administrative communication |

I.E. CIRCULARS

I.E CIRCULARS

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 80-01 | Service Advice for General Electric Induction Disc Relays (1/80) | Not Applicable Procurement issue |
| 80-02 | Nuclear Power Plant Staff Work Hours (2/80) | Not Applicable Administrative issue |
| 80-03 | Protection from Toxic Gas Hazards (3/80) | SSAR Section 6.4, 2.2 |
| 80-04 | Securing of Threaded Locking Devices on Safety-Related Equipment (3/80) | Not Applicable Maintenance/Installation issue |
| 80-05 | Emergency Diesel Generator Lubricating Oil Addition and Onsite Supply (4/80) | SSAR Section 16.2 |
| 80-06 | Control and Accountability Systems for Implant Therapy Devices (4/80) | Not applicable to commercial nuclear power plants |
| 80-07 | Problems with HPCI Turbine Oil System (4/80) | Not Applicable BWR only |
| 80-08 | BWR Technical Specification Inconsistencies - RPS Response Time (4/80) | Not Applicable BWR only |
| 80-09 | Problems with Plant Internal Communications Systems (4/80) | SSAR Section 9.5.2 |
| 80-10 | Failure to Maintain Environmental Qualification of Equipment (4/80) | Not Applicable Procedure issue |
| 80-11 | Emergency Diesel Generator Lube Oil Cooler Failures (5/80) | SSAR Section 16.2 |
| 80-12 | Valve-Shaft-To-Actuator Key May Fall Out of Place When Mounted Below Horizontal Axis (5/80) | Not Applicable Procurement issue |
| 80-13 | Grid Strap Damage in Westinghouse Fuel Assemblies (5/80) | Not Applicable Fuel Handling Procedure |
| 80-14 | Radioactive Contamination of Plant Demineralized Water System and Resultant Internal Contamination of Personnel (6/80) | Not Applicable Procurement issue |

1.E CIRCULARS

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 30-15 | Loss of Reactor Coolant Pump Cooling and Natural Circulation Cooldown (6/80) | Not Applicable Procedural/Training issue |
| 80-16 | Operational Deficiencies in Rosemount Model 510DU Trip Units and Model 1152 Pressure Transmitters (6/80) | Not Applicable Procurement issue |
| 80-17 | Fuel Pin Damage Due to Water Jet from Baffle Plate Corner (7/80) | Not Applicable AP600 design does not have this configuration |
| 80-18 | 10 CFR 50.59 Safety Evaluation for Changes to Radioactive Waste Treatment Systems (8/80) | Not Applicable Part of COL |
| 80-19 | Noncompliance with License Requirements for Medical Licensees (8/80) | Not applicable to commercial nuclear power plants |
| 80-20 | Changes in Safe-Slab Tank Dimensions (8/80) | Not applicable to commercial nuclear power plants |
| 80-21 | Regulation of Refueling Crews (9/80) | Not Applicable Administrative communication |
| 80-22 | Confirmation of Employee Qualification (10/80) | Not Applicable Administrative communication |
| 80-23 | Potential Defects in Beloit Power Systems Emergency Generators (10/80) | Not Applicable Procurement issue |
| 80-24 | AECL Teletherapy Unit Malfunctions (12/80) | Not applicable to commercial nuclear power plants |
| 80-25 | Case Histories of Radiography Events (12/80) | Not applicable to commercial nuclear power plants |
| 81-01 | Design Problems Involving Indicating Pushbutton Switches Manufactured by Honeywell Incorporated (1/81) | Not Applicable Procurement issue |
| 81-02 | Performance of NRC-Licensed Individuals While on Duty (2/81) | Not Applicable Administrative communication |
| 81-03 | Inoperable Seismic Monitoring Instrumentation (3/81) | Not Applicable Maintenance issue |

1.E CIRCULARS

| NUMBER | TITLE | COMMENT |
|--------|--|--|
| 81-04 | The Role of Shift Technical Advisors and Importance of Reporting Operational Events (4/81) | Not Applicable Administrative communication |
| 81-05 | Self-Aligning Rod End Bushing for Pipe Supports (3/81) | Not Applicable Procurement issue |
| 81-06 | Potential Deficiency Affecting Certain Foxboro 10 to 50 Milliampere Transmitters (4/81) | Not Applicable Procurement issue |
| 81-07 | Control of Radioactively Contaminated Material (5/81) | Not Applicable Administrative/Training issue |
| 81-08 | Foundation Materials (5/81) | Not Applicable Procurement/Installation issue |
| 81-09 | Containment Effluent Water That Bypasses Radioactivity Monitor (7/81) | SSAR Section 6.2.3 |
| 81-10 | Steam Voiding in the Reactor Coolant System During Decay Heat Removal Cooldown (7/81) | Not Applicable Procedural issue |
| 81-11 | Inadequate Decay Heat Removal During Reactor Shutdown (7/81) | Not Applicable BWR only |
| 81-12 | Inadequate Periodic Test Procedures of PWR Protection System (7/81) | SSAR Section 7.1.2.2.4 |
| 81-13 | Torque Switch Electrical Bypass Circuit for Safeguard Service Valve Motors (9/81) | Not Applicable Installation issue |
| 81-14 | Main Steam Isolation Valve Failure to Close (11/81) | SSAR Section 9.3.1 |
| 81-15 | Unnecessary Radiation Exposure to the Public and Workers During Events Involving Thickness and Level Measuring Devices (12/81) | Not Applicable Procedural issue |

INFORMATION NOTICES

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 80-01 | Fuel Handling Events (1/80) | SSAR Section 9.1.4 |
| 80-02 | 8 X 8R Water Rod Lower End Plug Wear (1/80) | Not Applicable BWR only |
| 80-03 | Main Turbine Electrohydraulic Control System (1/80) | Not Applicable Procurement issue |
| 80-04 | BWR Fuel Exposure in Excess of Limits (2/80) | Not Applicable BWR only |
| 80-05 | Chloride Contamination of Safety Related Piping and Components (2/80) | Not Applicable Procurement issue |
| 80-06 | Notification of Significant Events (2/80) | Not Applicable Administrative issue |
| 80-06 S1 | Notification of Significant Events (4/80) | Not Applicable Administrative issue |
| 80-07 | Pump Shaft Fatigue Cracking (2/80) | Not Applicable Procurement issue No safety-related pumps |
| 80-08 | The States Company Sliding Link Electrical Terminal Block (3/80) | Not Applicable Procurement issue |
| 80-09 | Possible Occupational Health Hazard Associated with Closed Cooling Systems for Operating Power Plants (3/80) | Not Applicable Maintenance issue |
| 80-10 | Partial Loss of Non-Nuclear Instrument System Power Supply During Operation (3/80) | Not Applicable Procurement issue |
| 80-11 | Generic Problems with Asco Valves in Nuclear Applications Including Fire Protection Systems (3/80) | Not Applicable Procurement/maintenance issue |
| 80-12 | Instrument Failure Causes Opening of PORV and Block Valve (3/80) | Not Applicable Procurement issue |
| 80-13 | General Electrical Type SBM Control Switches Defective Cam Followers (4/80) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 80-14 | Safety Suggestions from Employees (4/80) | Not Applicable Administrative communication |
| 80-15 | Axial (Longitudinal) Oriented Cracking in Piping (4/80) | Not Applicable Part of COL Surveillance issue |
| 80-16 | Shaft Seal Packing Causes Binding in Main Steam Swing Disc Check and Isolation Valves (4/80) | Not Applicable Procurement/maintenance issue |
| 80-17 | Potential Hazards Associated with Interchangeable Parts on Radiographic Equipment (5/80) | Not Applicable to commercial nuclear power plants |
| 80-18 | Possible Weapons Smuggling Pouch (5/80) | Not Applicable Administrative/Procedural issue |
| 80-19 | NIOSH Recall of Recirculating-Mode (Closed Circuit) Self-Contained Breathing Apparatus (Rebreather) (5/80) | Not Applicable Procurement issue |
| 80-20 | Loss of Decay Heat Removal Capability at Davis-Besse Unit 1 While in a Refueling Mode (5/80) | Not Applicable Procedural/Maintenance issue |
| 80-21 | Anchorage and Support of Safety-Related Electrical Equipment (5/80) | Not Applicable Installation/Procurement issue |
| 80-22 | Breakdowns in Containment Control Programs (5/80) | Not Applicable Procedural issue |
| 80-23 | Loss of Suction to Emergency Feedwater Pumps (5/80) | AP600 design has no safety-related EFW or similar safety-related system |
| 80-24 | Low-Level Radioactive Waste Burial Criteria (5/80) | Not Applicable Administrative communication |
| 80-25 | Transportation of Pyrophoric Uranium (5/80) | Not Applicable Administrative communication |
| 80-26 | Evaluation of Contractor QA Program (6/80) | Not Applicable Administrative communication |
| 80-27 | Degradation of Reactor Coolant Pump Studs (6/80) | Not Applicable Procurement/Maintenance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 80-28 | Prompt Reporting of Information in Accordance with 10 CFR 50.55(e) (6/80) | Not Applicable Administrative communication |
| 80-29 | Broken Studs on Terry Turbine Steam Inlet Flange (8/80) | Not Applicable AP600 design has no Terry Turbine pumps |
| 80-29 S1 | Broken Studs on Terry Turbine Steam Inlet Flange (11/80) | Not Applicable AP600 design has no Terry Turbine pumps |
| 80-30 | Potential for Unacceptable Interaction Between the Control Rod Drive Scram Function and Non-Essential Control Air at Certain GE BWR Facilities (8/80) | Not Applicable BWR only |
| 80-31 | Maloperation of Gould-Brown Boveri 480 Volt - Type K-600S and K-DON 600S Circuit Breakers (8/80) | Not Applicable Procurement issue |
| 80-32 | Clarification of Certain Requirements for Exclusive-Use Shipments of Radioactive Materials (8/80) | Not Applicable Administrative communication |
| 80-32 R1 | Clarification of Certain Requirements for Exclusive-Use Shipments of Radioactive Materials (2/82) | Not Applicable Administrative communication |
| 80-33 | Determination of Teletherapy Timer Accuracy (9/80) | Not Applicable to commercial nuclear power plants |
| 80-34 | Boron Dilution of Reactor Coolant During Steam Generator Decontamination (9/80) | Not Applicable Maintenance/Procedural issue |
| 80-35 | Leaking and Dislodged Iodine-125 Implant Seeds (10/80) | Not applicable to commercial nuclear power plants |
| 80-35 S1 | Leaking and Dislodged Iodine-125 Implant Seeds (10/82) | Not applicable to commercial nuclear power plants |
| 80-36 | Failure of Steam Generator Support Bolting (10/80) | Not Applicable Procurement/Maintenance issue |
| 80-37 | Containment Cooler Leaks and Reactor Cavity Flooding at Indian Point Unit 2 (10/80) | Not Applicable Part of COL Surveillance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 80-38 | Cracking in Charging Pump Casing Cladding (10/80) | Not Applicable Procurement/Maintenance issue |
| 80-39 | Malfunction of Solenoid Valves Manufactured By Valcor Engineering Corporation (10/80) | Not Applicable Procurement issue |
| 80-40 | Excessive Nitrogen Supply Pressure Actuates Safety-Relief Valve Operation to Cause Reactor Depressurization (11/80) | Not Applicable Procurement/Installation issue |
| 80-41 | Failure of Swing Check Valve in the Decay Heat Removal System at Davis-Besse Unit No. 1 (11/80) | Not Applicable Procurement/Maintenance issue |
| 80-42 | Effect of Radiation on Hydraulic Snubber Fluid (11/80) | Not Applicable Procurement issue |
| 80-43 | Failures of the Continuous Water Level Monitor for the Scram Discharge Volume at the Dresden Unit No. 2 (12/80) | Not Applicable BWR only |
| 80-44 | Actuation of the ECCS in the Recirculation Mode While in Hot Shutdown (12/80) | Not Applicable AP600 design has no safety-related pumps |
| 80-45 | Potential Failures of BWR Backup Manual Scram Capability (12/80) | Not Applicable BWR only |
| 81-01 | Possible Failures of General Electric Type HFA Relays (1/81) | Not Applicable Procurement issue |
| 81-02 | Transportation of Radiography Devices (1/81) | Not applicable to commercial nuclear power plants |
| 81-03 | Checklist for Licensees Making Notifications of Significant Events in Accordance with 10 CFR 50.72 (2/81) | Not Applicable Administrative communication |
| 81-04 | Cracking in Main Steam Lines (2/81) | Not Applicable Part of COL Surveillance issue |
| 81-05 | Degraded DC System at Palisades (3/81) | Not Applicable Maintenance/Personnel issue |
| 81-06 | Failure of ITE Model K-600 Circuit Breaker (3/81) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|--|
| 81-07 | Potential Problem with Water-Soluble Purge Dam Materials Used During Inert Gas Welding (3/81) | Not Applicable Procedural issue |
| 81-08 | Repetitive Failures of Limitorque Operator SMB-4 Motor-To-Shaft Key (3/81) | Not Applicable Procurement issue |
| 81-09 | Degradation of Residual Heat Removal (RHR) System (3/81) | SSAR Section 16.2 |
| 81-10 | Inadvertent Containment Spray Due to Personnel Error (3/81) | Not Applicable AP600 design has no containment spray system |
| 81-11 | Alternate Rod Insertion For BWR Scram Represents a Potential Path For Loss of Primary Coolant (3/81) | Not Applicable BWR only |
| 81-12 | Guidance on Order Issued January 9, 1981 Regarding Automatic Control Rod Insertion On Low Control Air Pressure (3/81) | Not Applicable BWR only |
| 81-13 | Jammed Source Rack in a Gamma Irradiator (4/81) | Not applicable to commercial nuclear power plants |
| 81-14 | Potential Overstress of Shafts on Fisher Series 9200 Butterfly Valve with Expandable T Rings (4/81) | Not Applicable Procurement issue |
| 81-15 | Degradation of Automatic ECCS Actuation Capability By Isolation of Instrument Lines (4/81) | Not Applicable Surveillance issue SSAR Section 16.1 |
| 81-16 | Control Rod Drive System Malfunctions (4/81) | Not Applicable BWR only |
| 81-17 | No Title (no date) | Not Issued |
| 81-18 | Excessive Radiation to the Fingers of Three Individuals Incurred During Cleaning and Wipe Testing of Radioactive Sealed Sources at a Sealed-source Manufacturing Facility (6/81) | Not applicable to commercial nuclear power plants |
| 81-19 | Lost Parts in Primary Coolant System (7/81) | Not Applicable Part of COL Surveillance/Procedural issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 81-20 | Test Failures of Electrical Penetration Assembly (7/81) | Not Applicable Procurement issue |
| 81-21 | Potential Loss of Direct Access to Ultimate Heat Sink (7/81) | Not Applicable BWR only |
| 81-22 | Section 235 and 236 Amendments to the Atomic Energy Act of 1954 (7/81) | Not Applicable Administrative issue |
| 81-23 | Fuel Assembly Damaged Due to Improper Positioning of Handling Equipment (8/81) | Not Applicable Procedural issue |
| 81-24 | Auxiliary Feed Pump Turbine Bearing Failure (8/81) | SSAR Section 16.2 |
| 81-25 | Open Equalizing of Differential Pressure Transmitter Causes Reactor Scram and Loss of Redundant Safety Signals (8/81) | Not Applicable Procedural issue |
| 81-26 | Compilation of Health Physics Related Information Items (9/81) | Not Applicable Procedural issue |
| 81-27 | Flammable Gas Mixtures in the Waste Gas Decay Tanks in PWR Plants (9/81) | Not Applicable AP600 design has no gas decay tanks |
| 81-28 | Failure of Rockwell-Edward Main Steam Isolation Valves (9/81) | Not Applicable Procurement issue |
| 81-29 | Equipment Qualification Testing Experience (9/81) | Not Applicable Procurement issue |
| 81-30 | Velan Swing Check Valves (9/81) | Not Applicable Procurement issue |
| 81-31 | Failure of Safety Injection Valves to Open Against Differential Pressure (10/81) | Not Applicable SSAR Section 3.9.6.2 Procurement/Surveillance issue |
| 81-32 | Transfer and/or Disposal of Spent Generators (10/81) | Not applicable to commercial nuclear power plants |
| 81-33 | Locking Devices Inadequately Installed on Main Steam Isolation Valves (11/81) | Not Applicable Procurement issue |
| 81-34 | Accidental Actuation of Prompt Public Notification System (11/81) | Not Applicable Procedural issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 81-35 | Check Valve Failures (12/81) | Not Applicable Procurement issue |
| 81-36 | Replacement Diaphragms For Robertshaw Valve (Model No. VC-210) (12/81) | Not Applicable Procurement issue |
| 81-37 | Unnecessary Radiation Exposure to the Public and Workers During Events Involving Thickness and Level Measuring Devices (12/81) | Not Applicable Procedural issue |
| 81-38 | Potential Significant Equipment Failures Resulting From Contamination of Air-operated Systems (12/81) | Not Applicable Part of COL Surveillance issue |
| 81-39 | EPA Crosscheck Program - Low Level Radioiodine in Water Intercomparison Study (12/81) | Not Applicable Administrative communication |
| 82-01 | Auxiliary Feedwater Pump Lockout Resulting From Westinghouse W-2 Switch Circuit Modification (1/82) | Not Applicable Procurement issue |
| 82-01 R1 | Auxiliary Feedwater Pump Lockout Resulting From Westinghouse W-2 Switch Circuit Modification (2/82) | Not Applicable Procurement issue |
| 82-02 | Westinghouse NBFD Relay Failures in Reactor Protection Systems at Certain Nuclear Power Plants (1/82) | Not Applicable Procurement issue |
| 82-03 | Environmental Tests of Electrical Terminal Blocks (3/82) | Not Applicable Maintenance issue |
| 82-04 | Potential Deficiencies of Certain Agastat E-7000 Series Time Delay Relays (3/82) | Not Applicable Procurement issue |
| 82-05 | Increasing Frequency of Drug-Related Incidents (3/82) | Not Applicable Administrative communication |
| 82-06 | Failure of Steam Generator Primary Side Manway Closure Studs (3/82) | Not Applicable Maintenance issue |
| 82-07 | Inadequate Security Screening Programs (3/82) | Not Applicable Administrative communication |
| 82-08 | Check Valve Failures on Diesel Generator Engine Cooling System (3/82) | SSAR Section 16.2 |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 82-09 | Cracking of Piping of Makeup Coolant Lines at B&W Plants (3/82) | Not Applicable Part of COL Surveillance issue |
| 82-10 | Following Up Symptomatic Repairs to Assure Resolution of the Problem (3/82) | Not Applicable Administrative communication |
| 82-11 | Potential Inaccuracies in Wide Range Pressure Instruments Used in Westinghouse Designed Plants (4/82) | SSAR Sections 3.11, Appendix 3D |
| 82-12 | Surveillance of Hydraulic Snubbers (4/82) | Not Applicable Part of COL Surveillance/Procurement issue |
| 82-13 | Failures of General Electric Type HFA Relays (5/82) | Not Applicable Procurement issue |
| 82-14 | TMI-1 Steam Generator/Reactor Coolant System Chemistry/Corrosion Problem (5/82) | Not Applicable Part of COL Surveillance/Procedural issue |
| 82-15 | Notification of the Nuclear Regulatory Commission (NRC) (5/82) | Not Applicable Administrative communication |
| 82-16 | HPCI/RCIC High Steam Flow Setpoints (5/82) | Not Applicable Procedural issue |
| 82-17 | Overpressurization of Reactor Coolant System (6/82) | SSAR Section 3.9.1.1.2.6 |
| 82-18 | Assessment of Intakes of Radioactive Material by Workers (6/82) | Not Applicable Administrative communication |
| 82-19 | Loss of High Head Safety Injection Emergency Boration and Reactor Coolant Makeup Capability (6/82) | Not Applicable AP600 design has no safety-related pumps |
| 82-20 | Check Valve Problems (6/82) | Not Applicable Procurement/Maintenance issue |
| 82-21 | Buildup of Enriched Uranium in Effluent Treatment Tanks (6/82) | Not applicable to commercial nuclear power plants |
| 82-22 | Failures in Turbine Exhaust Lines (7/82) | Not Applicable Part of COL Surveillance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 82-23 | Main Steam Isolation Valve (MSIV) Leakage (7/82) | Not Applicable BWR only |
| 82-24 | Water Leaking From Uranium Hexafluoride Overpacks (7/82) | Not applicable commercial nuclear power plants |
| 82-25 | Failures of Hiller Actuators Upon Gradual Loss of Air Pressure (7/82) | Not Applicable Procurement issue |
| 82-26 | RCIC and HPCI Turbine Exhaust Check Valve Failures (7/82) | Not Applicable BWR only |
| 82-27 | Fuel Rod Degradation Resulting from Baffle Water Jet Impingement (8/82) | Not Applicable Plant specific issue |
| 82-28 | Hydrogen Explosion While Grinding in the Vicinity of Drained and Open Reactor Coolant System (7/82) | Not Applicable Procedural issue |
| 82-29 | Control Rod Drive (CRD) Guide Tube Support Pin Failures at Westinghouse PWRs (7/82) | Not Applicable Isolated issue |
| 82-30 | Loss of Thermal Sleeves in Reactor Coolant System Piping at Certain Westinghouse PWR Power Plants (7/82) | Not Applicable AP600 design has no thermal sleeves |
| 82-31 | Overexposure of Diver During Work in Fuel Storage Pool (7/82) | Not Applicable Procedural issue |
| 82-32 | Contamination of Reactor Coolant System by Organic Cleaning Solvents (8/82) | Not Applicable Procedural issue |
| 82-33 | Control of Radiation Levels in Unrestricted Areas Adjacent to Brachytherapy Patients (8/82) | Not applicable to commercial nuclear power plants |
| 82-34 | Welds in Main Control Panel (8/82) | Not Applicable Procurement issue |
| 82-34 R1 | Welds in Main Control Panel (9/82) | Not Applicable Procurement issue |
| 82-35 | Failure of Three Check Valves on High Pressure Injection Lines to Pass Flow (8/82) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 82-36 | Respirator Users Warning for Certain 5-Minute Emergency Escape Self-Contained Breathing Apparatus (9/82) | Not Applicable Procurement issue |
| 82-37 | Cracking in the Upper Shell to Transition Cone Girth Weld of a Steam Generator at a Operating Pressurized Water Reactor (9/82) | Not Applicable Surveillance issue weld relocated |
| 82-38 | Changes in Format and Distribution Systems for IE Bulletins, Circulars, and Information Notices (9/82) | Not Applicable Administrative communication |
| 82-39 | Service Degradation of Thick Wall Stainless Steel Recirculation System Piping at a BWR Plant (9/82) | Not Applicable BWR only |
| 82-40 | Deficiencies in Primary Containment Electrical Penetration Assemblies (9/82) | Not Applicable Procurement issue |
| 82-41 | Failure of Safety/Relief Valves to Open at a BWR (10/82) | Not Applicable Procurement issue |
| 82-42 | Defects Observed in Panasonic Model 801 and Model 802 Thermoluminescent Dosimeters (11/82) | Not Applicable Procurement issue |
| 82-43 | Deficiencies in LWR Air Filtration/Ventilation Systems (11/82) | Not Applicable Part of COL Maintenance/Surveillance issue |
| 82-44 | Clarification of Emergency Plan Exercise Requirements (11/82) | Not Applicable Administrative communication |
| 82-45 | PWR Low Temperature Overpressure Protection (11/82) | SSAR Section 3.9.1.1.2.6 |
| 82-46 | Defective and Obsolete Padlocks (11/82) | Not Applicable Procurement issue |
| 82-47 | Transportation of Type A Quantities of Non-Fissile Radioactive Material (11/82) | Not Applicable Administrative communication |
| 82-48 | Failures of Agastat CR 0095 Relay Sockets (12/82) | Not Applicable Procurement issue |
| 82-49 | Correction for Sample Conditions for Air and Gas Monitoring (12/82) | SSAR Sections 11.5.2 and 11.5.3 |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 82-50 | Modification of Solid State AC Undervoltage Relays Type ITE-27 (12/82) | Not Applicable Procurement issue |
| 82-51 | Overexposures in PWR Cavities (12/82) | Not Applicable Procedural issue |
| 82-52 | Equipment Environmental Qualification Testing Experience - Updating of Test Summaries Previously Published in IN-29 (12/82) | Not Applicable Procurement issue |
| 82-53 | Main Transformer Failures at the North Anna Nuclear Power Station (12/82) | Not Applicable Procurement/Maintenance issue |
| 82-54 | Westinghouse NBFD Relay Failures in Reactor Protection Systems (12/82) | Not Applicable Procurement issue |
| 82-55 | Seismic Qualification of Westinghouse AR Relay with Latch Attachments Used in Westinghouse Solid State Protection System (12/82) | Not Applicable Procurement issue |
| 82-56 | Robertshaw Thermostatic Flow Control Valves (12/82) | Not Applicable Procurement issue |
| 83-01 | Ray Miller, Inc. (1/83) | Not Applicable Procurement issue |
| 83-01 S1 | Ray Miller, Inc. (4/83) | Not Applicable Procurement issue |
| 83-02 | Limitorque H0BC, H1BC, H2BC and H3BC Gearheads (1/83) | Not Applicable Procurement issue |
| 83-03 | Calibration of Liquid Level Instruments (1/83) | Not Applicable Procedural/Training issue |
| 83-04 | Failure of ELMA Power Supply Units (1/83) | Not Applicable Procurement issue |
| 83-05 | Obtaining Approval for Disposing of Very-Low-Level Radioactive Waste - 10 CFR Section 20.302 (2/83) | Not Applicable Administrative communication |
| 83-06 | Nonidentical Replacement Parts (2/83) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 83-07 | Nonconformities with Materials Supplied by Tube Line Corporation (3/83) | Not Applicable Procurement issue |
| 83-08 | Component Failures Caused by Elevated DC Control Voltage (3/83) | Not Applicable Procurement issue |
| 83-09 | Safety and Security of Irradiators (3/83) | Not applicable to commercial nuclear power plants |
| 83-10 | Clarification of Several Aspects Relating to Use of NRC-Certified Transport Packages (3/83) | Not Applicable Administrative communication |
| 83-11 | Possible Seismic Vulnerability of Old Lead Storage Batteries (3/83) | SSAR Sections 3.10, 3.11, Appendix 3D |
| 83-12 | Incorrect Boron Standards (3/83) | Not Applicable Procurement issue |
| 83-13 | Design Misapplication of Bergen-Patterson Standard Strut Restraint Clamp (3/83) | Not Applicable Procurement issue |
| 83-14 | Dewatered Spent Ion Exchange Resin Suscepability to Exothermic Chemical Reaction (3/83) | Not Applicable Procedural issue |
| 83-15 | Falsified Pre-Employment Screening Records (3/83) | Not Applicable Administrative communication |
| 83-16 | Contamination of the Auburn Steel Company Property with Cobalt-60 (3/83) | Not applicable to commercial nuclear power plants |
| 83-17 | Electrical Control Logic Problem Resulting in Inoperable Auto-Start of Emergency Diesel Generator Units (3/83) | SSAR Section 8.3.1.1.2.1 |
| 83-18 | Failures of the Undervoltage Trip Function of Reactor Trip System Breakers (4/83) | Not Applicable Procurement/Maintenance issue |
| 83-19 | General Electric Type HFA Relay Contact Gap and Wipe Setting Adjustments (4/83) | Not Applicable Procurement/Maintenance issue |
| 83-20 | ITT Grinnell Figure 306/307 Mechanical Snubber Attachment Interference (4/83) | Not Applicable Procurement issue |
| 83-21 | Defective Emergency-Use Respirator (4/83) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 83-22 | Boiling Water Reactor Safety/Relief Valve Failures (4/83) | Not Applicable Procurement issue |
| 83-23 | Inoperable Containment Atmosphere Sensing Systems (4/83) | Not Applicable Procedural issue |
| 83-24 | Loose Parts in the Secondary Side of Steam Generators at Pressurized Water Reactors (4/83) | Not Applicable Part of COL Installation/Surveillance issue |
| 83-25 | Standby Gas Treatment System Heater High Temperature Trip Setpoint Adjustment (4/83) | Not Applicable Procurement issue |
| 83-26 | Failure of Safety/Relief Valve Discharge Line Vacuum Breakers (5/83) | Not Applicable BWR only |
| 83-27 | Operational Response to Events Concerning Deliberate Acts Directed Against Plant Equipment (5/83) | Not Applicable Procedural issue |
| 83-28 | Criteria for Protective Action Recommendations for General Emergencies (5/83) | Not Applicable Part of COL |
| 83-29 | Fuel Binding Caused by Fuel Rack Deformation (5/83) | SSAR Section 9.1.2.2.1 |
| 83-30 | Misapplication of Generic Emergency Operating Procedures (EOP) Guidelines (5/83) | SSAR Section 18.9.8.1.1.3 |
| 83-31 | Error in the ADLPIPE Computer Program (5/83) | Not Applicable Procurement issue |
| 83-32 | Rupture of Americium-241 Source(s) Contained in a Well-logging Device (5/83) | Not applicable to commercial nuclear power plants |
| 83-33 | Nonrepresentative Sampling of Contaminated Oil (5/83) | Not Applicable Procedural issue |
| 83-34 | Event Notification Information Worksheet (5/83) | Not Applicable Administrative communication |
| 83-35 | Fuel Movement with Control Rods Withdrawn at BWRs (5/83) | Not Applicable BWR only |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 83-36 | Impact of Security Practices on Safe Operations (6/83) | SSAR Section 13.6.3.2 |
| 83-37 | Transformer Failure Resulting from Degraded Internal Connection Cables (6/83) | Not Applicable Installation issue |
| 83-38 | Defective Heat Sink Adhesive and Seismically Induced Chatter in Relays within Printed Circuit Cards (6/83) | Not Applicable Procurement issue |
| 83-39 | Failure of Safety/Relief Valves to Open at BWR - Interim Report (6/83) | Not Applicable Procurement issue |
| 83-40 | Need to Environmentally Qualify Epoxy Grouts and Sealers (6/83) | Not Applicable Procurement issue |
| 83-41 | Actuation of Fire Suppression System Causing Inoperability of Safety Related Equipment (6/83) | SSAR Section 9.5.1.3 |
| 83-42 | Reactor Mode Switch Malfunctions (6/83) | Not Applicable BWR only |
| 83-43 | Improper Settings of Intermediate Range (IR) High Flux Trip Setpoints (6/83) | Not Applicable Procedural issue |
| 83-44 | Potential damage to Redundant Safety Equipment as a result of Backflow Through the Equipment and Floor Drain System (7/83) | SSAR Section 9.3.5 |
| 83-44 S1 | Potential damage to Redundant Safety Equipment as a result of Backflow Through the Equipment and Floor Drain System (8/90) | SSAR Section 9.3.5 |
| 83-45 | Environmental Qualification Test of General Electric Company "CR-2940" Position Selector Control Switch (7/83) | Not Applicable Procurement issue |
| 83-46 | Common Mode Valve Failures Degrade Surry's Recirculation Spray Subsystem (7/83) | Not Applicable AP600 design has no recirculation spray system |
| 83-47 | Failure of Hydraulic Snubbers as a Result of Contaminated Hydraulic Fluid (7/83) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 83-48 | Gaseous Effluent Releases of Radioactive Iodine-125 and Iodine-131 in Excess of Nuclear Regulatory Commission Limits (7/83) | Not applicable to commercial nuclear power plants |
| 83-49 | Sampling and Prevention of Intrusion of Organic Chemicals into Reactor Coolant Systems (7/83) | SSAR Section 9.3.3.2 |
| 83-50 | Failures of Class 1E Safety-Related Switchgear Circuit Breakers to Close on Demand (8/83) | SSAR Section 16.2 |
| 83-51 | Diesel Generator Events (8/83) | SSAR Section 16.2 |
| 83-52 | Radioactive Waste Gas System Events (8/83) | Not Applicable Procurement/Maintenance issue |
| 83-53 | Primary Containment Isolation Valve Discrepancies (8/83) | Not Applicable BWR only |
| 83-54 | Common Mode Failure of Main Steam Isolation Nonreturn Valves (8/83) | Not Applicable AP600 design has no nonreturn valves |
| 83-55 | Misapplication of Valves by Throttling Beyond Design Basis (8/83) | Not Applicable B&W and CE plants only |
| 83-56 | Operability of Required Auxiliary Equipment (8/83) | Not Applicable Plant specific issue |
| 83-57 | Potential Misassembly Problem with Automatic Switch Company (ASCO) Solenoid Valve Model NP 8316 (8/83) | Not Applicable Procurement issue |
| 83-58 | Transamerica Delaval Diesel Generator Crankshaft Failure (8/83) | SSAR Section 16.2 |
| 83-59 | Dose Assignment for Workers in Non-Uniform Radiation Fields (9/83) | Not Applicable Administrative communication |
| 83-60 | Falsification of Test Results for Protective Coatings (9/83) | Not Applicable Procurement issue |
| 83-61 | Alleged Use of Stand-Ins for Welder Qualification Tests (9/83) | Not Applicable Administrative communication |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 83-62 | Failure of Redundant Toxic Gas Detectors Positioned at Control Room Ventilation Air Intakes (9/83) | Not Applicable Part of COL |
| 83-63 | Potential Failures of Westinghouse Electric Corporation Type SA-1 Differential Relays (9/83) | Not Applicable Procurement issue |
| 83-63 S1 | Potential Failures of Westinghouse Electric Corporation Type SA-1 Differential Relays (2/84) | Not Applicable Procurement issue |
| 83-64 | Lead Shielding Attached to Safety-Related Systems Without 10 CFR 50.59 Evaluations (9/83) | Not Applicable Administrative communication |
| 83-65 | Surveillance of Flow in RTD Bypass Loops Used in Westinghouse Plants (10/83) | Not Applicable AP600 design has no bypass loops |
| 83-66 | Fatality at Argentine Critical Facility (10/83) | Not applicable to commercial nuclear power plants |
| 83-66 S1 | Fatality at Argentine Critical Facility (5/84) | Not applicable to commercial nuclear power plants |
| 83-67 | Emergency-Use Respirator Material Defect Causes Production of Noxious Gases (10/83) | Not Applicable Procurement issue |
| 83-68 | Respirator User Warning - Defective Self-Contained Breathing Apparatus Air Cylinders (10/83) | Not Applicable Procurement issue |
| 83-69 | Improperly Installed Fire Dampers at Nuclear Power Plants (10/83) | Not Applicable Installation issue |
| 83-70 | Vibration-Induced Valve Failures (10/83) | Not Applicable Procurement/Maintenance issue |
| 83-70 S1 | Vibration-Induced Valve Failures (3/85) | Not Applicable Procurement/Maintenance issue |
| 83-71 | Defects in Load-Bearing Welds on Lifting Device for Vessel Head and Internals (10/83) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 83-72 | Environmental Qualification Testing Experience (10/83) | Not Applicable Specific EQ test data Procurement issue |
| 83-73 | Radiation Exposure from Gloves Contaminated with Uranium Daughter Products (10/83) | Not applicable to commercial nuclear power plants |
| 83-74 | Rupture of Cesium-137 Sources Used in Well-Logging Operations (11/83) | Not applicable to commercial nuclear power plants |
| 83-75 | Improper Control Rod Manipulations (11/83) | Not Applicable Procedures/Training issue |
| 83-76 | Reactor Trip Breaker Malfunctions (Undervoltage Trip Devices on GE Type AK-2-25 Breakers) (11/83) | Not Applicable Procurement issue |
| 83-77 | Air/Gas Entrainment Events Resulting in System Failures (11/83) | Not Applicable AP600 design has no safety-related pumps |
| 83-78 | Apparent Improper Modification of a Component Affecting Plant Safety (11/83) | Not Applicable Procedural issue |
| 83-79 | Apparently Improper Use of Commercial Grade Components in Safety Related Systems (11/83) | Not Applicable Procurement issue |
| 83-80 | Use of Specialized "Stiff" Pipe Clamps (11/83) | Not Applicable Installation issue |
| 83-81 | Entry Into High Radiation Areas Which Are Not Under Direct Surveillance (12/83) | Not Applicable Administrative communication |
| 83-82 | Failure of Safety/Relief Valves to Open at BWR - Final Report (12/83) | Not Applicable Procurement/Maintenance issue |
| 83-83 | Use of Portable Radio Transmitters Inside Nuclear Power Plants (12/83) | Not Applicable Procurement issue |
| 83-84 | Cracked and Broken Piston Rods in Brown Boveri Electric Type 5HK Breakers (12/83) | Not Applicable Procurement issue |
| 84-01 | Excess Lubricant in Electric Cable Sheaths (1/84) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 84-02 | Operating a Nuclear Power Plant at Voltage Levels Lower Than Analyzed (1/84) | Not Applicable AP600 design has no safety-related AC power |
| 84-03 | Compliance with Conditions of Licensee and Notification of Disability by Licensed Operators (1/84) | Not Applicable Administrative communication |
| 84-04 | Failure of Elastomer Seated Butterfly Valves Used Only During Cold Shutdowns (1/84) | Not Applicable Procurement/Maintenance issue |
| 84-05 | Exercise Frequency (1/84) | Not Applicable Administrative communication |
| 84-05 R1 | Exercise Frequency (3/84) | Not Applicable Administrative communication |
| 84-06 | Steam Binding of Auxiliary Feedwater Pumps (1/84) | Not Applicable AP600 design has no safety-related AFW or similar safety-related system |
| 84-07 | Design-Basis Threat and Review of Vehicular Access Controls (2/84) | Not Applicable Administrative communication |
| 84-08 | 10 CFR 50.7, "Employee Protection" (2/84) | Not Applicable Administrative communication |
| 84-09 | Lessons Learned from NRC Inspections of Fire Protection Safe Shutdown Systems (10 CFR 50, Appendix R) (2/84) | SSAR Section 9.5.1 |
| 84-09 R1 | Lessons Learned from NRC Inspections of Fire Protection Safe Shutdown Systems (10 CFR 50, Appendix R) (3/84) | Not Applicable Administrative communication |
| 84-10 | Motor Operated Valve Torque Switch Set Below the Manufacturer's Recommended Value (2/84) | Not Applicable Procedure issue |
| 84-11 | Training Program Deficiencies (2/84) | Not Applicable Administrative communication |
| 84-12 | Failure of Soft Seat Valve Seals (2/84) | Not Applicable Procurement/Installation issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 84-13 | Potential Deficiencies in Motor Operated Valve Control Circuits and Annunciation (2/84) | SSAR Sections 8.3.2.1.1.1 and Appendix 1A |
| 84-14 | Highlights of Recent Transport Regulatory Revisions by DOT and NRC (3/84) | Not Applicable Administrative communication |
| 84-15 | Reporting of Radiological Releases (3/84) | Not Applicable Administrative communication |
| 84-16 | Failure of Automatic Sprinkler System Valves to Operate (3/84) | Not Applicable Procurement issue |
| 84-17 | Problems with Liquid Nitrogen Cooling Components Below the Nil Ductility Temperature (3/84) | SSAR Section 9.3.2.2.2 |
| 84-18 | Stress Corrosion Cracking in Pressurized Water Reactor Systems (3/84) | SSAR Section 5.4.2.4.3 |
| 84-19 | Two Events Involving Unauthorized Entries into Reactor Cavities (3/84) | Not Applicable Administrative communication |
| 84-20 | Service Life of Relays in Safety-Related Systems (3/84) | Not Applicable Procurement issue |
| 84-21 | Inadequate Shutdown Margin (3/84) | Not applicable to commercial nuclear power plants |
| 84-22 | Deficiency in Comsip, Inc. Standard Bell Catalyst (3/84) | Not Applicable Procurement issue |
| 84-23 | Results of the NRC-Sponsored Qualification Methodology Research Test on ASCO Solenoid Valves (4/84) | Not Applicable Procurement issue |
| 84-24 | Physical Qualification of Individuals To Use Respiratory Protective Devices (4/84) | Not Applicable Part of COL |
| 84-25 | Recent Serious Violations of NRC Requirements by Radiography Licensees (4/84) | Not applicable to commercial nuclear power plants |
| 84-26 | Recent Serious Violations of NRC Requirements by Moisture Density Gauge Licensees (4/84) | Not applicable to commercial nuclear power plants |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 84-27 | Recent Serious Violations of NRC Requirements by Medical Licensees (4/84) | Not applicable to commercial nuclear power plants |
| 84-28 | Recent Serious Violations of NRC Requirements by Well-Logging Licensees (4/84) | Not applicable to commercial nuclear power plants |
| 84-29 | General Electric Magne-Blast Circuit Breakers Problems (4/84) | Not Applicable Procurement/Maintenance issue |
| 84-30 | Discrepancies in Record Keeping and Material Defects in Bahnson Heating, Ventilation, and Air Conditioning Units (4/84) | Not Applicable Procurement issue |
| 84-31 | Increased Stroking time of Bettis Actuators Because of Swollen Ethylene-Propylene Rubber Seals and Seal Set (4/84) | Not Applicable Procurement/Maintenance issue |
| 84-32 | Auxiliary Feedwater Sparger and Pipe Hanger Damage (4/84) | USI A-1 SSAR Section 1.9.4 |
| 84-33 | Main Steam Safety Valve Failures Caused by Failed Cotter Pins (4/84) | Not Applicable Procurement/Maintenance issue |
| 84-34 | Respirator User Warning: Defective Self-Contained Breathing Apparatus Air Cylinders (4/84) | Not Applicable Procurement issue |
| 84-35 | BWR Post Scram Drywell Pressurization (4/84) | Not Applicable BWR only |
| 84-36 | Loosening of Locking Nut on Limitorque Operator (5/84) | Not Applicable Procurement/Maintenance issue |
| 84-36 S1 | Loosening of Locking Nut on Limitorque Operator (9/84) | Not Applicable Procurement/Maintenance issue |
| 84-37 | Use of Lifted Leads and Jumpers During Maintenance or Surveillance Testing (5/84) | SSAR Sections 7.1.4.2.10, Appendix 1A |
| 84-38 | Problems with Design, Maintenance, and Operation of Offsite Power Systems (5/84) | Not Applicable Part of COL |
| 84-39 | Inadvertent Isolation of Containment Spray Systems (5/84) | Not Applicable Procedural issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 84-40 | Emergency Worker Doses (5/84) | Not Applicable Part of COL |
| 84-41 | IGSCC in BWR Plants (6/84) | Not Applicable BWR only |
| 84-42 | Equipment Availability for Conditions During Outages Not Covered by Technical Specifications (6/84) | SSAR Section 16.2 |
| 84-43 | Storage and Handling of Ophthalmic Beta Radiation Applicators (6/84) | Not applicable to commercial nuclear power plants |
| 84-44 | Environmental Qualification of Rockbestos Cables (6/84) | Not Applicable Procurement issue |
| 84-45 | Reversed Differential Pressure Instrument Sensing Lines (6/84) | Not Applicable Installation issue |
| 84-46 | Circuit Breaker Position Verification (6/84) | Not Applicable Procedural/Training issue |
| 84-47 | Environmental Qualification Tests of Electrical terminal Blocks (6/84) | SSAR Section 3.11, Appendix 3D |
| 84-48 | Failures of Rockwell International Globe Valves (6/84) | Not Applicable Procurement/Maintenance issue |
| 84-48 S1 | Failures of Rockwell International Globe Valves (6/84) | Not Applicable Procurement/Maintenance issue |
| 84-49 | Intergranular Stress Corrosion Cracking Leading to Steam Generator Tube Failure (6/84) | SSAR Section 5.4.2.4.3 |
| 84-50 | Clarification of Scope of Quality Assurance Programs for Transport Packages Pursuant to 10 CFR 50, Appendix B (6/84) | Not Applicable Administrative issue |
| 84-51 | Independent Verification (6/84) | Not Applicable Administrative issue |
| 84-52 | Inadequate Material Procurement Controls on the Part of Licensees and Vendors (6/84) | Not Applicable Administrative/Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 84-52 S1 | Inadequate Material Procurement Controls on the Part of Licensees and Vendors (5/85) | Not Applicable Administrative/Procurement issue |
| 84-53 | Information Concerning the Use of Loctite 242 and Other Anaerobic Adhesives/Sealants (7/84) | Not Applicable Procurement issue |
| 84-54 | Deficiencies in Design Base Documentation and Calculations Supporting Nuclear Power Plant Design (7/84) | Not Applicable Administrative communication |
| 84-55 | Seal Tables Leaks at PWRs (7/84) | Not Applicable Installation/Maintenance issue |
| 84-55 S1 | Seal Tables Leaks at PWRs (5/85) | Not Applicable Installation/Maintenance issue |
| 84-56 | Respirator Users Notice for Certain 5-Minute Emergency Escape Self-Contained Breathing Apparatus (7/84) | Not Applicable Procurement issue |
| 84-57 | Operating Experience Related to Moisture Intrusion in Safety-Related Electrical Equipment at Commercial power Plants (7/84) | SSAR Section 3.11, Appendix 3D |
| 84-58 | Inadvertent Defeat of safety Function Caused by Human Error Involving Wrong Unit, Wrong Train, or Wrong System (7/84) | Not Applicable Administrative communication |
| 84-59 | Deliberate Circumventing of Station Heat Physics Procedures (8/84) | Not Applicable Administrative communication |
| 84-60 | Failure of Air Purifying Respirator Filters To Meet Efficiency Requirements (8/84) | Not Applicable Procurement issue |
| 84-61 | Overexposure of Diver in Pressurized Water reactor (PWR) Refueling Cavity (8/84) | Not Applicable Administrative communication |
| 84-62 | Therapy Misadministrations To Patients Undergoing Cobalt-60 Teletherapy Treatments (8/84) | Not applicable to nuclear power plants |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 84-63 | Defective RHR Replacement Piping (8/84) | Not Applicable Procurement issue |
| 84-64 | BWR High-Pressure Coolant Injection (HPCI) Initiation Seal-In and Indication (8/84) | Not Applicable BWR only |
| 84-65 | Underrated Fuses Which May Adversely Affect Operation of Essential Electrical Equipment (8/84) | Not Applicable Procurement issue |
| 84-66 | Undetected Unavailability of the Turbine-Driven Auxiliary Feedwater Train (8/84) | Not Applicable AP600 design has no turbine driven pumps |
| 84-67 | Recent Snubber Inservice Testing with High Failure Rates (8/84) | Inservice Testing Program |
| 84-68 | Potential Deficiencies in Improperly Rated Field Wiring to Solenoid Valves ((8/84) | Not Applicable Installation issue |
| 84-69 | Operation of Emergency Diesel Generators (8/84) | SSAR Section 16.2 |
| 84-69 S1 | Operation of Emergency Diesel Generators (2/86) | SSAR Section 16.2 |
| 84-70 | Reliance on Water Level Instrumentation with a Common Reference Leg (9/84) | SSAR Section 7.1.4.2.2 |
| 84-70 S1 | Reliance on Water Level Instrumentation with a Common Reference Leg (8/85) | SSAR Section 7.1.4.2.2 |
| 84-71 | Graphitic Corrosion of Cast Iron in Salt Water (9/84) | Not Applicable AP600 design has no safety-related service water system |
| 84-72 | Clarification of Conditions for Waste Shipments Subject to Hydrogen Gas Generation (9/84) | Not Applicable Administrative communication |
| 84-73 | Downrating of Self-Aligning Ball Bushings Used in Snubbers (9/84) | Not Applicable Procurement issue |
| 84-74 | Isolation of Reactor Coolant System from Low-Pressure Systems Outside Containment (9/84) | SSAR Section 7.6.1 |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 84-75 | Calibration Problems - Eberline Instrument Model 6112B Analog Detectors (10/84) | Not Applicable Procurement/Procedural issue |
| 84-76 | Loss of All AC Power (10/84) | Not Applicable AP600 design has no safety-related AC power |
| 84-77 | Incident Involving Teletherapy Unit (AECL Eldorado-78) (10/84) | Not applicable to commercial nuclear power plants |
| 84-78 | Underrated Terminal Blocks That May Adversely Affect Operation of Essential Electrical Equipment (11/84) | Not Applicable Procurement issue |
| 84-79 | Failure to Properly Install Steam Separator at Vermont Yankee (11/84) | Not Applicable BWR only |
| 84-80 | Plant Transients Induced By Failure of Non-Nuclear Instrumentation Power (11/84) | Not Applicable B&W facilities only |
| 84-81 | Inadvertent reduction in primary Coolant Inventory in BWRs During Shutdown and Startup (11/84) | Not Applicable BWR only |
| 84-82 | Guidance for Posting Radiation Areas (11/84) | Not Applicable Administrative communication |
| 84-83 | Various Battery Problems (11/84) | Not Applicable Maintenance issue |
| 84-84 | Deficiencies in Ferro-Resonant Transformers (11/84) | Not Applicable Procurement issue |
| 84-84 R1 | Deficiencies in Ferro-Resonant Transformers (4/85) | Not Applicable Procurement issue |
| 84-85 | Molybdenum Breakthrough from Technetium-99m Generators (11/84) | Not applicable to commercial nuclear power plants |
| 84-86 | Isolation Between Signals of the Protection System and Non-Safety Related Equipment (11/84) | SSAR Section 7.1.2.11 |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 84-87 | Piping Thermal Deflection Induced by Stratified Flow (12/84) | USI A-1 SSAR Section 1.9.4 |
| 84-88 | Standby Gas Treatment Systems Problems (12/84) | Not Applicable Procurement issue |
| 84-89 | Stress Corrosion Cracking in Nonsensitized Stainless Steel (12/84) | Not Applicable BWR only |
| 84-90 | Main Steam Line Break Effect on Environmental Qualification of Equipment (12/84) | SSAR Section 3.11, Appendix 3D |
| 84-91 | Quality Control Problems of Meteorological Measurements Programs (12/84) | Not Applicable Procedural/Maintenance issue |
| 84-92 | Cracking of Flywheels on Cummings Fire Pump Diesel Engines (12/84) | Not Applicable Procurement issue |
| 84-93 | Potential for Loss of Water from the Refueling Cavity (12/84) | Not Applicable AP600 design has permanent seal |
| 84-94 | Reconcentration of Radionuclides Involving Discharges into Sanitary Sewage Systems Permitted Under 10 CFR 20.303 (12/84) | Not applicable to commercial nuclear power plants |
| 85-01 | Continuous Supervision of Irradiators (1/85) | Not applicable to commercial nuclear power plants |
| 85-02 | Improper Installation and Testing of Pressure Differential Transmitters (1/85) | Not Applicable Installation issue |
| 85-03 | Separation of Primary Reactor Coolant Pump Shaft and Impeller (1/85) | Not Applicable Maintenance issue |
| 85-03 S1 | Separation of Primary Reactor Coolant Pump Shaft and Impeller (4/85) | Not Applicable Maintenance issue |
| 85-04 | Inadequate Management of Security Response Drills (1/85) | Not Applicable Administrative issue |
| 85-05 | Pipe Whip Restraints (1/85) | Not Applicable Procurement issue |
| 85-06 | Contamination of Breathing Air Systems (1/85) | Not Applicable Procedural issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 85-07 | Contaminated Radiography Source Shipments (1/85) | Not applicable to commercial nuclear power plants |
| 85-08 | Industry Experience on Certain Materials Used in Safety-Related Equipment (1/85) | Not Applicable Part of COL Surveillance/Procedural issue |
| 85-09 | Isolation Transfer Switches and Post-Fire Shutdown Capability (1/85) | SSAR Section 9A.3.1.2.5.1 |
| 85-10 | Posttentioned Containment Tendon Anchor Head Failure (2/85) | Not Applicable SSAR Section 3.8.1 |
| 85-10 S1 | Posttentioned Containment Tendon Anchor Head Failure (3/85) | Not Applicable SSAR Section 3.8.1 |
| 85-11 | Licensee Programs for Inspection of Electrical Raceway and Cable Installations (2/85) | Not Applicable Installation/Inspection issue |
| 85-12 | Recent Fuel Handling Events (2/85) | Not Applicable Procedural issue |
| 85-13 | Consequences of Using Soluable Dams (2/85) | Not Applicable Maintenance issue |
| 85-14 | Failure of a Heavy Control Rod (B4C) Drive Assembly To Insert on a trip Signal (2/85) | Not Applicable AP600 design has no B4C control rods |
| 85-15 | Nonconforming Structural Steel for Safety-Related Use (2/85) | Not Applicable Procurement issue |
| 85-16 | Time/Current Trip Curve Discrepancy of ITE/Siemens-Allis Molded Case Circuit Breaker (2/85) | Not Applicable Procurement issue |
| 85-17 | Possible Sticking of ASCO Solenoid Valves (3/85) | Not Applicable Procurement issue |
| 85-17 S1 | Possible Sticking of ASCO Solenoid Valves (10/85) | Not Applicable Procurement issue |
| 85-18 | Failures of Undervoltage Output Circuit Boards in the Westinghouse Designed Solid State Protection System (3/85) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 85-19 | Alleged Falsification of Certifications and Alteration of Markings on Piping, Valves, and Fittings (3/85) | Not Applicable Procurement issue |
| 85-20 | Motor-Operated Valve Failures Due to Hammering Effect (3/85) | SSAR Section 10.4.7.2.2 Procurement issue |
| 85-20 S1 | Motor-Operated Valve Failures Due to Hammering Effect (5/85) | SSAR Section 10.4.7.2.2 Procurement issue |
| 85-21 | Main Steam Isolation Valve Closure Logic (3/85) | Not Applicable Plant Specific issue |
| 85-22 | Failure of Limitorque Motor-Operator Valve Resulting from Incorrect Installation of Pinion Gear (3/85) | Not Applicable Procurement issue |
| 85-23 | Inadequate Surveillance and Postmaintenance and Postmodification System Testing (3/85) | Not Applicable Part of COL Surveillance issue |
| 85-24 | Failures of Protective Coatings in Pipes and Heat Exchangers (3/85) | Not Applicable Procurement issue |
| 85-25 | Consideration of Thermal Conditions in the Design and Installation of Supports for Diesel Generator Exhaust Silencers (4/85) | SSAR Section 8.3.1.1.2.1 |
| 85-26 | Vacuum relief System for Boiling Water Reactor Mark I and Mark II Containments (4/85) | Not Applicable BWR only |
| 85-27 | Notifications to the NRC Operations Center and Reporting Events in Licensee Event Reports (4/85) | Not Applicable Administrative communication |
| 85-28 | Partial Loss of AC Power and Diesel Generator Degradation (4/85) | SSAR Section 16.2 |
| 85-29 | Use of Unqualified Sources in Well Logging Applications (4/85) | Not applicable to commercial nuclear power plants |
| 85-30 | Microbiologically Induced Corrosion of Containment Service Water System (4/85) | SSAR Section 16.2 |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 85-31 | Buildup of Enriched Uranium in Ventilation Ducts and Associated Effluent Treatment Systems (4/85) | Not applicable to commercial nuclear power plants |
| 85-32 | Recent Engine Failures of Emergency Diesel Generators (4/85) | SSAR Section 16.2 |
| 85-33 | Undersized Nozzle-to-Shell Welded Joints in Tanks and Heat Exchangers Constructed Under the Rules of the ASME Boiler and Pressure Vessel Code (4/85) | Not Applicable Procurement issue |
| 85-34 | Heat Tracing Contributes to Corrosion Failure of Stainless Steel Piping (4/85) | SSAR Section 9.3.6.1.2.3 |
| 85-35 | Failure of Air Check Valves to Seat (4/85) | Not Applicable Procurement issue |
| 85-35 S1 | Failure of Air Check Valves to Seat (5/88) | Not Applicable Procurement issue |
| 85-36 | Malfunction of a Dry-Storage, Panoramic, Gamma Exposure Irradiator (5/85) | Not applicable to commercial nuclear power plants |
| 85-37 | Chemical Cleaning of Steam Generators at Millstone 2 (5/85) | Not Applicable Maintenance issue |
| 85-38 | Loose Parts Obstruct Control Drive Mechanism (5/85) | Not Applicable B & W facilities only |
| 85-39 | Auditability of Electrical Equipment Qualification Records at Licensees' Facilities (5/85) | SSAR Sections 1.9 and 3.11 |
| 85-40 | Deficiencies in Equipment Qualification Testing and Certification Process (5/85) | SSAR Sections 1.9 and 3.11 |
| 85-41 | Scheduling of Pre-Licensing Emergency Preparedness Exercises (5/85) | Not Applicable Administrative communication |
| 85-42 | Loose Phosphor in Panasonic 800 Series Badge Thermoluminescent Dosimeter (TLD) Elements (5/85) | Not Applicable Procurement issue |
| 85-42 R1 | Loose Phosphor in Panasonic 800 Series Badge Thermoluminescent Dosimeter (TLD) Elements (5/85) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 85-43 | Radiography Events at Power Reactors (5/85) | Not Applicable Procedural issue |
| 85-44 | Emergency Communication System Monthly Test (5/85) | Surveillance issue Part of COL |
| 85-45 | Potential Seismic Interaction Involving the Movable In-Core Flux Mapping System Used in Westinghouse Designed Plants (6/85) | SSAR Section 3.9.7.1 |
| 85-46 | Clarification of Several Aspects of Removable Radioactive Surface Contamination Limits for Transport Packages (6/85) | Not Applicable Administrative communication |
| 85-47 | Potential Effect of Line-Induced Vibration on Certain Target Rock Solenoid-Operated Valves (6/85) | Not Applicable Procurement issue |
| 85-48 | Respirators Users Notice: Defective Self-Contained Breathing Apparatus Air Cylinders (6/85) | Not Applicable Procurement issue |
| 85-49 | Relay Calibration Problem (7/85) | Not Applicable Procurement issue |
| 85-50 | Complete Loss of Main and Auxiliary Feedwater at a PWR Designed by Babcock & Wilcox (7/85) | Not Applicable AP600 design has no safety-related AFW or similar safety-related system |
| 85-51 | Inadvertent Loss or Improper Actuation of Safety Related Equipment (7/85) | Not Applicable Procedural issue |
| 85-52 | Errors in Dose Assessment Computer Codes and Reporting Requirements Under 10 CFR Part 21 (7/85) | Not Applicable Procurement issue |
| 85-53 | Performance of Non-Licensed Individuals While on Duty (7/85) | Not Applicable Administrative issue |
| 85-54 | Teletherapy Unit Malfunctions (7/85) | Not applicable to commercial nuclear power plants |
| 85-55 | Revised Emergency Exercise Frequency Rule (7/85) | Not Applicable Part of COL |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 85-56 | Inadequate Environmental Control for Components and Systems in Extended Storage or Layup (7/85) | Not Applicable Storage issue |
| 85-57 | Lost Iridium-192 Source Resulting in the Death of Eight Persons in Morocco (7/85) | Not applicable to commercial nuclear power plants |
| 85-58 | Failure of a General Electric Type AK-2-25 Reactor Trip Breaker (7/85) | Not Applicable Procurement issue |
| 85-58 S1 | Failure of a General Electric Type AK-2-25 Reactor Trip Breaker (11/85) | Not Applicable Procurement issue |
| 85-59 | Valve Stem Corrosion Failures (7/85) | Not Applicable Procurement issue |
| 85-60 | Defective Negative Pressure, Air-Purifying, Full Facepiece Respirators (7/85) | Not Applicable Procurement issue |
| 85-61 | Misadministrations to Patients Undergoing Thyroid Scans (7/85) | Not applicable to commercial nuclear power plants |
| 85-61 S1 | Misadministrations to Patients Undergoing Thyroid Scans (7/85) | Not applicable to commercial nuclear power plants |
| 85-62 | Backup Telephone Numbers to the NRC Operations Center (7/85) | Not Applicable Administrative communication |
| 85-63 | Potential for Common Mode Failures of Standby Gas Treatment System on Loss of Off-Site Power (7/85) | Not Applicable Procurement issue |
| 85-64 | BBC Brown Boveri Low Voltage K-Line Circuit Breakers, with Deficient Overcurrent Trip Devices Models OD-4 and 5 (7/85) | Not Applicable Procurement issue |
| 85-65 | Crack Growth in Steam Generator Girth Welds (7/85) | Not Applicable Surveillance issue weld relocated |
| 85-66 | Discrepancies Between As-Built Construction Drawings and Equipment Installations (8/85) | Not Applicable Installation issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 85-67 | Valve-Shaft-To-Actuator Key May Fall Out of Place When Mounted Below Horizontal Axis (8/85) | Not Applicable Procurement issue |
| 85-68 | Diesel Generator Failure at Calvert Cliffs Nuclear Station Unit 1 (8/85) | SSAR Section 16.2 |
| 85-69 | Recent Felony Conviction for Cheating on Reactor Operator Requalification Tests (8/85) | Not Applicable Training/Administrative issue |
| 85-70 | Teletherapy Unit Full Calibration and Qualified Expert Requirements (10 CFR 35.23 and 10 CFR 35.24) (8/85) | Not applicable to commercial nuclear power plants |
| 85-71 | Containment Integrated Leak Tests (8/85) | SSAR Section 6.2.5 |
| 85-72 | Uncontrolled Leakage of Reactor Coolant Outside Containment (8/85) | Not Applicable BWR only |
| 85-73 | Emergency Diesel Generator Control Circuit Logic Design Error (8/85) | SSAR Section 8.3.1.1.2.1 |
| 85-74 | Station Battery Problems (8/85) | Not Applicable Maintenance/Training issue |
| 85-75 | Improperly Installed Instrumentation, Inadequate Quality Control and Inadequate Postmodification Testing (8/85) | Not Applicable Installation issue |
| 85-76 | Recent Water Hammer Events (9/85) | USI A-1 SSAR Section 1.9.4 |
| 85-77 | Possible Loss of Emergency Notification System Due to Loss of AC Power (9/85) | Not Applicable Part of COL |
| 85-78 | Event Notification (9/85) | Not Applicable Administrative communication |
| 85-79 | Inadequate Communication Between Maintenance, Operations, and Security Personnel (9/85) | Not Applicable Administrative communication |
| 85-80 | Timely Declaration of an Emergency Class, Implementation of an Emergency Plan, and Emergency Notifications (10/85) | Not Applicable Administrative communication |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 85-81 | Problems Resulting in Erroneously High reading with Panasonic 800 Series Thermoluminescent Dosimeters (10/85) | Not Applicable Procurement issue |
| 85-82 | Diesel Generator Differential Protection Relay Not Seismically Qualified (10/85) | SSAR Section 16.2 |
| 85-83 | Potential Failures of General Electric PK-2 Test Blocks (10/85) | Not Applicable Procurement issue |
| 85-84 | Inadequate Inservice Testing of Main Steam Isolation Valves (10/85) | Not Applicable SSAR Section 10.3.4.4 Surveillance issue |
| 85-85 | Systems Interaction Event Resulting in Reactor System Safety-Relief Valve Opening Following a Fire-Protection Deluge System Malfunction (10/85) | SSAR Section 9.4.1 |
| 85-86 | Lightning Strikes at Nuclear Power Generating Stations (11/85) | Not Applicable Part of COL |
| 85-87 | Hazards of Inerting Atmospheres (11/85) | Not Applicable AP600 design has no inerting atmospheres |
| 85-88 | Licensee Control of Contracted Service Providing Training (11/85) | Not Applicable Training issue |
| 85-89 | Potential Loss of Solid-State Instrumentation Following Failure of Control Room Cooling (11/85) | SSAR Section 3.11, Appendix 3D |
| 85-90 | Use of Sealing Compounds in an Operating System (11/85) | Not Applicable Maintenance issue |
| 85-91 | Load Sequencers for Emergency Diesel Generators (11/85) | SSAR Section 8.3.1.1.2.1 |
| 85-92 | Surveys of Waste Before Disposal from Nuclear Reactor Facilities (12/85) | Not Applicable Administrative issue |
| 85-93 | Westinghouse Type DS Circuit Breakers, Potential Failure of Electric Closing Feature Because of Broken Spring Release Latch Lever (12/85) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|--|
| 85-94 | Potential for Loss of Minimum Flow Paths Leading to ECCS Pump Damage During a LOCA (12/85) | Not Applicable AP600 design has no valves in miniflow lines |
| 85-95 | Leak of Reactor Water to Reactor Building Caused by Scram Solenoid Valve Problem (12/85) | Not Applicable BWR only |
| 85-96 | Temporary Strainers Left Installed in Pump Suction Piping (12/85) | Not Applicable Installation issue |
| 85-97 | Jail Term for Former Contractor Employee Who Intentionally Falsified Welding Inspection Records (12/85) | Not Applicable Procurement issue |
| 85-98 | Missing Jumpers from Westinghouse Reactor Protection System Cards for the Over-Power Delta Temperature Trip Function (12/85) | Not Applicable Maintenance/Procurement issue |
| 85-99 | Cracking in Boiling Water Reactor Mark I and II Containments Caused by Failure of the Inerting System (12/85) | Not Applicable BWR only |
| 85-100 | Rosemount Differential Pressure Transmitter Zero Point Shift (1/85) | Not Applicable Procurement issue |
| 85-101 | Applicability of 10 CFR 21 to Consulting Firm Providing Training (1/85) | Not Applicable Administrative communication |
| 86-01 | Failure of Main Feedwater Check Valve Causes Loss of Feedwater System Integrity and Water Hammer Damage (1/86) | SSAR Section 10.4.7.2.2 |
| 86-02 | Failure of Valve Operated Motor During Environmental Qualification Testing (1/86) | Not Applicable Procurement issue |
| 86-03 | Potential Deficiencies in Environmental Qualification of Limitorque Motor Valve Operator Wiring (1/86) | Not Applicable Procurement issue |
| 86-04 | Transient Due to Loss of Power to Integrated Control System at a Pressurized Water Reactor Designed by Babcock & Wilcox (1/86) | Not Applicable Plant specific issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 86-05 | Main Steam Safety Valve Test Failures and Ring Setting Adjustments (1/86) | Not Applicable Procurement issue |
| 86-05 S1 | Main Steam Safety Valve Test Failures and Ring Setting Adjustments (10/86) | Not Applicable Procurement issue |
| 86-06 | Failure of Lifting Rig Attachment While Lifting the Upper Guide Structure at St. Lucie Unit 1 (2/86) | Not Applicable Procedural issue |
| 86-07 | Lack of Detailed Instruction and Adequate Observance of Precautions During Maintenance and Testing of Diesel Generator Woodward Governors (2/86) | Not Applicable Maintenance issue |
| 86-08 | Licensee Event Report (LER) Format Modification (2/86) | Not Applicable Administrative communication |
| 86-09 | Failure of Check and Stop Check Valves Subjected to Low Flow Conditions (2/86) | Not Applicable Procurement issue |
| 86-10 | Safety Parameter Display System Malfunctions (2/86) | SSAR Section 18.9.5 |
| 86-11 | Inadequate Service Water Protection Against Core Melt Frequency (2/86) | SSAR Section 16.2 |
| 86-12 | Target Rock Two-Stage SRV Setpoint Drift (2/86) | Not Applicable Procurement issue |
| 86-13 | Standby Liquid Control System Valves Failure to Fire (2/86) | Not Applicable Procurement issue |
| 86-13 S1 | Standby Liquid Control System Valves Failure to Fire (8/86) | Not Applicable Procurement issue |
| 86-14 | PWR Auxiliary Feedwater Pump Turbine Control Problems (3/86) | Not Applicable AP600 design has no turbine driven pumps |
| 86-14 S1 | Overspeed Trips of AFW, HPCI, and RCIC Turbine (12/86) | Not Applicable AP600 design has no turbine driven pumps |
| 86-14 S2 | Overspeed Trips of AFW, HPCI, and RCIC Turbine (12/86) | Not Applicable AP600 design has no turbine driven pumps |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 86-15 | Loss of Offsite Power Caused by Problems in Fiber Optics Systems (3/86) | Not Applicable Part of COL |
| 86-16 | Failures to Identify Containment Leakage Due to Inadequate Local Testing of BWR Vacuum Relief System Valves (3/86) | Not Applicable BWR only |
| 86-17 | Update of Failure of Automatic Sprinkler System Valves to Operate (3/86) | Not Applicable Procurement issue |
| 86-18 | NRC On-Scene Response During a Major Emergency (3/86) | Not Applicable Administrative issue |
| 86-19 | Reactor Coolant Pump Shaft Failure at Crystal River (3/86) | Not Applicable Procurement issue |
| 86-20 | Low-Level Radioactive Waste Scaling Factors, 10 CFR Part 61 (3/86) | Not Applicable Administrative issue |
| 86-21 | Recognition of American Society of Mechanical Engineers Accreditation Program for N Stamp Holders (3/86) | Not Applicable Administrative communication |
| 86-21 S1 | Recognition of American Society of Mechanical Engineers Accreditation Program for N Stamp Holders (12/86) | No. Applicable Administrative communication |
| 86-21 S2 | Recognition of American Society of Mechanical Engineers Accreditation Program for N Stamp Holders (4/91) | Not Applicable Procurement issue |
| 86-22 | Underresponse of Radiation Survey Instrument to High Radiation Fields (3/86) | Not Applicable Procedural issue |
| 86-23 | Excessive Skin Exposures Due to Contamination with Hot Particles (4/86) | Not Applicable Procedural issue |
| 86-24 | Respirators Users Notice: Increase Inspection Frequency for Certain Self-Contained Breathing Apparatus Air Cylinders (4/86) | Not Applicable Procedural issue |
| 86-25 | Traceability and Material Control of Material and Equipment, Particularly Fasteners (4/86) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 86-25 S1 | Traceability and Material Control of Material and Equipment, Particularly Fasteners (10/86) | Not Applicable Procurement issue |
| 86-26 | Potential Problems in Generators Manufactured by Electrical Products Incorporated (4/86) | Not Applicable Procurement issue |
| 86-27 | Access Control at Nuclear Facilities (4/86) | Not Applicable Administrative communication |
| 86-28 | Telephone Numbers to the NRC Operations Center and Regional Offices (4/86) | Not Applicable Administrative communication |
| 86-29 | Effects of Changing Valve Motor-Operator Switch Settings (4/86) | Not Applicable Procedural issue |
| 86-30 | Design Limitations of Gaseous Effluent Monitoring Systems (4/86) | Not Applicable Procurement issue |
| 86-31 | Unauthorized Transfer and Loss of Control of Industrial Nuclear Gauges (5/86) | Not applicable to commercial nuclear power plants |
| 86-31 S1 | Unauthorized Transfer and Loss of Control of Industrial Nuclear Gauges (7/86) | Not applicable to commercial nuclear power plants |
| 86-32 | Request for Collection of Licensee Radioactivity Measurements Attributed to the Chernobyl Nuclear Plant Accident (5/86) | Not Applicable Administrative communication |
| 86-32 S1 | Request for Collection of Licensee Radioactivity Measurements Attributed to the Chernobyl Nuclear Plant Accident (6/86) | Not Applicable Administrative communication |
| 86-33 | Information for Licensees Regarding the Chernobyl Nuclear Plant Accident (5/86) | Not Applicable Administrative communication |
| 86-34 | Improper Assembly, Material Selection, and Test of Valves and Their Actuators (5/86) | Not Applicable Procurement/Installation issue |
| 86-35 | Fire in Compressible Material at Dresden Unit 3 (5/86) | Not Applicable Plant specific issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 86-36 | Changes in NRC Practice Regarding Issuance of Confirming Letters to Principal Contractors (5/86) | Not Applicable Administrative communication |
| 86-37 | Degradation of Station Batteries (5/86) | Not Applicable Procurement issue |
| 86-38 | Deficient Operator Actions Following Dual Function Valve Failures (5/86) | Not Applicable AP600 design has no dual function valves for core cooling |
| 86-39 | Failures of RHR Pump Motors and Internals (5/86) | Not Applicable Procurement issue |
| 86-40 | Degraded Ability to Isolate the Reactor Coolant System from Low-Pressure Coolant Systems in BWRs (6/86) | SSAR Section 7.6.1 |
| 86-41 | Evaluation of Questionable Exposure Readings of Licensee Personnel Dosimeters (6/86) | Not applicable to commercial nuclear power plants |
| 86-42 | Improper Maintenance of Radiation Monitoring Systems (6/86) | Not Applicable Maintenance issue |
| 86-43 | Problems with Silver Zeolite Sampling of Airborne Radioiodine (6/86) | Not Applicable Part of COL Surveillance issue |
| 86-44 | Failure to Follow Procedures When Working in High Radiation Areas (6/86) | Not Applicable Procedural/Training issue |
| 86-45 | Potential Falsification of Test Reports on Flanges Manufactured By Golden Gate Forge and Flange, Inc. (6/86) | Not Applicable Procurement issue |
| 86-46 | Improper Cleaning and Decontamination of Respiratory Protection Equipment (6/86) | Not Applicable Maintenance issue |
| 86-47 | Erratic Behavior of Static "O" Ring Differential Pressure Switches (6/86) | Not Applicable Procurement issue |
| 86-48 | Inadequate Testing of Boron Solution Concentration in the Standby Liquid Control System (6/86) | Not Applicable BWR only |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 86-49 | Age/Environment Induced Electrical Cable Failures (6/86) | Not Applicable Procurement issue |
| 86-50 | Inadequate Testing to Detect Failures of Safety-Related Pneumatic Components or Systems (6/86) | Not Applicable Part of COL Surveillance issue |
| 86-51 | Excessive Pneumatic Leakage in the Automatic Depressurization System (6/86) | Not Applicable BWR only |
| 86-52 | Conductor Insulation Degradation on Foxboro Model E Controllers (6/86) | Not Applicable Procurement issue |
| 86-53 | Improper installation of Heat Shrinkable Tubing (6/86) | Not Applicable Installation issue |
| 86-54 | Criminal Prosecution of a Former Radiation Safety Officer Who Willfully Directed an Unqualified Individual to Perform Radiography (6/86) | Not applicable to commercial nuclear power plants |
| 86-55 | Delayed Access to Safety-Related Areas and Equipment During Plant Emergencies (7/86) | Not Applicable Procedural issue |
| 86-56 | Reliability of Main Steam Safety Valves (7/86) | Not Applicable Procurement issue |
| 86-57 | Operating Problems with Solenoid Operated Valves at Nuclear Power Plants (7/86) | Not Applicable Procurement/Maintenance issue |
| 86-58 | Dropped Fuel Assembly (7/86) | Not Applicable Procedural issue |
| 86-59 | Increased Monitoring of Certain Patients with Implanted Coratomic, Inc. Model C-100 and C-101 Nuclear Powered Cardiac Pacemakers (7/86) | Not applicable to commercial nuclear power plants |
| 86-60 | Unanalyzed Post-LOCA Release Paths (7/86) | Not Applicable SSAR 1.2.1.2.3, no recycle path in makeup |
| 86-61 | Failure of Auxiliary Feedwater Manual Isolation Valve (7/86) | Not Applicable Maintenance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 86-62 | Potential Problems in Westinghouse Molded Circuit Breakers Equipped with a Shunt Trip (7/86) | Not Applicable Procurement issue |
| 86-63 | Loss of Safety Injection Capability (8/86) | Not Applicable AP600 has no safety-related pumps |
| 86-64 | Deficiencies in Upgrade Programs for Plant Emergency Operating Procedures (8/86) | SSAR Section 18.9.8.1.1.3 |
| 86-64 S1 | Deficiencies in Upgrade Programs for Plant Emergency Operating Procedures (4/87) | SSAR Section 18.9.8.1.1.3 |
| 86-65 | Malfunctions of ITT Barton Model 580 Series Switches During Requalification Testing (8/86) | Not Applicable Procurement issue |
| 86-66 | Potential for Failure of Replacement AC Coils Supplied by the Westinghouse Electric Corporation for Use in Class 1E Motor Starters and Contactors (8/86) | Not Applicable Procurement issue |
| 86-67 | Portable Moisture/Density Gauges: Recent Incidents and Common Violations of Requirements for Use, Transportation, and Storage (8/86) | Not applicable to commercial nuclear power plants |
| 86-68 | Stuck Control Rod (8/86) | Not Applicable BWR only |
| 86-69 | Spurious System Isolations Caused by the Panalarm Model 86 Thermocouple Monitor (8/86) | Not Applicable Procurement issue |
| 86-70 | Potential Failure of All Emergency Diesel Generators (8/86) | SSAR Section 16.2 |
| 86-71 | Recent Identified Problems with Limitorque Motor Operators (8/86) | Not Applicable Procurement issue |
| 86-72 | Failure of 17-7 PH Stainless Steel Springs in Valcor Valve Due to Hydrogen Embrittlement (8/86) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 86-73 | Recent Emergency Diesel Generator Problems (8/86) | SSAR Section 16.2 |
| 86-74 | Reduction of Reactor Coolant Inventory Because of Misalignment of RHR Valves (8/86) | Not Applicable BWR only |
| 86-75 | Incorrect Maintenance Procedure on Traversing Incore Probe Lines (8/86) | Not Applicable Maintenance issue |
| 86-76 | Problems Noted in Control Room Emergency Ventilation Systems (8/86) | SSAR Chapter 16, Section 3.7.6 |
| 86-77 | Computer Program Error Report Handling (8/86) | SSAR Section 17.0 |
| 86-78 | Scram Solenoid Pilot Valve (SSPV) Rebuild Kit Problems (9/86) | Not Applicable Procurement issue |
| 86-79 | Degradation or Loss of Charging Systems at PWR Nuclear Power Plants Using Swing-Pump Designs (9/86) | Not Applicable AP600 has no safety-related pumps |
| 86-80 | Unit Startup with Degraded High Pressure Safety Injection System (9/86) | SSAR Chapter 16, Section 1.0 |
| 86-81 | Broken Inner-External Closure Springs on Atwood & Morrill Main Steam Isolation Valves (9/86) | Not Applicable Procurement issue |
| 86-81 S1 | Broken Inner-External Closure Springs on Atwood & Morrill Main Steam Isolation Valves (1/88) | Not Applicable BWR only |
| 86-82 | Failures of Scram Discharge Volume Vent and Drain Valves (9/86) | Not Applicable BWR only |
| 86-82 R1 | Failures of Scram Discharge Volume Vent and Drain Valves (11/86) | Not Applicable Procurement issue |
| 86-83 | Underground Pathways into Protected Areas, Vital Areas, Material Access Areas, and Controlled Access Areas (8/86) | SSAR Section 13.6 |
| 86-84 | Rupture of a Nominal 40-Millicurie Iodine-125 Brachytherapy Seed Causing Significant Spread of Radioactive Contamination (9/86) | Not applicable to commercial nuclear power plants |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 86-85 | Enforcement Actions Against Medical Licensees for Willful Failure to Report Misadministrations (10/86) | Not applicable to commercial nuclear power plants |
| 86-86 | Clarification of Requirements for Fabrication and Export of Certain Previously Approved Type B Packages (10/86) | Not Applicable Administrative communication |
| 86-87 | Loss of Offsite Power Upon an Automatic Bus Transfer (10/86) | Not Applicable Plant specific issue |
| 86-88 | Compensatory Measures for Prolonged Periods of Security System Failures (10/86) | Not Applicable Administrative communication |
| 86-89 | Uncontrolled Rod Withdrawal Because of a Single Failure (10/86) | Not Applicable BWR only |
| 86-90 | Request to Dispose of Very Low Level Radioactive Waste Pursuant to 10 CFR 20.302 (11/86) | Not Applicable Administrative communication |
| 86-91 | Limiting Access Authorizations (11/86) | SSAR Section 13.6 |
| 86-92 | Pressurizer Safety Valve Reliability (11/86) | Not Applicable Maintenance/Procurement issue |
| 86-93 | IEB 85-03 Evaluation of Motor Operators Identifies Improper Torque Switch Settings (11/86) | Not Applicable SSAR Section 3.9.6.2 Surveillance issue |
| 86-94 | Hilti Concrete Expansion Anchor Bolts (11/86) | Not Applicable Procurement issue |
| 86-95 | Leak Testing Iodine-125 Sealed Sources in Lixi, Inc. Imaging Devices and Bone Mineral Analyzers (11/86) | Not applicable to commercial nuclear power plants |
| 86-96 | Heat Exchanger Fouling Can Cause Inadequate Operability of Service Water Systems (11/86) | Not Applicable Maintenance issue |
| 86-97 | Emergency Communications System (11/86) | Not Applicable Part of COL |
| 86-98 | Offsite Medical Services (12/86) | Not Applicable Administrative communication |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|-----------|---|--|
| 86-99 | Degradation of Steel Containments (12/86) | SSAR Section 3.8.2.7 |
| 86-99 S1 | Degradation of Steel Containments (2/91) | SSAR Section 3.8.2.7 |
| 86-100 | Loss of Offsite Power to Vital Buses at Salem 2 (12/86) | Not Applicable Plant specific event |
| 86-101 | Loss of Decay Heat Removal Due to Loss of Fluid in Reactor Coolant System (12/86) | Not Applicable Procedural/Maintenance issue |
| 86-102 | Repeated Multiple Failures of Steam Generator Hydraulic Snubbers Due to Control Valve Sensitivity (12/86) | SSAR Sections 5.2.4, 6.6 |
| 86-103 | Respirator Coupling Nut Assembly Failures (12/86) | Not Applicable Procurement issue |
| 86-104 | Unqualified Butt Splice Connectors Identified in Qualified Penetrations (12/86) | Not Applicable Procurement issue |
| 86-105 | Potential for Loss of Reactor Trip Capability at Intermediate Power Levels (12/86) | SSAR Section 7.2.1.1.10 |
| 86-106 | Feedwater Line Break (12/86) | SSAR Section 10.3.6 |
| 86-106 S1 | Feedwater Line Break (2/87) | SSAR Section 10.3.6 |
| 86-106 S2 | Feedwater Line Break (3/87) | SSAR Section 10.3.6 |
| 86-106 S3 | Feedwater Line Break (11/88) | SSAR Section 10.3.6 |
| 86-107 | Entry Into PWR Cavity with Retractable Incore Detector Thimbles Withdrawn (12/86) | Not Applicable Procedural/Maintenance issue |
| 86-108 | Degradation of Reactor Coolant System Pressure Boundary Resulting from Boric Acid Corrosion (4/87) | Part of COL Surveillance issue |
| 86-108 S1 | Degradation of Reactor Coolant System Pressure Boundary Resulting from Boric Acid Corrosion (4/87) | Part of COL Surveillance issue |
| 86-108 S2 | Degradation of Reactor Coolant System Pressure Boundary Resulting from Boric Acid Corrosion (11/87) | Part of COL Surveillance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 86-109 | Diaphragm Failure in Scram Outlet Valve Causing Rod Insertion (12/86) | Not Applicable BWR only |
| 86-110 | Anomalous Behavior of Recirculation Loop Flow in Jet Pump BWR (12/86) | Not Applicable BWR only |
| 87-01 | RHR Valve Misalignment Causes Degradation of ECCS in PWRs (1/87) | Not Applicable Maintenance/Procedural issue |
| 87-02 | Inadequate Seismic Qualification of Diaphragm Valves by Mathematical Modeling and Analysis (1/87) | Not Applicable Procurement issue |
| 87-03 | Segregation of Hazardous and Low-Level Radioactive Wastes (1/87) | Not Applicable Administrative communication |
| 87-04 | Diesel Generator Fails Test Because of Degraded Fuel (1/87) | SSAR Section 16.2 |
| 87-05 | Miswiring in a Westinghouse Rod Control System (2/87) | Not Applicable Procurement/Installation issue |
| 87-06 | Loss of Suction to Low Pressure Service Water System Pumps Resulting from Loss of Siphon (1/87) | SSAR Section 16.2 |
| 87-07 | Quality Control of Onsite Dewatering/Solidification Operations By Outside Contractors (2/87) | Not Applicable Administrative communication |
| 87-08 | Degraded Motor Leads in Linitorque DC Motor Operators (2/87) | Not Applicable Procurement issue |
| 87-09 | Emergency Diesel Generator Room Cooling Design Deficiency (2/87) | SSAR Section 9.4.10 |
| 87-10 | Potential for Water Hammer During Restart of Residual Heat Removal Pumps (2/87) | Not Applicable BWR only |
| 87-11 | Enclosure of Vital Equipment Within Designated Vital Areas (2/87) | SSAR Section 13.6.5.2 |
| 87-12 | Potential Problem with Metal Clad Circuit Breakers, General Electric Type AKF-2-25 | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 87-13 | Potential for High Radiation Fields Following Loss of Water from Fuel pool (2/87) | Not Applicable Procedural issue |
| 87-14 | Actuation of Fire Suppression System Causing Inoperability of Safety-Related Ventilation Equipment (3/87) | Not Applicable AP600 design has no safety-related ventilation system |
| 87-15 | Compliance with the Posting Requirements of Subsection 223b of the Atomic Energy Act of 1954, As Amended (3/87) | Not Applicable Administrative communication |
| 87-16 | Degradation of Static "O" Ring Pressure Switches (4/87) | Not Applicable Procurement issue |
| 87-17 | Response Time of Scram Instrument Volume Level Detectors (4/87) | Not Applicable BWR only |
| 87-18 | Unauthorized Service on Teletherapy Units by Nonlicensed Maintenance Personnel (4/87) | Not applicable to commercial nuclear power plants |
| 87-19 | Perforation and Cracking of Rod Cluster Control Assemblies (4/87) | Part of COL Surveillance issue |
| 87-20 | Hydrogen Leak in Auxiliary Building (4/87) | SSAR Section 9.3.2 |
| 87-21 | Shutdown Order Issued Because Licensed Operators Asleep While on Duty (5/87) | Not Applicable Administrative communication |
| 87-22 | Operator Licensing Requalification Examinations at Nonpower Reactors (5/87) | Not applicable to commercial nuclear power plants |
| 87-23 | Loss of Decay Heat Removal During Low Reactor Coolant Level Operation (5/87) | Not Applicable Procedural/Maintenance issue |
| 87-24 | Operational Experience Involving Losses of Electrical Inverters (6/87) | SSAR Section 8.3 |
| 87-25 | Potentially Significant Problems Resulting from Human Error Involving Wrong Unit, Wrong Train, or Wrong Component Events (6/87) | Not Applicable Administrative communication |
| 87-26 | Cracks in Stiffening Rings on 48-Inch-Diameter UF ₆ Cylinders (6/87) | Not applicable to commercial nuclear power plants |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 87-27 | Iranian Official Implies Vague Threat to U.S. Reactors (6/87) | Not Applicable Administrative communication |
| 87-28 | Air Systems Problems at U.S. Light Water Reactors (6/87) | Not Applicable Maintenance issue |
| 87-28 S1 | Air Systems Problems at U.S. Light Water Reactors (12/87) | Not Applicable Administrative communication |
| 87-29 | Recent Safety-Related Incidents at Large Irradiators (6/87) | Not applicable to commercial nuclear power plants |
| 87-30 | Cracking of Surge Ring Brackets in Large General Electric Company Electric Motors (7/87) | Not Applicable Procurement issue |
| 87-31 | Blocking, Bracing, and Securing of Radioactive Materials Packages in Transportation (7/87) | Not Applicable Administrative communication |
| 87-32 | Deficiencies in the Testing of Nuclear Grade Charcoal (7/87) | SSAR Section 9.4.7 |
| 87-33 | Applicability of 10 CFR Part 21 to Nonlicensees (7/87) | Not Applicable Administrative/Procurement issue |
| 87-34 | Single Failures in Auxiliary Feedwater Systems (7/87) | SSAR Section 16.2 |
| 87-35 | Reactor Trip Breaker, Westinghouse Model DS-416, Failed to Open on Manual Initiation from the Control Room (7/87) | Not Applicable Procurement issue |
| 87-35 S1 | Reactor Trip Breaker, Westinghouse Model DS-416, Failed to Open on Manual Initiation from the Control Room (12/87) | Not Applicable Procurement issue |
| 87-36 | Significant Unexpected Erosion of Feedwater Lines (8/87) | Surveillance issue SSAR Section 10.3.6 |
| 87-37 | Compliance with the General License Provisions of 10 CFR Part 31 (8/87) | Not Applicable Administrative communication |
| 87-38 | Inadequate or Inadvertent Blocking of Valve Movement (8/87) | Not Applicable Procedural/Maintenance issue |
| 87-39 | Control of Hot Particle Contamination at Nuclear Power Plants (8/87) | Not Applicable Administrative communication |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 87-40 | Backseating Valves Routinely to Prevent Packing Leakage (8/87) | Not Applicable Procurement/Procedural issue |
| 87-41 | Failures of Certain Brown Boveri Electric Circuit Breakers (8/87) | Not Applicable Procurement issue |
| 87-42 | Diesel Generator Fuse Contacts (9/87) | SSAR Section 16.2 |
| 87-43 | Gaps in Neutron Absorbing Material in High Density Spent Fuel Storage Racks (9/87) | Not Applicable Procurement issue |
| 87-44 | Thimble Tube Thinning in Westinghouse Reactors (9/87) | Not Applicable AP600 does not have this design |
| 87-44 S1 | Thimble Tube Thinning in Westinghouse Reactors (3/88) | Not Applicable AP600 does not have this design |
| 87-45 | Recent Safety Related Violations of NRC Requirements by Industrial Radiography Licensees (9/87) | Not applicable to commercial nuclear power reactors |
| 87-46 | Undetected Loss of Reactor Coolant (9/87) | Not Applicable Administrative/Procedural issue |
| 87-47 | Transportation of Radiography Devices (Update of IE Information Notice No. 81-02, 1/23/81) (10/87) | Not applicable to commercial nuclear power plants |
| 87-48 | Information Concerning the Use of Anaerobic Adhesives/Sealants (10/87) | Not Applicable Maintenance/Procedural issue |
| 87-49 | Deficiencies in Outside Containment Flooding Protection (10/87) | SSAR Section 3.4.1 |
| 87-50 | Potential LOCA at High and Low Pressure Interfaces from Fire Damage (10/87) | SSAR Section 9.5.1 |
| 87-51 | Failure of Low Pressure Safety Injection Pump Due to Seal Problems (10/87) | SSAR Section 16.2 |
| 87-52 | Insulation Breakdown of Silicone Rubber - Insulated Single Conductor Cables During High Potential Testing (10/87) | Not Applicable Procurement issue |
| 87-53 | Auxiliary Feedwater Pump Trips Resulting From Low Suction Pressure (10/87) | SSAR Section 16.2 |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 87-54 | Emergency Response Exercises (10/87) | Not Applicable Administrative communication |
| 87-55 | Portable Moisture/Density Gauges: Recent Incidents of Portable Gauges Being Stolen or Lost (10/87) | Not Applicable Administrative communication |
| 87-56 | Improper Hydraulic Control Unit Installation at BWR Units (11/87) | Not Applicable BWR only |
| 87-57 | Loss of Emergency Boration Capability Due to Nitrogen Gas Intrusion (11/87) | Not Applicable AP600 design has no safety-related pumps |
| 87-58 | Continuous Communications Following Emergency Notifications (11/87) | Not Applicable Administrative communication |
| 87-59 | Potential RHR Pump Loss (11/87) | SSAR Section 16.2 |
| 87-60 | Depressurization of Reactor Coolant Systems in Pressurized Water Reactors (12/87) | Not Applicable Procedural/Maintenance issue |
| 87-61 | Failure of westinghouse W-2 Type Circuit Breaker Cell Switches (12/87) | Not Applicable Procurement issue |
| 87-61 S1 | Failure of westinghouse W-2 Type Circuit Breaker Cell Switches (5/88) | Not Applicable Procurement issue |
| 87-62 | Mechanical Failure of Indicating Type Fuses (12/87) | Not Applicable Procurement issue |
| 87-63 | Inadequate NPSH in Low Pressure Safety Systems (12/87) | SSAR Section 16.2 |
| 87-64 | Conviction for Falsification of Security Training Records (12/87) | Not Applicable Administrative communication |
| 87-65 | Plant Operator, Beyond Analyzed Conditions (12/87) | Not Applicable Procedural issue |
| 87-66 | Inappropriate Application of Commercial Grade Components (12/87) | Not Applicable Procurement issue |
| 87-67 | Lessons Learned from Regional Inspections of Licensee Actions in Response to IE Bulletin 80-11 (12/87) | Not Applicable AP600 design has no safety-related masonry walls |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 88-01 | Safety Injection Pipe Failure (1/88) | Not Applicable Surveillance issue Part of COL |
| 88-02 | Lost or Stolen Gauges (2/88) | Not applicable to commercial nuclear power plants |
| 88-03 | Cracks in Shroud Support Access Hole Cover Welds (2/88) | Not Applicable BWR only |
| 88-04 | Inadequate Qualification and Documentation of Fire Barrier Penetration Seals (2/88) | Not Applicable Procurement issue |
| 88-04 S1 | Inadequate Qualification and Documentation of Fire Barrier Penetration Seals (8/88) | Not Applicable Procurement issue |
| 88-05 | Fire in Annunciator Control Cabinets (2/88) | Not Applicable Procurement issue |
| 88-06 | Foreign Objects in Steam Generators (2/88) | Not Applicable Procedural issue |
| 88-07 | Inadvertent Transfer of Licensed Material to Uncontrolled Locations (3/88) | Not applicable to commercial nuclear power plants |
| 88-08 | Chemical Reactions with Radioactive Waste Solidification Agents (3/88) | SSAR Section 11.4.2.4.1 |
| 88-09 | Reduced Reliability of Steam Driven Auxiliary Feedwater Pumps Caused by Instability of Woodward PG-PL Type Governors (3/88) | Not Applicable AP600 design has no turbine driven pumps |
| 88-10 | Material Licensees: Lack of Management Controls Over Licensed Programs (3/88) | Not applicable to commercial nuclear power plants |
| 88-11 | Potential Loss of Motor Control Center and/or Switchboard Functions Due to Faulty Tie Bolts (4/88) | Not Applicable Procurement issue |
| 88-12 | Overgreasing of Electric Motor Bearings (4/88) | Not Applicable Maintenance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 88-13 | Water Hammer and Possible Piping Damage Caused by Misapplication of Kerotest Packless Metal Diaphragm Globe Valves (4/88) | Not Applicable Procurement issue |
| 88-14 | Potential Problems with Electric Relays (4/88) | Not Applicable Procurement issue |
| 88-15 | Availability of U.S. Food and Drug Administration (FDA) Approved Potassium Iodine for Use in Emergencies Involving Radioactive Iodine (4/88) | Not Applicable Administrative communication |
| 88-16 | Identifying Waste Generators in Shipments of Low Level Waste to Land Disposal Facilities (4/88) | Not Applicable Procedural issue |
| 88-17 | Summary of Responses to NRC Bulletin 87-01, "Thinning of Pipe Walls in Nuclear Power Plants" (4/88) | Surveillance issue SSAR Section 5.4.3.4, 10.3.6 |
| 88-18 | Malfunction of Lockbox on Radiography Device (4/88) | Not applicable to commercial nuclear power plants |
| 88-19 | Questionable Certification of Class 1E Components (4/88) | Not Applicable Procurement issue |
| 88-20 | Unauthorized Individuals Manipulating Controls and Performing Control Room Activities (5/88) | Not Applicable Procedural issue |
| 88-21 | Inadvertent Criticality Events at Oskarshamn and at U.S. Nuclear Power Plants (5/88) | Not Applicable Procedural issue |
| 88-22 | Disposal of Sludge from Onsite Sewage Treatment Facilities at Nuclear Power Stations (5/88) | Not Applicable Procedural issue |
| 88-23 | Potential for Gas Binding of High Pressure Safety Injection Pumps During a Loss of Coolant Accident (5/88) | Not Applicable AP600 design has no safety-related pumps |
| 88-23 S1 | Potential for Gas Binding of High Pressure Safety Injection Pumps During a Loss of Coolant Accident (1/89) | Not Applicable AP600 design has no safety-related pumps |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 88-23 S2 | Potential for Gas Binding of High Pressure Safety Injection Pumps During a Loss of Coolant Accident (1/90) | Not Applicable AP600 design has no safety-related pumps |
| 88-23 S3 | Potential for Gas Binding of High Pressure Safety Injection Pumps During a Loss of Coolant Accident (12/90) | Not Applicable AP600 design has no safety-related pumps |
| 88-24 | Failures of Air-Operated Valves Affecting Safety-Related Systems (5/88) | Not Applicable Installation/Procurement issue |
| 88-25 | Minimum Edge Distance for Expansion Anchor Bolts (5/88) | Not Applicable Procurement issue |
| 88-26 | Falsified Pre-Employment Screening Records (5/88) | Not Applicable Administrative issue |
| 88-27 | Deficient Electrical Terminations Identified in Safety Related Components (5/88) | Not Applicable Procurement issue |
| 88-28 | Potential for Loss of post-LOCA Recirculation Capability Due to Insulation Debris Blockage (5/88) | Not Applicable Procurement issue |
| 88-29 | Deficiencies in Primary Containment Low-Voltage Electrical Penetration Assemblies (5/88) | Not Applicable Procurement issue |
| 88-30 | Target Rock Two-Stage SRV Setpoint Drift Update (5/88) | Not Applicable Procurement issue |
| 88-30 S1 | Target Rock Two-Stage SRV Setpoint Drift Update (2/90) | Not Applicable Procurement issue |
| 88-31 | Steam Generator Tube Rupture Analysis Deficiency (5/88) | SSAR Section 15.6.3 |
| 88-32 | Prompt Reporting to NRC of Significant Incidents Involving Radioactive Material (5/88) | Not Applicable Procedural issue |
| 88-33 | Recent Problems Involving the Model Spec 2-T Radiographic Exposure Device (5/88) | Not applicable to commercial nuclear power plants |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 88-34 | Nuclear Material Control and Accountability of Non-Fuel Special Nuclear Material at Power Reactors (5/88) | Not Applicable Procedural issue |
| 88-35 | Inadequate Licensee Performed Vendor Audits (6/88) | Not Applicable Procedural issue |
| 88-36 | Possible Sudden Loss of RCS Inventory During Low Coolant Level Operation (6/88) | Not Applicable Procedural issue |
| 88-37 | Flow Blockage of Cooling Water to Safety System Components (6/88) | Not Applicable Maintenance issue |
| 88-38 | Failure of Undervoltage Trip Attachment on General Electric Circuit Breakers (6/88) | Not Applicable Procurement issue |
| 88-39 | Lasalle Unit 2 Loss of Recirculation Pumps with Power Oscillation Event (6/88) | Not Applicable BWR only |
| 88-40 | Examiner's Handbook for Developing Operator Licensing Examinations (6/88) | Not Applicable Administrative communication |
| 88-41 | Physical Protection Weaknesses Identified Through Regulatory Effectiveness Reviews (RERs) (6/88) | SSAR Section 13.6 |
| 88-42 | Circuit Breaker Failures Due to Loose Charging Spring Motor Mounting Bolts (6/88) | Not Applicable Procurement issue |
| 88-43 | Solenoid Valve Problems (6/88) | Not Applicable Procurement issue |
| 88-44 | Mechanical Binding of Spring Release Devices in Westinghouse Type DS-416 Circuit Breakers (6/88) | Not Applicable Procurement issue |
| 88-45 | Problems in Protective Relay and Circuit Breaker Coordination (7/88) | Not Applicable Installation issue |
| 88-46 | Licensee Report of Defective Refurbished Circuit Breakers (7/88) | Not Applicable Procurement issue |
| 88-46 S1 | Licensee Report of Defective Refurbished Circuit Breakers (7/88) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 88-46 S2 | Licensee Report of Defective Refurbished Circuit Breakers (12/88) | Not Applicable Procurement issue |
| 88-46 S3 | Licensee Report of Defective Refurbished Circuit Breakers (6/89) | Not Applicable Procurement issue |
| 88-46 S4 | Licensee Report of Defective Refurbished Circuit Breakers (9/89) | Not Applicable Procurement issue |
| 88-47 | Slower Than Expected Rod Drop Times (7/88) | SSAR Section 14.2.8.2.11 |
| 88-48 | Licensee Report of Defective Refurbished Valves (7/88) | Not Applicable Procurement issue |
| 88-48 S1 | Licensee Report of Defective Refurbished Valves (8/88) | Not Applicable Procurement issue |
| 88-48 S2 | Licensee Report of Defective Refurbished Valves (8/89) | Not Applicable Procurement issue |
| 88-49 | Marking, Handling, Control, Storage, and Destruction of Safeguards Information (7/88) | Not Applicable Administrative/Procedural issue |
| 88-50 | Effect of Circuit Breaker Capacitance on Availability of Emergency Power (7/88) | Not Applicable Plant specific issue |
| 88-51 | Failures of Main Steam Isolation Valve (MSIVs) (7/88) | SSAR Section 10.3.4.4 Surveillance issue |
| 88-52 | Failure of Interuterine Tandem of Fletcher Suit Applicator Brachytherapy Devices During Patient Treatment (7/88) | Not applicable to commercial nuclear power plants |
| 88-53 | Licensee Violations of NRC Regulations, Which Led to Medial Diagnostic Misadministrations (7/88) | Not applicable to commercial nuclear power plants |
| 88-54 | Failure of Circuit Breaker Following Installation of Amptector Direct Trip Attachment (7/88) | Not Applicable Procurement issue |
| 88-55 | Potential Problems Caused by Single Failure of an Engineered Safety Feature Swing Bus (8/88) | Not Applicable AP600 design has no ESF swing bus |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 88-56 | Potential Problems with Silicone Foam Fire Barrier Penetration Seals (8/88) | Not Applicable Procurement issue |
| 88-57 | Potential Loss of Safe Shutdown Equipment Due to Premature Silicon Controlled Rectifier Failure (8/88) | Not Applicable Procurement issue |
| 88-58 | Potential Problems with Asea Brown Boveri ITE-51L Time-Overcurrent Relays (8/88) | Not Applicable Procurement issue |
| 88-59 | Main Steam Isolation Valve Guide Rail Failure at Waterford Unit 3 (8/88) | Not Applicable Procurement issue |
| 88-60 | Inadequate Design and Installation of Watertight Penetration Seals (8/88) | Not Applicable Procurement/Installation issue |
| 88-61 | Control Room Habitability - Recent Reviews of Operating Experience (8/88) | SSAR Section 6.4.1.1 |
| 88-62 | Recent Findings Concerning Implementation of Quality Assurance Programs By Suppliers of Transport Packages (8/88) | Not applicable to commercial nuclear power plants |
| 88-63 | High Radiation Hazards from Irradiated Incore Detectors and Cables (8/88) | Not Applicable Procedural issue |
| 88-63 S1 | High Radiation Hazards from Irradiated Incore Detectors and Cables (10/90) | Not Applicable Procedural issue |
| 88-63 S2 | High Radiation Hazards from Irradiated Incore Detectors and Cables (10/90) | Not Applicable Procedural issue |
| 88-64 | Reporting Fires in Nuclear Process Systems at Nuclear Power Plants (8/88) | Not Applicable Administrative communication |
| 88-65 | Inadvertent Drainages of Spent Fuel Pools (8/88) | Not Applicable Procedural issue |
| 88-66 | Industrial Radiography Inspection and Enforcement (8/88) | Not applicable to commercial nuclear power plants |
| 88-67 | PWR Auxiliary Feedwater Pump Turbine Overspeed Trip Failure (8/88) | Not Applicable AP600 design has no turbine driven pumps |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 88-6L | Setpoint Testing of Pressurizer Safety Valves with Filled Loop Seals Using Hydraulic Assist Devices (8/88) | Not Applicable Procedural issue |
| 88-69 | Movable Contact Finger Binding in HFA Relays Manufactured by General Electric (8/88) | Not Applicable Procurement issue |
| 88-69 S1 | Movable Contact Finger Binding in HFA Relays Manufactured by General Electric (9/88) | Not Applicable Procurement issue |
| 88-70 | Check Valve Inservice Testing Program Deficiencies (8/88) | Not Applicable Part of COL |
| 88-71 | Possible Environmental Effect of the Reentry of Cosmos 1900 and Request for Collection of Licensee Radioactivity Measurements Attributed to that Event (9/88) | Not Applicable Administrative communication |
| 88-72 | Inadequacies in the Design of DC Motor Operated Valves (9/88) | Not Applicable Procurement issue |
| 88-73 | Direction-Dependent Leak Characteristics of Containment Purge Valves (9/88) | Not Applicable Procurement issue |
| 88-73 S1 | Direction-Dependent Leak Characteristics of Containment Purge Valves (2/89) | Not Applicable Procurement issue |
| 88-74 | Potentially Inadequate Performance of ECCS in PWRs During Recirculation Operation Following a LOCA (9/88) | SSAR Section 16.6.5.4B.3.1 |
| 88-75 | Disabling of Diesel Generator Output Circuit Breakers by Anti-Pump Circuitry (9/88) | SSAR Section 8.3.1.1.2.1 |
| 88-75 S1 | Disabling of Diesel Generator Output Circuit Breakers by Anti-Pump Circuitry (4/89) | SSAR Section 8.3.1.1.2.1 |
| 88-76 | Recent Discovery of a Phenomenon Not Previously Considered in the Design of Secondary Containment Pressure Control (9/88) | Not Applicable Procedural issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 88-77 | Inadvertent Reactor Vessel Overfill (9/88) | Not Applicable BWR only |
| 88-78 | Implementation of Revised NRC Administered Requalification Examinations (9/88) | Not Applicable Administrative communication |
| 88-79 | Misuse of Flashing Lights for High Radiation Area Controls (10/88) | Not Applicable Procedural issue |
| 88-80 | Unexpected Piping Movement Attributed to thermal Stratification (10/88) | SSAR Section 5.4.10.1 |
| 88-81 | Failure of AMP Window Indent Kynar Splices and Thomas and Betts Nylon Wire Caps During Environmental Qualification Testing (10/88) | Not Applicable Procurement issue |
| 88-82 | Torus Shells with Corrosion and Degraded Coatings in BWR Containments (10/88) | Not Applicable BWR only |
| 88-82 S1 | Torus Shells with Corrosion and Degraded Coatings in BWR Containments (5/89) | Not Applicable BWR only |
| 88-83 | Inadequate Testing of Relay Contacts in Safety Related Logic Systems (10/88) | Not Applicable Procedural issue |
| 88-84 | Defective Motor Shaft Keys in Limitorque Motor Actuators (10/88) | Not Applicable Procurement issue |
| 88-85 | Broken Retaining Block Studs on Anchor Darling Check Valves (10/88) | Not Applicable Procurement issue |
| 88-86 | Operating with Multiple Grounds in Direct Current Distribution Systems (10/88) | SSAR Section 8.3.2.2 |
| 88-86 S1 | Operating with Multiple Grounds in Direct Current Distribution Systems (3/89) | SSAR Section 8.3.2.2 |
| 88-87 | Pump Wear and Foreign Objects in Plant Piping Systems (11/88) | Not Applicable Inspection/Maintenance issue |
| 88-88 | Degradation of Westinghouse ARD Relays (11/88) | Not Applicable Procurement issue |
| 88-88 S1 | Degradation of Westinghouse ARD Relays (5/89) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 88-89 | Degradation of Kapton Electrical Insulation (11/88) | Not Applicable Procurement issue |
| 88-90 | Unauthorized Removal of Industrial Nuclear Gauges (11/88) | Not applicable to commercial nuclear power plants |
| 88-91 | Improper Administration and Control of Psychological Tests (11/88) | Not Applicable Procedural issue |
| 88-92 | Potential for Spent Fuel Pool Draindown (11/88) | Not Applicable Procurement issue |
| 88-93 | Teletherapy Events (12/88) | Not applicable to commercial nuclear power plants |
| 88-94 | Potentially Undersized Valve Actuators (12/88) | Not Applicable Procurement issue |
| 88-95 | Inadequate Procurement Requirements Imposed by Licensees on Vendors (12/88) | Not Applicable Procurement issue |
| 88-96 | Electrical Shock Fatalities at Nuclear Power Plants (12/88) | Not Applicable Procedural issue |
| 88-97 | Potential Substandard Valve Replacement Parts (12/88) | Not Applicable Procurement issue |
| 88-97 S1 | Potential Substandard Valve Replacement Parts (4/89) | Not Applicable Procurement issue |
| 88-98 | Electrical Relay Degradation Caused by Oxidation of Contact Surfaces (12/88) | Not Applicable Procurement/Maintenance issue |
| 88-99 | Detection and Monitoring of Sudden and/or Rapidly Increasing Primary to Secondary Leakage (12/88) | SSAR Section 11.5 |
| 88-100 | Memorandum of Understanding Between NRC and OSHA Relating to NRC-Licensed Facilities (53 FR 43950, 10/31/88) (12/88) | Not Applicable Administrative issue |
| 88-101 | Shipments of Contaminated Equipment Between Nuclear Power Stations (12/88) | Not Applicable Procedural issue |
| 89-01 | Valve Body Erosion (1/89) | SSAR Section 3.9.6.2 Surveillance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 89-02 | Criminal Prosecution of Licensee's Former President for Intentional Safety Violations (1/89) | Not Applicable Administrative communication |
| 89-03 | Potential Electrical Equipment Problems (1/89) | Not applicable to commercial nuclear power plants |
| 89-04 | Potential Problems from the Use of Space Heaters (1/89) | Not Applicable Administrative communication |
| 89-05 | Use of Deadly Force by Guards Protecting Nuclear Power Reactors Against Radiological Sabotage (1/89) | Not Applicable Administrative communication |
| 89-06 | Bent Anchor Bolts in Boiling Water Reactor Torus Supports (1/89) | Not Applicable BWR only |
| 89-07 | Failures of Small Diameter Tubing in Control Air, Fuel Oil, and Lube Oil Systems Which Render Emergency Diesel Generators Inoperable (1/89) | SSAR Section 16.2 |
| 89-08 | Pump Damage Caused by Low Flow Operation (1/89) | Not Applicable AP600 design has no valves in miniflow lines |
| 89-09 | Credit for Control Rods Without Scram Capability in the Calculation of the Shutdown Margin (1/89) | Not applicable to commercial nuclear power plants |
| 89-10 | Undetected Installation Errors in Main Steam Line Pipe Tunnel Differential Temperature Sensing Elements at Boiling Water Reactors (1/89) | Not Applicable Plant specific issue |
| 89-11 | Failures of DC Motor Operated Valves to Develop Rated Torque Because of Improper Cable Sizing (2/89) | Not Applicable Installation/Procurement issue |
| 89-12 | Dose Calibrator Quality Control (2/89) | Not applicable to commercial nuclear power plants |
| 89-13 | Alternative Waste Management Procedures in Case of Denial of Access to Low-Level Waste Disposal Sites (2/89) | Not Applicable Administrative communication |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 89-14 | Inadequate Dedication Process for Commercial Grade Components Which Could Lead to Common Mode Failures of a Safety System (2/89) | Not Applicable Procurement issue |
| 89-15 | Second Reactor Coolant Pump Shaft Failure at Crystal River (2/89) | Not Applicable Procurement issue |
| 89-16 | Excessive Voltage drop in DC Systems (2/89) | Not Applicable Installation/Procurement issue |
| 89-17 | Contamination and Degradation of Safety-Related Battery Cells (2/89) | Not Applicable Procurement /Maintenance issue |
| 89-18 | Criminal Prosecution of Wrongdoing Committed by Suppliers of Nuclear Products or Services (2/89) | Not Applicable Administrative communication |
| 89-18 S1 | Criminal Prosecution of Wrongdoing Committed by Suppliers of Nuclear Products or Services (8/90) | Not Applicable Administrative communication |
| 89-19 | Health Physics Network (2/89) | Not Applicable Administrative communication |
| 89-20 | Weld Failures in a Pump of Byron-Jackson Design (2/89) | Not Applicable Procurement issue |
| 89-21 | Changes in Performance Characteristics of Molded Case Circuit Breakers (2/89) | Not Applicable Procurement issue |
| 89-22 | Questionable Certification of Fasteners (3/89) | Not Applicable Procurement issue |
| 89-23 | Environmental Qualification of Litton-Veam CIR Series Electrical Connectors (3/89) | Not Applicable Procurement issue |
| 89-24 | Nuclear Criticality Safety (3/89) | Not applicable to commercial nuclear power plants |
| 89-25 | Unauthorized Transfer of Ownership or Control of Licensed Activities (3/89) | Not applicable to commercial nuclear power plants |
| 89-26 | Instrument Air Supply to Safety-Related Equipment (3/89) | Not Applicable Maintenance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 89-27 | Limitations on the Use of waste Forms and High Integrity Containers for the Disposal of Low-Level Radioactive Waste (3/89) | Not Applicable Administrative communication |
| 89-28 | Weight and Center of Gravity Discrepancies for Copes-Vulcan Air-Operated Vaives (3/89) | Not Applicable Procurement issue |
| 89-29 | Potential Failure of Asea Brown Boveri Circuit Breakers During Seismic Event (3/89) | Not Applicable Procurement issue |
| 89-30 | High Temperature Environments at Nuclear Power Plants (3/89) | SSAR Section 3.11, Appendix 3D |
| 89-30 S1 | High Temperature Environments at Nuclear Power Plants (11/90) | SSAR Section 3.11, Appendix 3D |
| 89-31 | Swelling and Cracking of Hafnium Control Rods (3/89) | Not Applicable AP600 design has no hafnium control rods |
| 89-32 | Surveillance Testing of Low-Temperature Overpressure Protection Systems (3/89) | SSAR Section 3.9.1.1.2.6 |
| 89-32 S1 | Surveillance Testing of Low-Temperature Overpressure Protection Systems (2/91) | SSAR Section 3.9.1.1.2.6 |
| 89-33 | Potential Failure of Westinghouse Steam Generator Tube Mechanical Plugs (3/89) | Not Applicable Procurement issue |
| 89-34 | Disposal of Americium Well-Logging Sources (3/89) | Not applicable to commercial nuclear power plants |
| 89-35 | Loss and Theft of Unsecured Licensed Material (3/89) | Not applicable to commercial nuclear power plants |
| 89-36 | Excessive Temperature in Emergency Core Cooling System Piping Located Outside Containment (4/89) | Not Applicable Plant specific event |
| 89-37 | Proposed Amendments to 40 CFR Part 61, Air Emissions Standards for Radionuclides (4/89) | Not Applicable Administrative communication |
| 89-38 | Atmospheric Dump Valve Failures at Palo Verde Units 1, 2, and 3 (4/89) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 89-39 | List of Parties Excluded from Federal Procurement or Nonprocurement Programs (4/89) | Not Applicable Administrative communication |
| 89-40 | Unsatisfactory Operator Test Results and Their Effect on the Requalification Program (4/89) | Not Applicable Administrative communication |
| 89-41 | Operator Response to Pressurization of Low Pressure Interfacing Systems (4/89) | Not Applicable Administrative communication |
| 89-42 | Failure of Rosemount Models 1153 and 1154 Transmitters (4/89) | Not Applicable Procurement issue |
| 89-43 | Permanent Deformation of Torque Switch Helical Springs in Limitorque SMA-Type Motor Operators (5/89) | Not Applicable Procurement issue |
| 89-44 | Hydrogen Storage on the Roof of the Control Room (4/89) | Not Applicable SSAR 9.3.2 |
| 89-45 | Metalcald, Low Voltage Power circuit Breakers Refurbished with Substandard Parts (5/89) | Not Applicable Procurement issue |
| 89-45 S1 | Metalcald, Low Voltage Power circuit Breakers Refurbished with Substandard Parts (7/89) | Not Applicable Procurement issue |
| 89-45 S2 | Metalcald, Low Voltage Power circuit Breakers Refurbished with Substandard Parts (12/89) | Not Applicable Procurement issue |
| 89-46 | Confidentiability of Exercise Scenarios (5/89) | Not applicable to commercial nuclear power plants |
| 89-47 | Potential Problems with Worn or Distorted Hose Clamps on Self-Contained Breathing Apparatus (5/89) | Not Applicable Maintenance issue |
| 89-48 | Design Deficiency in the Turbine-Driven Auxiliary Feedwater Pumps Cooling Water System (5/89) | Not Applicable AP600 design has no turbine driven pumps |
| 89-49 | Failure to Close Service Water Cross-Connect Isolation Valves (5/89) | Not Applicable Procedural issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 89-50 | Inadequate Emergency Diesel Generator Fuel Supply (5/89) | SSAR Section 16.2 |
| 89-51 | Potential Loss of Required Shutdown Margin During Refueling Operations (5/89) | Not Applicable Procedural issue |
| 89-52 | Potential fire Damper Operational Problems (6/89) | Not Applicable Procurement issue |
| 89-53 | Rupture of Extraction Steam Line on High Pressure Turbine (6/89) | Surveillance issue SSAR Section 5.4.3.4, 10.3.4 |
| 89-54 | Potential Overpressurization of the Component Cooling Water System (6/89) | SSAR Section 9.2.2.4.5.2 |
| 89-55 | Degradation of Containment Isolation by a High Energy Line Break (6/89) | Not Applicable BWR only |
| 89-56 | Questionable Certification of Material Supplied to the Defense Department by Nuclear Suppliers (7/89) | Not Applicable Procurement issue |
| 89-56 S1 | Questionable Certification of Material Supplied to the Defense Department by Nuclear Suppliers (11/89) | Not Applicable Procurement issue |
| 89-56 S2 | Questionable Certification of Material Supplied to the Defense Department by Nuclear Suppliers (1/90) | Not Applicable Procurement issue |
| 89-57 | Unqualified Electrical Splices in Vendor Supplied Environmentally Qualified Equipment (7/89) | Not Applicable Procurement issue |
| 89-58 | Disablement of Turbine Driven Auxiliary Feedwater Pump Due to Closure of One of the Parallel Steam Supply Valves (8/89) | Not Applicable AP600 design has no turbine driven pumps |
| 89-59 | Suppliers of Potentially Misrepresented Fasteners (8/89) | Not Applicable Procurement issue |
| 89-59 S1 | Suppliers of Potentially Misrepresented Fasteners (12/89) | Not Applicable Procurement issue |
| 89-59 S2 | Suppliers of Potentially Misrepresented Fasteners (3/90) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 89-60 | Maintenance of Teletherapy Units (8/89) | Not applicable to commercial nuclear power plants |
| 89-61 | Failure of Borg-Warner Gate Valves to Close Against Differential Pressure (8/89) | Not Applicable Procurement issue |
| 89-62 | Malfunctions of Borg-Warner Pressure Seal Bonnet Check Valves by Vertical Misalignment of Disk (8/89) | Not Applicable Procurement issue |
| 89-63 | Possible Submergence of Electrical Circuits Located Above the Flood Level Because of Water Intrusion and Lack of Drainage (9/89) | Not Applicable BWR only |
| 89-64 | Electrical Bus Bar Failures (9/89) | Not Applicable Part of COL Surveillance/Maintenance issue |
| 89-65 | Potential Stress Corrosion Cracking in Steam Generator Tube Plugs Supplied by Babcock & Wilcox (9/89) | Not Applicable B&W facilities only |
| 89-66 | Qualification Life of Solenoid Valves (9/89) | Not Applicable Procurement issue |
| 89-67 | Loss of Residual Heat Removal by Accumulator Nitrogen Injection (9/89) | Not Applicable Procedural issue |
| 89-68 | Evaluation of Instrument Setpoints During Modification (9/89) | Not Applicable Maintenance issue |
| 89-69 | Loss of Thermal Margin Caused by Channel Box Bow (9/89) | Not Applicable BWR only |
| 89-70 | Possible Indications of Misrepresented Vendor Products (10/89) | Not Applicable Procurement issue |
| 89-70 S1 | Possible Indications of Misrepresented Vendor Products (4/90) | Not Applicable Procurement issue |
| 89-71 | Diversion of the Residual Heat Removal Pump Seal Cooling Water Flow During Recirculation Operation Following a LOCA (10/89) | Not Applicable AP600 design has no safety-related pumps |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|---|
| 89-72 | Failure of Licensed Senior Operators to Classify Emergency Events Properly (10/89) | Not Applicable Procurement issue |
| 89-73 | Potential Overpressurization of Low Pressure Systems (11/89) | Not Applicable SSAR Section 7.6.1 |
| 89-74 | Clarification of Transportation Requirements Applicable to Return of Spent Radiopharmacy Dosages from Users to Suppliers (11/89) | Not applicable to commercial nuclear power plants |
| 89-75 | Falsification of Welder Qualification for Contractor Employees (11/89) | Not Applicable Administrative communication |
| 89-76 | Biofouling Event: Zebra Mussel (11/89) | Not Applicable Location/Site specific |
| 89-77 | Debris in Containment Emergency Sumps and Incorrect Screen Configurations (11/89) | Not Applicable Part of COL Surveillance/Maintenance issue |
| 89-78 | Failure of Packing Nuts on One-inch Uranium Hexafluoride Cylinder Valves (11/89) | Not Applicable Procurement/Maintenance issue |
| 89-79 | Degraded Coatings and Corrosion of Steel Containment Vessels (12/89) | SSAR Section 3.8.2 |
| 89-79 S1 | Degraded Coatings and Corrosion of Steel Containment Vessels (6/90) | SSAR Section 3.8.2 |
| 89-80 | Potential for Water Hammer, Thermal Stratification and Steam Binding in High Pressure Coolant Injection Piping (12/89) | Not Applicable BWR only |
| 89-81 | Inadequate Control of Temporary Modifications to Safety-Related Systems (12/89) | Not Applicable Administrative communication |
| 89-82 | Recent Safety Related Incidents at Large Irradiators (12/89) | Not applicable to commercial power plants |
| 89-83 | Sustained Degradation of Voltage on the Offsite Electrical Grid or Loss of Other Generating Stations as a Result of a Plant Trip (12/89) | Not Applicable Procedural/Plant specific issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 89-84 | Failure of Ingersoll Rand Air Start Motors as a Result of Pinion Gear Assembly Fitting Problems (12/89) | Not Applicable Procurement issue |
| 89-85 | EPA's Interim Final Rule on Medical Waste Tracking and Management (12/89) | Not applicable to commercial nuclear power plants |
| 89-86 | Type HK Circuit Breakers Missing Close Latch Anti-Shock Springs (12/89) | Not Applicable Procurement issue |
| 89-87 | Disabling of Emergency Diesel Generators by Their Neutral Ground-Fault Protection Circuitry (12/89) | SSAR Section 8.3.1.1.2.1 |
| 89-88 | Recent NRC Sponsored Testing of Motor Operated Valves (12/89) | Not Applicable Administrative issue |
| 89-89 | Event Notification Worksheets (12/89) | Not Applicable Administrative communication |
| 89-90 | Pressurizer Safety Valve Lift Setpoint Shift (12/89) | Not Applicable Procedural issue |
| 90-S1 | Pressurizer Safety Valve Lift Setpoint Shift (4/91) | Not Applicable Procedural issue |
| 90-S2 | Pressurizer Safety Valve Lift Setpoint Shift (4/91) | Not Applicable Procedural issue |
| 90-01 | Importance of Proper Response to Self-Identified Violations by Licensees (1/90) | Not Applicable Administrative communication |
| 90-02 | Potential Degradation of Secondary Containment (1/90) | Not Applicable BWR only |
| 90-03 | Malfunction of Borg-Warner Bolted Bonnet Check valve Caused by Failure of the Swing Arm (1/90) | Not Applicable Procurement issue |
| 90-04 | Cracking of the Upper Shell-To-Transition Cone Girth Welds in Steam Generators (1/90) | Not Applicable weld relocated |
| 90-05 | Inter-System Discharge of Reactor Coolant (1/90) | Not Applicable Maintenance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 90-06 | Potential for Loss of Shutdown Cooling While at low Reactor Coolant Levels (1/90) | Not Applicable Procedural issue |
| 90-07 | New Information Regarding Insulation Material Performance and debris Blockage of PWR Containment Sumps (1/90) | Not Applicable Procurement issue |
| 90-08 | Kr-85 Hazards from Decayed Fuel (2/90) | Not Applicable Procedural issue |
| 90-09 | Extended Interim Storage of Low-Level Radioactive Waste by Fuel Cycle and Material Licensees (2/90) | Not applicable to commercial nuclear power plants |
| 90-10 | Primary Water Stress Corrosion Cracking (PWSCC) of Inconel 600 (2/90) | Not Applicable Procurement issue |
| 90-11 | Maintenance Deficiencies Associated with Solenoid Operated Valves (2/90) | Not Applicable Maintenance issue |
| 90-12 | Monitoring or Interruption of Plant Communications (2/90) | Not Applicable Administrative communication |
| 90-13 | Importance of Review and Analysis of Safeguard Event Logs (3/90) | Not Applicable Procedural issue |
| 90-14 | Accidental Disposal of Radioactive Material (3/90) | Not applicable to commercial nuclear power plants |
| 90-15 | Reciprocity: Notification of Agreement State Radiation Control Directors Before Beginning Work in Agreement States (3/90) | Not Applicable Administrative communication |
| 90-16 | Compliance with New Decommissioning Rule (3/90) | Not applicable to comercial nuclear power plants |
| 90-17 | Weight and Center of Gravity Discrepancies for Copes-Vulcan Valves (3/90) | Not Applicable Procurement issue |
| 90-18 | Potential Problems with Crosby Safety Relief Valves Used on Diesel Generator Air Start Receiver Tanks (3/90) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 90-19 | Potential Loss of Effective Volume for Containment Recirculation Spray at PWR Facilities (3/90) | Not Applicable AP600 design has no containment spray |
| 90-20 | Personnel Injuries Resulting from Improper Operation of Radwaste Incinerators (3/90) | Not applicable to commercial nuclear power plants |
| 90-21 | Potential Failure of Motor Operated Butterfly Valves to Operate Because Valve Seat Friction Was Underestimated (3/90) | Not Applicable Procurement issue |
| 90-22 | Unanticipated Equipment Actuation Following Restoration of Power to Rosemount Transmitter Trip Units (3/90) | Not Applicable Procurement issue |
| 90-23 | Improper Installation of Patel Conduit Seals (4/90) | Not Applicable Installation issue |
| 90-24 | Transportation of Model Spec 2-T Radiographic Exposure Device (4/90) | Not applicable to commercial nuclear power plants |
| 90-25 | Loss of Vital AC Power with Subsequent Reactor Coolant System Heat-up (4/90) | Not Applicable Procedural issue |
| 90-25 S1 | Loss of Vital AC Power with Subsequent Reactor Coolant System Heat-up (3/91) | Not Applicable Procedural issue |
| 90-26 | Inadequate Flow of Essential Service Water to Room Coolers and Heat Exchangers for Engineered Safety-Feature System (4/90) | Not Applicable Procedural/Maintenance issue |
| 90-27 | Clarification of the Recent Revisions to the Regulatory Requirements for Packaging of Uranium Hexafluoride (UF ₆) for Transportation (4/90) | Not applicable to commercial nuclear power plants |
| 90-28 | Potential Error in High Steam Line Flow Setpoint (4/90) | Not Applicable Procedural issue |
| 90-29 | Cracking of Cladding and Its Heat Affected Zone in the Base Metal of a Reactor Vessel Head (5/90) | Not Applicable BWR only |
| 90-30 | Ultrasonic Inspection Techniques for Dissimilar Metal Welds (5/90) | Not Applicable Part of COL Surveillance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|--|
| 90-31 | Update on Waste Form and High Integrity Container Topical Report Review Status, Identification of Problems with Cement Solidification and Reporting of Waste Mishaps (5/90) | SSAR Section 11.4.2.4.1 |
| 90-32 | Surface Crack and Subsurface Indications in the Weld of a Reactor Vessel Head (5/90) | Not Applicable SSAR 5.3.4.7 Surveillance issue |
| 90-32 S1 | Surface Crack and Subsurface Indications in the Weld of a Reactor Vessel Head (6/90) | Not Applicable SSAR 5.3.4.7 Surveillance issue |
| 90-33 | Sources of Unexpected Occupational Radiation Exposures at Spent Fuel Storage Pools (5/90) | Not Applicable Procedural issue |
| 90-34 | Response to False Siren Activations (5/90) | Not Applicable Procedural issue |
| 90-35 | Transportation of Type A Quantities of Non-Fissile Radioactive Material (5/90) | Not Applicable Administrative/Procedural issue |
| 90-36 | Apparent Falsification of State of Connecticut Weight Certificates (5/90) | Not Applicable Administrative/Procurement issue |
| 90-37 | Sheared Pinion Gear-to-Shaft Keys in Limitorque Motor Actuators (5/90) | Not Applicable Procurement issue |
| 90-38 | Requirements for Processing Financial Assurance Submittals for Decommissioning (5/90) | Not applicable to commercial nuclear power plants |
| 90-38 S1 | Requirements for Processing Financial Assurance Submittals for Decommissioning (11/90) | Not applicable to nuclear power plants |
| 90-39 | Recent Problems with Service Water Systems (6/90) | SSAR Section 16.2 |
| 90-40 | Results of NRC Sponsored Testing of Motor Operated Valves (6/90) | Not Applicable Procurement issue |
| 90-41 | Potential Failure of General Electric Magne-Blast Circuit Breakers and AK Circuit Breakers (6/90) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 90-42 | Failure of Electrical Power Equipment Due to Solar Magnetic Disturbances (6/90) | Not Applicable Part of COL |
| 90-43 | Mechanical Interferences with Thermal trip Function in GE Molded Case Circuit Breakers (6/90) | Not Applicable Procurement issue |
| 90-43 S1 | Mechanical Interferences with Thermal trip Function in GE Molded Case Circuit Breakers (3/91) | Not Applicable Procurement issue |
| 90-44 | Dose-Rate Instruments Underresponding to the True Radiation Fields (6/90) | Not Applicable Procurement issue |
| 90-45 | Overspeed of the Turbine Driven Auxiliary Feedwater Pumps and Overpressurization of the Associated Piping Systems (7/90) | Not Applicable AP600 design has no turbine driven pumps |
| 90-46 | Criminal Prosecution of Wrongdoing Committed by Suppliers of Molded Case Circuit Breakers and Related Components (7/90) | Not Applicable Procurement issue |
| 90-47 | Unplanned Radiation Exposures to Personnel Extremities Due to Improper Handling of Potentially Highly Radioactive Sources (7/90) | Not Applicable Procedural issue |
| 90-48 | Enforcement Policy for Hot Particle Exposures (8/90) | Not Applicable Procedural issue |
| 90-49 | Stress Corrosion Cracking in PWR Steam Generator Tubes (8/90) | SSAR Section 5.4.2.4.3 |
| 90-50 | Minimization of Methane Gas in Plant Systems and Radwaste Shipping Containers (8/90) | SSAR Sections 11.4.2.2.1, 1.2 and part of COL |
| 90-51 | Failures of Voltage Dropping Resistors in the Power Supply Circuitry of Electric Governor Systems (8/90) | Not Applicable Procurement issue |
| 90-51 S1 | Failures of Voltage Dropping Resistors in the Power Supply Circuitry of Electric Governor Systems (9/90) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|--|
| 90-52 | Retention of Broken Charpy Specimens (8/90) | Not Applicable Administrative communication |
| 90-53 | Potential Failures of Auxiliary Steam Piping and the Possible Effects on the Operability of Vital Equipment (8/90) | SSAR Section 3.6.2 |
| 90-54 | Summary of Requalification Program Deficiencies (8/90) | Not Applicable Administrative communication |
| 90-55 | Recent Operating Experience on Loss of reactor Coolant Inventory While in a Shutdown Condition (8/90) | Not Applicable Procedural/Personnel issue |
| 90-56 | Inadvertent Shipment of a Radioactive Source in a Container Thought to be Empty (9/90) | Not Applicable Procedural issue |
| 90-57 | Substandard, Refurbished Potter & Brumfield Relays Misrepresented as New (9/90) | Not Applicable Procurement issue |
| 90-58 | Improper Handling of Ophthalmic Strontium-90 Beta Radiation Applicators (9/90) | Not applicable to commercial nuclear power plants |
| 90-59 | Errors in the Use of Radioactive Iodine-131 (9/90) | Not applicable to commercial nuclear power plants |
| 90-60 | Availability of Failure Data in the Government Industry Data Exchange Program (9/90) | Not Applicable Administrative communication |
| 90-61 | Potential for Residual Heat Removal Pump Damage Caused by Parallel Pump Interaction (9/90) | Not Applicable AP600 design has no safety-related pumps |
| 90-62 | Requirements for Import and Distribution of Neutron-Irradiated Gems (9/90) | Not applicable to commercial nuclear power plants |
| 90-63 | Management Attention to the Establishment and Maintenance of a Nuclear Criticality Safety Program (10/90) | Not Applicable Procedural/Program issue |
| 90-64 | Potential for Common Mode Failure of High Pressure Safety Injection Pumps or Release of reactor Coolant Outside of Containment During a LOCA (10/90) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|--|--|
| 90-65 | Recent Orifice Plate Problems (10/90) | Not Applicable Installation issue |
| 90-66 | Incomplete Draining and Drying of Shipping Casks (10/90) | Not Applicable Procedural issue |
| 90-67 | Potential Security Equipment Weaknesses (10/90) | Not Applicable Administrative communication |
| 90-68 | Stress Corrosion Cracking of Reactor Coolant Pump Bolts (10/90) | Not Applicable B&W only |
| 90-69 | Adequacy of Emergency and Essential Lighting (10/90) | SSAR Section 9.5.3.2.2 |
| 90-70 | Pump Explosions Involving Ammonium Nitrate (11/90) | Not applicable to commercial nuclear power plants |
| 90-71 | Effective Use of Radiation Safety Committees to Exercise Control Over Medical Use Programs (11/90) | Not applicable to commercial nuclear power plants |
| 90-72 | Testing of Parallel Disc Gate Valves in Europe (11/90) | Not Applicable Part of COL Surveillance issue |
| 90-73 | Corrosion of Valve-To-Torque Tube Keys in Spray Pond Cross Connect Valves (11/90) | Not Applicable Procurement issue |
| 90-74 | Information on Precursors to Severe Accidents (12/90) | Not Applicable Administrative communication |
| 90-75 | Denial of Access to Current Low-Level Radioactive Waste Disposal Facilities (12/90) | Not applicable to commercial nuclear power plants |
| 90-76 | Failure of Turbine Overspeed Trip Mechanism Because of Inadequate Spring Tension (12/90) | Not Applicable AP600 design has no turbine driven pumps |
| 90-77 | Inadvertent Removal of Fuel Assemblies from the Reactor Core (12/90) | Not Applicable Maintenance issue |
| 90-77 S1 | Inadvertent Removal of Fuel Assemblies from the Reactor Core (12/90) | Not Applicable Maintenance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 90-78 | Previously Unidentified Release Path From Boiling Water Reactor Control Rod Hydraulic Units (12/90) | Not Applicable BWR only |
| 90-79 | Failures of Main Steam Isolation Check Valves Resulting in Disc Separation (12/90) | Not Applicable Procurement issue |
| 90-80 | Sand Intrusion Resulting in Two Diesel Generators Becoming Inoperable (12/90) | Not Applicable Maintenance issue |
| 90-81 | Fitness for Duty (12/90) | Not Applicable Administrative communication |
| 90-82 | Requirements for Use of Nuclear Regulatory Commission (NRC) Approved Transport Packages for Shipment of Type A Quantities of Radioactive Materials (12/90) | Not Applicable Administrative communication |
| 91-01 | Supplier of Misrepresented Resistors (1/91) | Not Applicable Procurement issue |
| 91-02 | Brachytherapy Sources Management (1/91) | Not applicable to commercial nuclear power plants |
| 91-03 | Management of wastes Contaminated with Radioactive Materials ("Red Bag" Waste and Ordinary Trash) (1/91) | Not applicable to commercial nuclear power plants |
| 91-04 | Reactor Scram Following Control Rod Withdrawal Associated with Low Power Turbine Testing (1/91) | Not Applicable Administrative/Procurement issue |
| 91-05 | Intergranular Stress Corrosion Cracking in Pressurized Water Reactor Safety Injection Accumulator Nozzles (1/91) | Not Applicable Part of COL Surveillance issue |
| 91-06 | Lock-Up of Emergency Diesel Generator and Load Sequencer Control Circuits Preventing Restart of Tripped Emergency Diesel Generator (1/91) | SSAR Section 8.3.1.1.2.1 |
| 91-07 | Maintenance Deficiency Associated with General Electric Horizontal Custom 8000 Induction Motors (2/91) | Not Applicable Maintenance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 91-08 | Medical examinations for Licensed Operators (2/91) | Not Applicable Administrative communication |
| 91-09 | Counterfeiting of Crane valves (2/91) | Not Applicable Procurement issue |
| 91-10 | Summary of Semiannual Program Performance Reports on Fitness-For-Duty (FFD) in the Nuclear Industry (2/91) | Not Applicable Administrative communication |
| 91-11 | Inadequate Physical Separation and Electrical Isolation of Non-Safety Related Circuits from Reactor Protection System Circuits (2/91) | Not Applicable Installation issue |
| 91-12 | Potential Loss of Net Positive Suction Head (NPSH) of Standby Liquid Control System Pumps (2/91) | Not Applicable BWR only |
| 91-13 | Inadequate Testing of Emergency Diesel Generators (EDG) (3/91) | SSAR Section 16.2 |
| 91-14 | Recent Safety-Related Incidents at Large Irradiators (3/91) | Not applicable to commercial nuclear power plants |
| 91-15 | Incorrect Configuration of Breaker Operating Springs in General Electric AK-Series Metal Clad Circuit Breakers (3/91) | Not Applicable Procurement issue |
| 91-16 | Unmonitored Release Pathways from Slightly Contaminated Recycle and Recirculation Water Systems at a Fuel Facilities (3/91) | Not applicable to commercial nuclear power plants |
| 91-17 | Fire Safety of Temporary Installations or Services (3/91) | Not Applicable Maintenance/Procedural issue |
| 91-18 | High Energy Piping Failures Caused by Wall Thinning (3/91) | Not Applicable Part of COL Surveillance issue |
| 91-19 | Steam Generator Feedwater Distribution Piping Damage (3/91) | Not Applicable CE facilities only |
| 91-20 | Electrical Wire Insulation Degradation Caused Failure in a Safety-Related Motor Control Center (3/91) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|--|
| 91-21 | Inadequate Quality Assurance Program of Vendor Supplying Safety-Related Equipment (3/91) | Not Applicable Administrative/Procurement issue |
| 91-22 | Four Plant Outage Events Involving Loss of AC Power or Coolant Spills (3/91) | Not Applicable Procedural issue |
| 91-23 | Accidental Radiation Exposures to Personnel Due to Industrial Radiography Accessory Equipment Malfunctions (3/91) | Not applicable to commercial nuclear power plants |
| 91-24 | Recent Operating Experience Involving Reactor Operation without a Licensed Reactor Operator or Senior Reactor Operator Present in the Control Room (3/91) | Not Applicable Administrative/Procedural issue |
| 91-25 | Commercial Grade Structural Framing Components Supplied as Nuclear Safety-Related Equipment (4/91) | Not Applicable Procurement issue |
| 91-26 | Potential Nonconservative Errors in the Working Format Hansen-Roach Cross Section Set Provided with the Keno and Scale Codes (4/91) | SSAR Section 4.3.2.6.1 |
| 91-27 | Incorrect Rotation of Positive Displacement Pump (4/91) | Not Applicable Maintenance issue |
| 91-28 | Cracking in Feedwater System Piping (4/91) | Not Applicable Administrative communication |
| 91-29 | Deficiencies Identified During Electrical Distribution System Functional Inspections (4/91) | Not Applicable Procedural issue |
| 91-30 | Inadequate Calibration of Thermoluminescent Dosimeters Utilized to Monitor Extremity Dose at Uranium Processing and Fabrication Facilities (4/91) | Not applicable to commercial nuclear power plants |
| 91-31 | Nonconforming Magnaflux Magnetic Particle (14AM) Prepared Bath (5/91) | Not Applicable Procurement issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 91-32 | Possible Flaws in Certain Piping Systems Fabricated by Associated Piping and Engineering (5/91) | Not Applicable Procurement issue |
| 91-33 | Reactor Safety Information for States During Exercises and Emergencies (5/91) | Not Applicable Administrative communication |
| 91-34 | Potential Problems in Identifying Causes of Emergency Diesel Generator Malfunctions (6/91) | SSAR Section 16.2 |
| 91-35 | Labeling Requirements for Transporting Multi-Hazard Radioactive Materials (6/91) | Not Applicable Administrative communication |
| 91-36 | Nuclear Plant Staff Working Hours (6/91) | Not Applicable Administrative communication |
| 91-37 | Compressed Gas Cylinder Missile Hazard (6/91) | Not Applicable Procedural issue |
| 91-38 | Thermal Stratification in Feedwater piping (6/91) | SSAR Section 10.4.7.2.2 |
| 91-39 | Compliance with 10 CFR Part 21, "Reporting of Defects and Noncompliance" (6/91) | Not applicable to commercial nuclear power plants |
| 91-40 | Contamination of Nonradioactive System Resulting Possibility for Unmonitored Uncontrolled Release to the Environment (6/91) | SSAR Section 11.5 |
| 91-41 | Potential Problems with the Use of Freeze Seals (6/91) | Not Applicable Maintenance issue |
| 91-42 | Plant Outage Events Involving Poor Coordination Between Operations and Maintenance Personnel During Valve Testing and Manipulations (6/91) | Not Applicable Maintenance issue |
| 91-43 | Recent Incidents Involving Rapid Increases in Primary to Secondary Leak Rate (7/91) | SSAR Section 11.5 |
| 91-44 | Improper Control of Chemicals in Nuclear Fuel Fabrication (7/91) | Not applicable to commercial nuclear power plants |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|----------|---|---|
| 91-45 | Possible Malfunction of Westinghouse ARD, BFD, and Nbfd Relays and A200 DC and DPC 250 Magnetic Contactors (7/91) | Not Applicable Procurement issue |
| 91-46 | Degradation of Emergency Diesel Generator Fuel Oil Delivery Systems (7/91) | SSAR Section 16.2 |
| 91-47 | Failure of Thermo-Lag Fire Barrier Material to Pass Fire Endurance Test (8/91) | Not Applicable Procurement issue |
| 91-48 | False Certificates of Conformance Provided by Westinghouse Electric Supply Company for Refurbished Commercial-Grade Circuit Breakers (8/91) | Not Applicable Procurement issue |
| 91-49 | Enforcement of Safety Requirements for Radiographers (8/91) | Not applicable to commercial nuclear power plants |
| 91-50 | A Review of Water Hammer Events After 1985 (8/91) | SSAR Section 10.4.7.2.2 |
| 91-51 | Inadequate Fuse Control Programs (8/91) | Not Applicable Procedural issue |
| 91-52 | Nonconservative Errors in Overtemperature Delta-T Setpoint Caused By Improper Gain Settings (8/91) | Not Applicable Plant specific issue |
| 91-52 S1 | Nonconservative Errors in Overtemperature Delta-T Setpoint Caused By Improper Gain Settings (4/92) | Not Applicable Plant specific issue |
| 91-53 | Failure of Remote Shutdown System Instrumentation Because of Incorrectly Installed Components (9/91) | Not Applicable Installation issue |
| 91-54 | Foreign Experience Regarding Boron Dilution (9/91) | SSAR Section 3.4.14 |
| 91-55 | Failures Caused by an Improperly Adjusted Test Link in a 4.16 KV General Electric Switchgear (9/91) | Not Applicable Procurement issue |
| 91-56 | Potential Radioactive Leakage to Tank Vented to Atmosphere (9/91) | Not Applicable SSAR Section 6.3 |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 91-57 | Operational Experience on Bus Transfers (9/91) | Not Applicable AP600 has no safety-related AC power |
| 91-58 | Dependency of Offset Disc Butterfly Valve's Operation on Orientation with Respect to Flow (9/91) | Not Applicable Installation issue |
| 91-59 | Problems with Access Authorization Programs (9/91) | Not Applicable Administrative communication |
| 91-60 | False Alarms of Alarm Ratemeters Because of Radiofrequency Interference (9/91) | Not applicable to commercial nuclear power plants |
| 91-61 | Preliminary Results of Validation Testing of Motor-Operated Valve Diagnostic Equipment (9/91) | Not Applicable Procurement issue |
| 91-62 | Diesel Engine Damage Caused by Hydraulic Lockup Resulting From Fluid Leakage Into Cylinders (9/91) | SSAR Section 16.2 |
| 91-63 | Natural Gas Hazards at Fort St. Vrain Nuclear Generating Station (10/91) | Not Applicable Site specific issue |
| 91-64 | Site Area Emergency Resulting From a Loss of Non-Class 1E Uninterruptible Power Supplies (10/91) | Not Applicable Procurement issue |
| 91-65 | Emergency Access to Low-Level Radioactive Waste Disposal Facilities (10/91) | Not Applicable Administrative communication |
| 91-66 | (1) Erroneous Data in "Nuclear Safety Guide, TID-7016, Revision 2," (NUREG/CR-0095, ORNL/NUREG/CSD-6 (1978)) and (2) Thermal Scattering Data Limitation in the Cross-Section Sets Provided with the Keno and Scale Codes (10/91) | Not applicable to commercial nuclear power plants/ SSAR Section 4.3.2.6.1 |
| 91-67 | Problems with the Reliable Detection of Intergranular Attack (IGA) of Steam Generator Tubing (10/91) | Not Applicable Part of COL Surveillance issue |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|--|---|
| 91-68 | Careful Planning Significantly Reduces the Potential Adverse Impacts of Loss of Offsite Power Events During Shutdown (10/91) | Not Applicable Administrative communication |
| 91-69 | Errors in Main Steam Line Break Analyses for Determining Containment Parameters (11/91) | SSAR Section 15.1.5 |
| 91-70 | Improper Installation of Instrumentation Models (11/91) | Not Applicable Installation issue |
| 91-71 | Training and Supervision of Individuals Supervised by an Authorized User (11/91) | Not applicable to commercial nuclear power plants |
| 91-72 | Issuance of a Revision to the EPA Manual of Protective Action Guides and Protective Actions for Nuclear Accidents (11/91) | Not Applicable Administrative communication |
| 91-73 | Loss of Shutdown Cooling During Disassembly of High Pressure Safety Injection Systems Check Valve (11/91) | Not Applicable Maintenance/Procedural issue |
| 91-74 | Changes in Pressurizer Safety Valve Setpoints Before Installation (11/91) | Not Applicable Installation issue |
| 91-75 | Static Head Corrections Mistakenly Not Included in Pressure Transmitter Calibration Procedures (11/91) | Not Applicable Procedural issue |
| 91-76 | 10 CFR Parts 21 and 50.55(e) Final Rules (11/91) | Not Applicable Administrative communication |
| 91-77 | Shift Staffing at Nuclear Power Plants (11/91) | Not Applicable Administrative communication |
| 91-78 | Status Indication of Control Power for Circuit Breakers Used in Safety-Related Applications (11/91) | Not Applicable Part of COL |
| 91-79 | Deficiencies in the Procedures for Installing Thermo-Lag Fire Barrier Materials (12/91) | Not Applicable Installation issue |
| 91-80 | Failure of Anchor Head Threads on Post Tensioning System During Surveillance Inspection (12/91) | SSAR Section 3.8.1 |

INFORMATION NOTICES

| NUMBER | TITLE | COMMENT |
|--------|---|---|
| 91-81 | Switchyard Problems that Contribute to Loss of Offsite Power (12/91) | Not Applicable Part of COL |
| 91-82 | Problems with Diaphragms in Safety-Related Tanks (12/91) | Not Applicable Inspection/Maintenance issue |
| 91-83 | Solenoid Operated Valve Failures Resulted in Turbine Overspeed (12/91) | Not Applicable Part of COL Surveillance issue |
| 91-84 | Problems with Criticality Alarm Components/Systems (12/91) | Not applicable to commercial nuclear power plants |
| 91-85 | Potential Failures of Thermostatic Control Valves for Diesel Generator Jacket Cooling Water (12/91) | SSAR Section 16.2 |
| 91-86 | New Reporting Requirements for Contamination Events at Medical Facilities (10 CFR 30.50) | Not applicable to commercial nuclear power plants |
| 91-87 | Hydrogen Embrittlement of Raychem Cryfit Couplings (12/91) | Not Applicable Procurement issue |

AEOD REPORTS

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/E001 | Crystal River Nuclear Power Plant Decay Heat Closed Cycle Cooling Water Pumps/DCP-1A and DCP-B (3/80) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E002 | BWR Jet Pump Integrity (5/80) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E003 | Comparison of Reactor Coolant Pump Events Contained in LERs, NPRDS, RECON, and Plant records (6/80) | Not Applicable Administrative communication |
| AEOD/E004 | Data Summaries of Licensee Event Reports of Pumps at U.S. Commercial Nuclear Power Plants, 1/1/72 - 4/30/78 (7/80) | Not Applicable Administrative communication |
| AEOD/E005 | Operational Restrictions for Class 1E 120 V AC Vital Instrument Buses (7/80) | SSAR Section 8.3.2 |
| AEOD/E006 | Loss of Residual Heat Removal at Beaver Valley, LER 80-031 (8/80) | SSAR Section 16.2 |
| AEOD/E007 | Potential for Unacceptable Interaction Between the Control Rod Drive System and Nonessential Air System at Browns Ferry (8/80) | Not Applicable BWR only |
| AEOD/E008 | Operational Restrictions During Surveillance Testing of Emergency Diesel Generators (8/80) | SSAR Section 16.2 |
| AEOD/E009 | Failures of Containment Isolation Valves at Zion (8/80) | Not Applicable Procurement issue |
| AEOD/E010 | Tie Breaker Between Redundant Class 1E Buses - Point Beach Units 1 and 2 (8/80) | Not Applicable AP600 design has no Class 1E AC power |
| AEOD/E011 | Concerns Related to the Integrity of a Polymer Coating for Surfaces Inside Containment (8/80) | Not Applicable Procurement issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|--|
| AEOD/E012 | Salem Unit 1 - Solenoid Valve of Containment Fan Coil Unit Service Water Flow Control Valve (9/80) | Not Applicable Procurement issue |
| AEOD/E013 | Excessive Main Feedwater Transient (9/80) | SSAR Section 7.3 |
| AEOD/E014 | Transient at Crystal River Unit 3 - 9/30/80 (10/80) | Not Applicable Maintenance issue |
| AEOD/E015 | 1/3/77, Quad Cities Unit 1 Loss-of-Air Event and Its Effect on Scram Capability (10/80) | Not Applicable BWR only |
| AEOD/E016 | Flow Blockage in Essential Equipment at ANO Caused by Corbicula sp. (Asiatic Clams) (10/80) | Not Applicable Site specific issue |
| AEOD/E017 | Engineering Evaluation of Steam Generator Overfill (10/80) | SSAR Section 7.3 |
| AEOD/E018 | Potential Failure of BWR Backup Scram (Mode Switch in Shutdown) Capability (12/80) | Not Applicable BWR only |
| AEOD/E019 | Davis Besse Unit 1 - Emergency Core Cooling System Actuation During Hot Shutdown on 12/5/80) | SSAR Section 7.1 |
| AEOD/E020 | Internal Appurtances in LWRs (12/80) | Not Applicable Administrative communication |
| AEOD/E101 | Degradation of Internal Appurtances in LWR Piping (1/81) | Not Applicable No issue/No action |
| AEOD/E102 | Sequoyah Unit 1 Loss of Annunciation (1/81) | Not Applicable Procurement issue |
| AEOD/E103 | Davis Besse Nuclear Power Station Unit 1 - Engineered Safety Features Actuation System (ESFAS) (2/81) | Not Applicable Plant specific issue |
| AEOD/E104 | Engineering Evaluation of Feedwater Transient and System Pipe Break at Turkey Point 3 (3/81) | Not Applicable Procurement issue |
| AEOD/E105 | Water Hammer During Restart of Residual Heat Removal Pumps (3/81) | Not Applicable BWR only |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/E106 | Water Hammer in the Steam Condensing Mode of the Residual Heat Removal System Operation (3/81) | Not Applicable BWR only |
| AEOD/E107 | Peach Bottom Occurrence on 2/25/81 (4/81) | Not Applicable Procedural issue |
| AEOD/E108 | Hatch Units 1 and 2 - Alternate Offsite Source Interlock With Emergency Diesel Generators (4/81) | Not Applicable Plant specific issue |
| AEOD/E109 | Potential Common Mode Failure of Diesel Generators (4/81) | SSAR Section 16.2 |
| AEOD/E110 | Requirements of the Preferred of Offsite Power System (4/81) | SSAR Section 8.1 and 8.2 |
| AEOD/E111 | Evaluation of High Pressure Safety Injection Pump Operability without Service Water (5/81) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E112 | Inoperability of Instrumentation Due to Extreme Cold Weather (6/81) | Not Applicable Part of COL |
| AEOD/E113 | Deliberate Pump Trip at Browns Ferry Unit 2 on 4/6/81 (6/81) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E114 | Control Systems Failures That Could Cause or Exacerbate Nuclear Power Plant Accidents (6/81) | Control System Failure Analysis |
| AEOD/E115 | Additional Information on Events at TMI-2 During Preoperational Testing (September 5-12, 1977) (7/81) | Not Applicable Plant specific issue |
| AEOD/E116 | Failure of B Phase Main Transformer and Subsequent Fire in the Transformer Area - North Anna Unit 2 (7/81) | Not Applicable Procurement issue |
| AEOD/E117 | Events at TMI-2 During Preoperational Testing (7/81) | Not Applicable Procedural issue |
| AEOD/E118 | Setpoint Drift Occurrences for the Barton Model 288 Instrument (7/81) | Not Applicable Procurement/Procedural issue |
| AEOD/E119 | Loss of Residual Heat Removal Capability at Brunswick Units 1 and 2 (7/81) | SSAR Section 16.2 |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|---|
| AEOD/E120 | Ignition of Gaseous Waste Decay Tank at San Onofre Unit 1 - 7/17/81 (8/81) | Not Applicable Part of COL Maintenance/Surveillance issue |
| AEOD/E121 | Crystal River 3 Engineered Safeguards Relay Failures (8/81) | Not Applicable No issue |
| AEOD/E122 | AEOD Concern Regarding Inadvertent Opening of Atmospheric Dump Valves on B&W Plants During Loss of Integrated Control System Nonnuclear/Instrumentation (9/81) | Not Applicable B&W facilities only |
| AEOD/E123 | Immediate Action Memo - Common Cause Failure Potential at Rancho Seco - Desiccant Contamination of Air Lines (9/81) | Not Applicable Part of COL Maintenance/Surveillance issue |
| AEOD/E124 | Review of Information on Purge Valves (9/81) | Not Applicable Procurement issue |
| AEOD/E125 | Engineering Evaluation Report on Shutdown Cooling System Heat Exchanger Failure at Oyster Creek, August 1981 (10/81) | Not Applicable No issue |
| AEOD/E126 | Event Sequences Not Considered in the Design of Emergency Bus Control Logic (10/81) | SSAR Section 8.3.1.1.2.1 |
| AEOD/E127 | Pressure Boundary Degradation Due to Pump Seal failure at Arkansas Nuclear One (10/81) | Not Applicable AP600 design has no seals on RCPs SSAR Section 1.2.1.2.3 |
| AEOD/E128 | Inoperable Teledyne Solenoid Valves (11/81) | Not Applicable Procurement/Maintenance issue |
| AEOD/E129 | Brunswick unit 2 Diesel Generator Jacket Water Temperature Control Valve and Manual Bypass Valve (12/81) | SSAR Section 16.2 |
| AEOD/E130 | Davis Besse LER 79-062 on Auxiliary Feedwater System Pressure Switches (12/81) | Not Applicable Procurement issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/E131 | High Circulating Current Associated With Inverter Output Due to Lack of Circuit Tuning (12/81) | Not Applicable Maintenance issue |
| AEOD/E132 | Abnormal Wear Encountered on Alloyco Swing Check Valves Installed in the Low Pressure Safety Injection System at Palisades (12/81) | Not Applicable Procurement issue |
| AEOD/E133 | Inadequacies in Periodic Testing of Combustion Engineering Reactor Protection System (12/81) | Not Applicable Procedural issue |
| AEOD/E201 | Methodology for Vital Area Determination (1/82) | SSAR Section 13.6 |
| AEOD/E202 | Loss of High Pressure Injection Lube Oil Cooling at Rancho Seco (1/82) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E203 | Inadvertent Isolation of Containment Fan Units at Salem Unit 1 (1/82) | Not Applicable Plant specific issue |
| AEOD/E204 | Effects of Fire Protection System Actuation on Safety-Related Equipment (1/82) | SSAR Section 9.5.1 |
| AEOD/E205 | Potential Consequences of Heavy Load Drop Accidents in LWRs (2/82) | USI A-36 SSAR Section 1.9.4 |
| AEOD/E206 | Load Reduction Transient on January 14, 1982, at Salem Unit 2 (2/82) | Not Applicable Administrative issue |
| AEOD/E207 | LER 50-336/81-26: Investigation of the Relative Frequency of Valve Overtravel Anomalies That Could Result in a Positive Centrifugal Pump Runout Exceeding Net Positive Suction Head (2/82) | Not Applicable AP600 design has no CCPs in SI system |
| AEOD/E208 | An Observed Difference in Lift Setpoint for Steam Generator and Pressurizer Safety Valves (2/82) | SSAR Chapter 16 Section 3.4.6, 3.7.1 |
| AEOD/E209 | Generator Rotor Retaining Ring as a Potential Missile (Incident at Barseback Unit 1 on 4/13/79) (2/82) | SSAR Section 10.2.2 |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|---|
| AEOD/E210 | Inadequate Switchgear Cooling at Beaver Valley Unit 1 (2/82) | SSAR Section 3.11.4 |
| AEOD/E211 | Repetitive Failures of Emergency Feedwater Flow Valves at Arkansas Unit 2 Because of Valve Operator Hydraulic Problems (2/82) | Not Applicable Maintenance issue |
| AEOD/E212 | Spurious Trip of the Generator Lockout Relay Associated With a Diesel Generator Unit (2/82) | SSAR Section 8.3.1.1.2.1 |
| AEOD/E213 | Trip of Two Inservice Auxiliary Feedwater Pumps From Low Suction at Zion Unit 2 on December 11, 1981 (2/82) | SSAR Section 10.4.9 |
| AEOD/E214 | Duane Arnold Loss of River Water System Loop (3/82) | SSAR Section 16.2 |
| AEOD/E215 | Engineering Evaluation of the Salt Service Water System Flow Blockage at the Pilgrim Nuclear Station by Blue Mussels (3/82) | SSAR Section 16.2 |
| AEOD/E216 | A Recently Evaluated Preoperational Test Precursor of the TMI-2 Accident (3/82) | Not Applicable Administrative communication |
| AEOD/E217 | Scram Pilot Solenoid Valve Failures Due to Low Voltage - Grand Gulf Unit 1 (3/82) | Not Applicable Plant specific issue |
| AEOD/E218 | Potential for Air Binding or Degraded Performance of BWR Residual Heat Removal System Pumps During the Recirculation Phase of a Loss-of-Coolant Accident (3/82) | Not Applicable BWR only |
| AEOD/E219 | Proposed Circular: Contamination of Air Serving Safety Related Equipment (4/82) | Not Applicable Part of COL Surveillance issue |
| AEOD/E220 | Water in the Fuel Oil Tank at Surry Power Station Unit 2 (4/82) | Not Applicable Plant Specific issue |
| AEOD/E221 | Indian Point Unit 2 Flooding Event (4/82) | SSAR Section 3.4 |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|--|
| AEOD/E222 | Loss of Reserve Station Service Transformer "B" on 1/18/82 , at Surry Unit 2 (5/82) | Not Applicable Plant specific issue |
| AEOD/E223 | Inadvertent Loss-of-Coolant Events at Sequoyah Units 1 and 2 (5/82) | Not Applicable Administrative/Procedural issue |
| AEOD/E224 | Generic Concerns Associated With the Ginna Steam Generator Tube Rupture Event (5/82) | SSAR Section 15.6.3 |
| AEOD/E225 | Degradation of BWR Scram Pilot Solenoid Valves Due to Abnormal Power Supply Voltage (6/82) | Not Applicable Installation issue |
| AEOD/E226 | Inoperability of Instrumentation Due to Extreme Cold Weather (6/82) | Not Applicable Part of COL |
| AEOD/E227 | Failure of Engineered Safety Features Manual Initiation Pushbutton Switches (6/82) | Not Applicable Procurement issue |
| AEOD/E228 | Repetitive Overspeed Trips of the Steam Driven Emergency Feedwater Pump on Initial Start at Arkansas Nuclear One, Unit 2 (6/82) | Not Applicable AP600 design has no turbine driven pumps |
| AEOD/E229 | Potential for Flooding in Control Room at San Onofre Units 2 and 3 (6/82) | SSAR Section 9.5.1.2.1.6 |
| AEOD/E230 | Water in the Fuel Oil Tank at Surry Power Station, Unit 2 - Additional Information (7/82) | Not Applicable Plant specific issue |
| AEOD/E231 | Millstone Unit 2 Loss of Shutdown Cooling Due to Trip of Low Pressure Safety Injection Pump (7/82) | Not Applicable Plant specific issue |
| AEOD/E232 | Potential Deficiency in the Sigma Lumigraph Indicators Model Number 9270 (7/82) | Not Applicable Procurement issue |
| AEOD/E233 | Carbon Dioxide Systems Used for Fire Protection in or Adjacent to Critical Areas (7/82) | SSAR Section 9.5.1.2.1.4 |
| AEOD/E234 | Failure in a Section of 4 KV Bus Cable Manufactured by Okonite (8/82) | Not Applicable No issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|--|
| AEOD/E235 | Wiring Error in Handswitch for Solenoid Control Valves Associated with High Pressure Coolant Injection System Steam Condensing Mode Pressure Control Valve at Duane Arnold (8/82) | Not Applicable Procurement issue |
| AEOD/E236 | Brunswick Steam Electric Plant Unit 2 Loss of Residual Heat Removal Service Water on 1/16/82 (8/82) | Not Applicable Plant specific issue |
| AEOD/E237 | Power Operated Relief Valve Failure at Robinson (8/82) | Not Applicable Maintenance issue |
| AEOD/E238 | Water in the Lube Oil in Safety Injection Pump IA-A at Sequoyah - LER 81-076 (8/82) | Not Applicable Maintenance issue |
| AEOD/E239 | Main Steam Isolation Valve Closures and Pressurizer Safety Valve Actuations at St. Lucie Unit 1 on 12/19/81 (9/82) | Not Applicable Maintenance issue |
| AEOD/E240 | Preliminary Account of Events Associated With a Reactor Trip at Hatch on 8/25/82 (9/82) | Not Applicable Procurement issue |
| AEOD/E241 | Emergency Diesel Generator System Problems at Fitzpatrick (10/82) | SSAR Section 16.2 |
| AEOD/E242 | Fuel Assembly Degradation While in the Spent Fuel Storage Pool (10/82) | Not Applicable Maintenance issue |
| AEOD/E243 | Plant Trip Followed by a Safety Injection Due to Loss of "A" Cooling Tower Pump at Palisades on 2/4/82 (10/82) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E244 | Loss of Residual Heat Removal System Event at Pilgrim Nuclear power Station on 12/21/81 (10/82) | Not Applicable Installation/Maintenance issue |
| AEOD/E245 | Failure of Westinghouse Type SC-1 No. 1876-072 Relays (10/82) | Not Applicable Procurement issue |
| AEOD/E246 | Events Involving Loss of Electrical Inverters Including Attendant Inverters to Vital Instrument Buses (10/82) | Not Applicable Maintenance issue |
| AEOD/E247 | Engineering Evaluation of Turbine/Reactor Trip at Rancho Seco on 8/7/81 (10/82) | Not Applicable No issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------------|--|---|
| AEOD/E248 | Engineering Evaluation Report on McGuire Overpressurization Event on 8/27/81 (11/82) | Not Applicable AP600 design has no turbine driven pumps |
| AEOD/E249 | Engineering Evaluation Memorandum - Licensee Reporting of the Turbine/Reactor Trip at Rancho Seco on 8/7/81 (11/82) | Not Applicable Administrative issue |
| AEOD/E250 | Quad Cities Unit 2 Loss of Auxiliary Electrical Power Event on 6/22/82 (11/82) | Not Applicable Plant specific issue |
| AEOD/E251 | Salem Unit 2 Loss of Vital Bus No. 2A (11/82) | Not Applicable Plant specific issue |
| AEOD/E252 | No Title (no date) | Not Issued |
| AEOD/E253 | Potential Control Logic Problem Resulting in Inoperable Auto-Start of Diesel Generator Units Under the Conditions of Loss-of-Coolant Accident and Loss of Station Power (LOSP) (11/82) | Not Applicable Procedural issue |
| AEOD/E254 | Review of Prairie Island Unit 1 LER 82-015-01T on Diesel Generator Operability (11/82) | Not Applicable Administrative/Procedural issue |
| AEOD/E255 | Failure of the Vent Line on the Common Discharge of the Two Motor Driven Auxiliary Feedwater Pumps at San Onofre Unit 2 From an Improper Valve Lineup (11/82) | Not Applicable AP600 design has no safety-related AFW or similar safety-related system |
| AEOD/E256 | Loss of Shutdown Cooling and Subsequent Boron Dilution at San Onofre Unit 2 (11/82) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E257 | Insufficient Net Positive Suction Head for Charging Pump Service Water Pumps at Surry Nuclear Power Station (12/82) | SSAR Section 9.2.1 |
| AEOD/E301 | Fuel Degradation at Westinghouse Plants (1/83) | Not Applicable Plant specific issue |
| AEOD/E301 R1 | Update to AEOD/E301 (Fuel Degradation at Westinghouse Plants) (4/83) | Not Applicable Plant specific issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|---|
| AEOD/E302 | Potential Loss of Service Water Flow Resulting From a Loss of Instrument Air (1/83) | SSAR Section 16.2 |
| AEOD/E303 | Valve Flooding Event at Surry (2/83) | Not Applicable Plant Specific issue |
| AEOD/E304 | Investigation of Backflow Protection in Common Equipment and Floor Drain Systems to Prevent Flooding of Vital Equipment in Safety-Related Compartments (3/83) | SSAR Section 3.4.1.2.2 |
| AEOD/E305 | Inoperable Motor Operated Valve Assemblies Due to Premature Degradation of Motors and/or Improper Limit Switch/Torque Switch Adjustment (4/83) | Not Applicable Procedural/Maintenance issue |
| AEOD/E306 | Cooldown During Loss of Control Room Test at McGuire Unit 1 (4/83) | Not Applicable Procedural issue |
| AEOD/E307 | Degradation of Safety-Related Batteries Due to Cracking of Battery Cell Cases and/or Other Possible Aging-Related Mechanisms (4/83) | Not Applicable Procurement issue |
| AEOD/E308 | Cracks and Leaks in Small-Diameter Piping (4/83) | Not Applicable Installation issue |
| AEOD/E309 | The Potential for Water Hammer During the Restart of Residual Heat Removal Pumps at BWR Nuclear Power Plants (4/83) | Not Applicable BWR only |
| AEOD/E310 | Loss of Shutdown Cooling and Subsequent Boron Dilution at San Onofre Unit 2 (4/83) | Not Applicable Training issue |
| AEOD/E311 | Loss of Salt Water Flow to the Service Water Heat Exchangers for 23 Minutes at Calvert Cliffs Unit 2 (4/83) | SSAR Section 16.2 |
| AEOD/E312 | Operability of Target Rock Safety Relief Valves in the Safety Mode with Pilot Valve Leakage (5/83) | Not Applicable Procurement/Maintenance issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/E313 | Potential Contamination of the Spent Fuel Pool and Primary Reactor System (6/83) | Not Applicable Part of COL |
| AEOD/E314 | Loss of All Three Charging Pumps Due to Empty Common Reference Leg in the Liquid Level Transducers for the Volume Control Tank at St. Lucie 1 (6/83) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E315 | Misuse of Valve resulting in Vibration and Damage to the Valve Assembly and Pipe Supports (7/83) | Not Applicable BWR only |
| AEOD/E316 | Frozen Ice Condenser Intermediate Deck Doors (7/83) | Not Applicable Ice Condenser only |
| AEOD/E317 | Loss of High Pressure Injection System (8/83) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E318 | Biofouling at Salem Units 1 & 2 (8/83) | SSAR Section 16.2 |
| AEOD/E319 | Loss of Drywell Torus Pressure Differential During Residual Heat Removal Pump Flow Testing at Cooper (9/83) | Not Applicable BWR only |
| AEOD/E320 | Power Operated Relief Valve Actuation Resulting in Safety Injection Actuation at Calvert Cliffs (9/83) | Not Applicable CE facilities only |
| AEOD/E321 | Three Similar Events of a Loss of Shutdown Cooling Flow at Combustion Engineering Plants (9/83) | Not Applicable CE facilities only |
| AEOD/E322 | Damage to Vacuum Breaker Valves as a Result of Relief Valve Lifting at Peach Bottom Unit 2 (9/83) | Not Applicable BWR only |
| AEOD/E323 | Load Reduction Transient at Salem Unit 2 on 1/14/82 (9/83) | Not Applicable Procedural/Training issue |
| AEOD/E324 | Review of Events Involving Failures of Power Supply in Instrumentation and Control Systems (9/83) | Not Applicable Procurement issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/E325 | Vapor Binding of Auxiliary Feedwater Pumps at Robinson Unit 2 (11/83) | Not Applicable AP600 design has no safety-related AFW system or similar safety-related system |
| AEOD/E326 | Steam Voiding of Oconee Unit 3 on 6/13/75: A Precursor Event to the TMI-2 Accident (11/83) | Not Applicable Administrative communication |
| AEOD/E327 | Gas Releases From Waste Gas Disposal System (11/83) | SSAR Section 11.3 |
| AEOD/E401 | Temporary Loss of All AC Power Due to Relay Failure in Diesel Generator Load Shedding Circuitry at Fort St. Vrain (1/84) | SSAR Section 16.2 |
| AEOD/E402 | Water Hammer in Boiling Water Reactor High-Pressure Coolant Injection Systems (1/84) | Not Applicable BWR only |
| AEOD/E403 | Deficiency in Automatic Switch Company (ASCO) Spare Parts Kits for Scram Pilot Solenoid Valves (1/84) | Not Applicable Procurement issue |
| AEOD/E404 | Failures in the Upper Head Injection System (2/84) | Not Applicable Procurement issue |
| AEOD/E405 | Common Mode Failure of HPCI Steam Flow Isolation Capability at Browns Ferry (3/84) | Not Applicable BWR only |
| AEOD/E406 | Mechanical Snubber Failure (3/84) | Not Applicable Procurement issue |
| AEOD/E407 | Initiation and Indication Circuitry for High Pressure Coolant Injection Systems (3/84) | Not Applicable BWR only |
| AEOD/E408 | Reversed Differential Pressure Instrument Sensing Lines (4/84) | Not Applicable Maintenance/Installation issue |
| AEOD/E409 | Operating Experience Involving Air in Instrument Sensing Lines (5/84) | Not Applicable No issue/no action |
| AEOD/E410 | Operational Experience Involving Standby Gas Treatment Systems That Illustrate Potential Common Cause Failure or Degradation Mechanisms (5/84) | Not Applicable Procurement issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|---|
| AEOD/E411 | Failure of Anticavitation Device in Residual Heat Removal Service Water Heat Exchanger Outlet Valve (5/84) | Not Applicable Procurement issue |
| AEOD/E412 | Adverse System Interaction with Domestic Water Systems (5/84) | SSAR Section 3.4.1.2.2.2 |
| AEOD/E413 | Natural Circulation in Pressurized Water Reactors (5/84) | Not Applicable No issue/no action |
| AEOD/E414 | Stuck Open Isolation Check Valve on the Residual Heat Removal System at Hatch Unit 2 (5/84) | Not Applicable Plant specific event |
| AEOD/E415 | Overcooling Transient (6/84) | Not Applicable Administrative No issue/no action |
| AEOD/E416 | Erosion in Nuclear Power Plants (6/84) | Not Applicable Part of COL Surveillance issue |
| AEOD/E417 | Loosening of Flange Bolts on Residual Heat Removal Heat Exchanger Leading to Primary to Secondary Side Leakage (7/84) | Not Applicable BWR only |
| AEOD/E418 | Feedwater Transients During Startup at Westinghouse Plants (7/84) | SSAR Section 7.7 |
| AEOD/E419 | Failures of Fischer-Porter Transmitters Used in Safety-Related Systems (7/84) | Not Applicable Procurement issue |
| AEOD/E420 | Operational Experiences Involving Shorted Lamp Sockets of Indication Lights (8/84) | Not Applicable Maintenance issue |
| AEOD/E421 | Loss of Pressurizer Heaters During Precore Hot Functional Testing (8/84) | Not Applicable No issue |
| AEOD/E422 | High Pressure Coolant Injection Performance at Hatch Units 1 and 2 (8/84) | Not Applicable BWR only |
| AEOD/E423 | Failures of Large Hydraulic Snubbers to Lock Up (9/84) | Not Applicable Part of COL Procurement/Surveillance issue |
| AEOD/E424 | Failure of Anchor Bolt on Diesel Generator Day Tank at Davis-Besse Unit (10/84) | SSAR Section 16.2 |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|--|
| AEOD/E425 | High Pressure Coolant Injection system Lockout at Vermont Yankee (10/84) | Not Applicable Procedural issue |
| AEOD/E426 | Single Failure Vulnerability of Power-Operated Relief Valve Actuation Circuitry for Low Temperature Overpressure Protection (10/84) | SSAR Section 3.9.1.1.2.6 |
| AEOD/E427 | Licensee Event Reports That Address Situation That Potentially Could Result in Overloading Electrical Equipment in the Emergency Power System or Prevent Operation of the Onsite Power System Sequencer (11/84) | SSAR Section 8.3.1.1.2.1 |
| AEOD/E501 | Motor Operated Valve Failures Due to Hammering Problem (1/85) | SSAR Section 10.4.7.2.2 Procurement issue |
| AEOD/E502 | Failure of Residual Heat Removal Suppression Pool Cooling Valve to Operate (1/85) | See Information Notice 83-70 |
| AEOD/E503 | Partial Failures of Control Rod Systems to Scram (3/85) | SSAR Section 2 3.9.4.4, 4.6.3 |
| AEOD/E504 | Loss or Actuation of Various Safety-Related Equipment Due to Removal Of Fuses or Opening of Circuit Breakers (3/85) | Not Applicable Maintenance Procedural issue |
| AEOD/E505 | Service Water System Air Release Valve Failures (3/85) | SSAR Section 16.2 |
| AEOD/E506 | Valve Stem Suscepability to Intragranular Stress Corrosion Cracking Due To Improper Heat treatment (5/85) | See Information Notice 85-59 |
| AEOD/E507 | Electrical Interaction Between Units During Loss of Offsite Power Event of August 21, 1984 at McGuire Units 1 and 2 (5/85) | Not Applicable Single unit design |
| AEOD/E508 | Nuclear Plant Operating Experience Involving Safety System Disturbances Due to Bumped Electromechanical Components (5/85) | Not Applicable Procurement issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|---|
| AEOD/E509 | Salem Unit 2 Depressurization Event (7/85) | Not Applicable Procedural issue |
| AEOD/E510 | Disabling of a Shared Diesel Generator Set Due to Electrical Power Supply Arrangement for Support Auxiliaries (7/85) | SSAR 8.3.1.1.2.1 |
| AEOD/E511 | Closure of Emergency Core Cooling System Minimum Flow Valves (8/85) | Not Applicable BWR only |
| AEOD/E512 | Failures of Safety-Related Pump Due to Debris (9/85) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E513 | High Pressure Core Spray System Relief Valve Failures (9/85) | Not Applicable BWR only |
| AEOD/E514 | Core Damage Precursor Event at Trojan (10/85) | Not Applicable Plant Specific issue |
| AEOD/E515 | Inadvertent Actuation of Safety System Due to Cross Talk (12/85) | Not Applicable Plant Specific issue |
| AEOD/E601 | Deficient Operator Actions Following Dual Function Valve Failures (1/86) | Not Applicable Procedural issue |
| AEOD/E602 | Unexpected Criticality Due to Incorrect Calculation and Failure to Follow Procedures (1/86) | Not Applicable Procedural issue |
| AEOD/E603 | Delayed Access to Safety-Related Areas During Plant Operation (2/86) | Not Applicable Procedural issue |
| AEOD/E604 | Spurious System Isolations Due to The Panalarm Model 86 Thermocouple Monitor (3/86) | Not Applicable Procurement issue |
| AEOD/E605 | Lightning Events at Nuclear Power Plants (4/86) | Not Applicable No conclusions |
| AEOD/E606 | Loss of Safety Injection Capability at Indian Point Unit 2 (5/86) | Not Applicable AP600 design has no BIT or safety related pumps |
| AEOD/E607 | Degradation of Loss of Charging System With Swing Pump Designs (7/86) | Not Applicable AP600 design has no safety-related pumps |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|---|
| AEOD/E608 | Reexamination of Water Hammer Occurrences (7/86) | Not Applicable No conclusions SSAR Section 10.4.7.2.2 |
| AEOD/E609 | Inadvertent Draining of Reactor Vessel During Shutdown Cooling Operation (8/86) | Not Applicable BWR only |
| AEOD/E610 | Loss of Low Pressure Coolant Injection Loop Selection Logic at Millstone Unit 1 (8/86) | Not Applicable BWR only |
| AEOD/E611 | Deficiencies in Seismic Anchorage for Electrical and Control Panels (10/86) | Not Applicable Installation issue |
| AEOD/E612 | Emergency Diesel Generator Component Failures Due to Vibration (12/86) | SSAR Section 16.2 |
| AEOD/E613 | Localized Rod Cluster Control Assembly Wear at PWR Plants (12/86) | Not Applicable Part of COL Surveillance issue |
| AEOD/E701 | Potential Containment Airlock Window Failure Due to Radiation (1/87) | Not Applicable Nonrepresentative Test |
| AEOD/E702 | MOV Failure Due to Hydraulic Lockup From Excessive Grease in Spring Pack (3/87) | Not Applicable Procurement/Maintenance issue |
| AEOD/E703 | Loss of Offsite Power Due to Unneeded Actuation of Startup Transformer Protection Differential Relay (3/87) | Not Applicable No issue/no action |
| AEOD/E704 | Discharge of Primary Coolant Outside of Containment at PWRs While on RHR Cooling (3/87) | Not Applicable Procedural issue |
| AEOD/E705 | RWCU System Automatic Isolation and Safety Considerations (3/87) | Not Applicable BWR only |
| AEOD/E706 | Inadequate Mechanical Blocking of Valves (3/87) | Not Applicable Procedural issue |
| AEOD/E707 | Design and Construction Problems at Operating Nuclear Plants (3/87) | Not Applicable Installation/Procurement issue |
| AEOD/E708 | Depressurization of Reactor Coolant Systems at PWRs (8/87) | Not Applicable Procedural issue |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|--|
| AEOD/E709 | Auxiliary Feedwater Pump Trips Due to Low Suction Pressure (8/87) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E710 | Inadequate NPSH in Low-Pressure Safety Systems in PWRs (10/87) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E801 | BWR Overfill Events Resulting in Steam Line Flooding (4/88) | Not Applicable BWR only |
| AEOD/E802 | Design and Operating Procedures for Control Room Emergency Response Systems (5/88) | SSAR Section 9.4.1 |
| AEOD/E803 | Inadequate NPSH in High Pressure Safety Injection Systems in PWRs (8/88) | AP600 design has no safety-related HHSI pumps |
| AEOD/E804 | Reliability of Recirculation Pump Breaker During an ATWS (8/88) | Not Applicable BWR only |
| AEOD/E805 | Potential LOCA Due to Energized Uncovered Pressurizer Heaters (9/88) | Not Applicable Procedural issue |
| AEOD/E806 | Loss of Decay Heat Removal Due to Rapid Refueling Cavity Pumpdown (10/88) | SSAR Section 5.4.14 |
| AEOD/E807 | Engineering Evaluation Report - Pump Damage Due to Low Flow Cavitation (10/88) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E808 | Operational Experience Review of Potential Large Openings in Containment (12/88) | SSAR Chapter 16, Section 3.6 |
| AEOD/E901 | Problems with Oils, Greases, Solvents, and Other Chemical Materials (2/89) | Not Applicable Maintenance issue |
| AEOD/E902 | Fires and Explosive Mixtures Resulted From Introduction of Hydrogen Into Plant Air Systems (7/89) | Not Applicable Procedural issue |
| AEOD/E903 | No Title (no date) | Not Issued |
| AEOD/E904 | On Demand Malfunctions of HPCI and RCIC (4/89) | Not Applicable BWR only |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-------------|---|--|
| AEOD/E905 | Electrical Bus Bar Failures (6/89) | Not Applicable Procurement issue |
| AEOD/E906 | Failure of Steam Generator Isolation Check Valve (8/89) | Not Applicable SSAR Section 10.3.2.2.4 |
| AEOD/E907 | Diversion of Seal Cooler Flow for RHR Pumps (8/89) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E908 | Excessive Valve Body Erosion at Brunswick (10/89) | Not Applicable No conclusions |
| AEOD/E909 | Operator Actions During Operational Events (12/89) | SSAR Chapter 18 |
| AEOD/E910 | Potential for Gas Binding of High Head Safety Injection Pumps Resulting From Inservice Testing of VCT Outlet Isolation Valves (12/89) | Not Applicable AP600 design has no safety-related HHSI or similar pumps |
| AEOD/E90-01 | Failures of Electrical Supply and Power Generation Equipment Which Disrupted Plant Function at Nuclear Power Plants (2/90) | Not Applicable Maintenance/Procedural issue |
| AEOD/E90-02 | Crosby Low Pressure Relief Valves (2/90) | Not Applicable Procurement issue |
| AEOD/E90-03 | Overpressurization of Auxiliary Feedwater Systems (5/90) | Not Applicable AP600 design has no turbine driven pumps |
| AEOD/E90-04 | Swelling and Cracking in Hafnium Control Rods (4/90) | Not Applicable AP600 design has no Hafnium control rods |
| AEOD/E90-05 | Operational Experience in Bus Transfer (5/90) | Not Applicable No issue |
| AEOD/E90-06 | Potential for Residual Heat Removal System Pump Damage (7/90) | Not Applicable AP600 design has no safety-related pumps |
| AEOD/E90-07 | Effects of Internal Flooding of Nuclear Power Plants on Safety Equipment (7/90) | SSAR Section 3.4.1 |

AEOD ENGINEERING EVALUATION REPORTS

| NUMBER | TITLE | COMMENT |
|-------------|---|--|
| AEOD/E90-08 | Low Temperature Overpressure Protection: Testing PORVs With the Pneumatic Supply (9/90) | SSAR Section 3.9.1.1.2.6 |
| AEOD/E90-09 | Additional Factors Affecting the Lift Setpoint of Pressurizer Safety Valves (10/90) | SSAR Section 3.6.4 Surveillance issue |
| AEOD/E90-10 | Evaluation of Boiling Water Reactor Mode Switch Events (12/90) | Not Applicable BWR only |
| AEOD/E91-01 | A Review of Water Hammer Events After 1985 (2/91) | Not Applicable Administrative communication |

AEOD CASE STUDY REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|---|
| AEOD/C001 | Report on the Browns Ferry Unit 3 Partial Failure to Scram Event on 6/28/80 (7/80) | Not Applicable BWR only |
| AEOD/C002 | Report on the Interim Equipment and Procedures at Browns Ferry Unit 3 to Detect Water in the Scram Discharge Volume (9/80) | Not Applicable BWR only |
| AEOD/C003 | Report on Loss-of-Offsite-Power Event at Arkansas Nuclear One, Units 1 and 2 (10/80) | Not Applicable Plant specific issue |
| AEOD/C004 | AEOD Actions Concerning the Crystal River Unit 3 Loss of Nonnuclear Instrumentation and Integrated Control System Power on 2/26/80 (11/80) | Not Applicable B&W facilities only |
| AEOD/C005 | AEOD Observations and Recommendations Concerning the Problem of Steam Generator Overfill and Combined Primary and Secondary Side Blowdown (12/80) | SSAR Section 7.3 |
| AEOD/C101 | Report on the St. Lucie Unit 1 Natural Circulation Cooldown on 6/11/80 (3/81) | Not Applicable Procedural issue |
| AEOD/C102 | Robinson Reactor Coolant System Leak on 1/29/81 (3/81) | Not Applicable Plant Specific issue |
| AEOD/C103 | AEOD Safety Concerns with Pipe Breaks in the BWR Scram System (3/81) | Not Applicable BWR only |
| AEOD/C104 | Millstone Unit 2 Loss of 125 V DC Bus Event on 1/2/81 (4/81) | Not Applicable Procedural issue |
| AEOD/C105 | Report on the Calvert Cliffs Unit 1 Loss of Service Water on 5/20/81 (12/81) | Not Applicable AP600 design has no safety-related service water system |
| AEOD/C201 | Safety Concern Associated With Reactor Vessel Level Instrumentation in Boiling Water Reactors (1/82) | Not Applicable BWR only |

AEOD CASE STUDY REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|---|
| AEOD/C202 | Report on Service Water System Flow Blockages by Bivalve Mollusks at Arkansas Nuclear One and Brunswick (2/82) | SSAR Section 16.2 |
| AEOD/C203 | Survey of Valve Operator Related Events Occurring During 1978, 1979, and 1980 (5/82) | Not Applicable Procurement issue |
| AEOD/C204 | San Onofre Unit 1 Loss of Salt Water Cooling Event on 3/10/80 (7/82) | Not Applicable AP600 design has no safety-related service water system |
| AEOD/C205 | Abnormal Transient Operating Guidelines as Applied to the April 1981 Overfill Event at Arkansas Nuclear One, Unit 1 (8/82) | SSAR Section 7.3.1.1.3.4 |
| AEOD/C206 | Inadvertent Loss of Reactor Coolant Events at the Sequoyah Nuclear Plant, Units 1 and 2 (10/82) | Not Applicable Administrative issue |
| AEOD/C301 | Failure of Class 1E Safety-Related Switchgear Circuit Breakers To Close on Demand (4/83) | Not Applicable Procurement/Maintenance issue |
| AEOD/C401 | Low Temperature Overpressure Events at Turkey Point Unit 4 (3/84) | SSAR Section 3.9.1.1.2.6 |
| AEOD/C402 | Operating Experience Related to Moisture Intrusion in Electrical Equipment at Commercial Power Reactors (6/84) | SSAR Sections 3.11, Appendix 3D |
| AEOD/C403 | Hatch Unit 2 Plant Systems Interaction Event on 8/25/82 (5/84) | Not Applicable BWR only |
| AEOD/C404 | Steam Binding of Auxiliary Feedwater Pumps (7/84) | Not Applicable AP600 design has no safety-related AFW or similar safety-related system |
| AEOD/C501 | Safety Implications Associated with In-Plant Pressurized Gas Storage and Distribution Systems in Nuclear Power Plants (6/85) | Not Applicable Procedural issue |

AEOD CASE STUDY REPORTS

| NUMBER | TITLE | COMMENT |
|-------------|---|---|
| AEOD/C502 | Overpressurization of ECCS in BWRs (9/85) | Not Applicable BWR only |
| AEOD/C503 | Decay Heat Removal Problems at U.S. Pressurized Water Reactors (12/85) | Not Applicable Procedural issue |
| AEOD/C504 | Loss of Safety System Function Events (12/85) | Not Applicable Procedural/Training issue |
| AEOD/C602 | Operational Experience Involving Turbine Overspeed Trips (8/86) | Not Applicable AP600 design has no turbine driven pumps |
| AEOD/C603 | A Review of Motor-Operated Valve Performance (12/86) | Not Applicable Data collection Administrative communication |
| AEOD/C604 | Effects of Ambient Temperature on Electronic Components in Safety-Related Instrumentation and Control Systems (12/86) | USI A-44 SSAR Section 1.9.4 |
| AEOD/C605 | Operational Experience Involving Losses of Electrical Inverters (12/86) | Not Applicable Procurement issue |
| AEOD/C701 | Air Problems at U.S. Nuclear Reactors (1/87) | Not Applicable Maintenance issue |
| AEOD/C801 | Service Water System Failures and Degradations in Light Water Reactors (8/88) | SSAR Section 16.2 |
| AEOD/C90-01 | Operating Experience Feedback Report - Solenoid Operated Valve Problems at U.S. Light Water Reactors (10/90) | Not Applicable Procurement/Installation/ Maintenance issue |

AEOD SPECIAL STUDIES REPORT

| NUMBER | TITLE | COMMENT |
|-----------|---|---|
| AEOD/S401 | Human Error in Events Involving Wrong Unit or Wrong Train (1/84) | Not Applicable Personnel error |
| AEOD/S402 | Pressure Locking of Flexible Disk Wedge Type Gate Valves (7/84) | Not Applicable Procurement issue |
| AEOD/S403 | Annual Report of U.S. NRC Participation in the Nuclear Energy Agency Incident Reporting System During 1983 (6/84) | Not Applicable Administrative communication |
| AEOD/S404 | Analysis of Foreign IRS Reports Submitted During CY 1983 (6/84) | Not Applicable Administrative communication |
| AEOD/S405 | Semiannual Report on AEOD Activities (9/84) | Not Applicable Administrative communication |
| AEOD/S406 | Application of risk Perspectives: A Procedures Guide | Not Applicable Administrative communication |
| AEOD/S501 | Review of Operational Experience From Non-Power Reactors (3/85) | Not applicable to commercial nuclear power plants |
| AEOD/S502 | AEOD Semiannual Report for July-December 1984 (4/85) | Not Applicable Administrative communication |
| AEOD/S503 | Evaluation of Recent Valve Operator Motor Burnout Events (9/85) | Not Applicable Maintenance issue |
| AEOD/S601 | AEOD Annual Report for 1985 (4/86) | Not Applicable Administrative communication |
| AEOD/S602 | An Overview of Nuclear Power Plant Operating Experience Feedback Programs (5/86) | Not Applicable Administrative communication |
| AEOD/S603 | Adequacy of the Scope of IE Bulletin 86-01 (6/86) | Not Applicable BWR only |
| AEOD/S702 | Loss of Decay Heat Removal function at Pressurized Water Reactors With Partially Drained Reactor Coolant Systems (5/87) | SSAR Section 5.4.7.1.2.1 |
| AEOD/S801 | Significant Events That Involved Procedures (3/88) | Not Applicable Procedural issue |

AEOD SPECIAL STUDIES REPORT

| NUMBER | TITLE | COMMENT |
|-------------|--|--|
| AEOD/S802 | Operational Experience feedback Evaluation Rancho Seco Nuclear Generating Station, Restart (3/88) | Not Applicable Plant specific |
| AEOD/S803 | AEOD Concerns Regarding the Power Oscillation Event at LaSalle 2 (6/88) | Not Applicable BWR only |
| AEOD/S804A | Preliminary Results of the Trial Program for Maintenance Performance Indicators (8/88) | Not Applicable Administrative issue |
| AEOD/S804 | Report to the U.S. Nuclear Regulatory Commission on Analysis and Evaluation of Operational Data - 1987 Power Reactors (9/88) | Not Applicable Administrative issue |
| AEOD/S901 | Maintenance Programs at Nuclear Power Plants (8/90) | Not Applicable Internal use for inspections |
| AEOD/S902 | Review of Thermal Stratification Operating Experience (4/89) | SSAR Sections 5.4.10.1 and 10.4.7.2.2 |
| AEOD/S90-01 | Recurrence of Important Safety Issues Reported in LERs (8/90) | Not Applicable Administrative communication |
| AEOD/S91-01 | Performance of Emergency Diesel Generators in Restoring Power to Their Associated Safety Buses - A review of Events Occurring at Power (9/91) | SSAR Section 16.2 |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|--|
| AEOD/T301 | Diesel Generator Load Sequencer Design Deficiency - LER 82-025/OIT (1/83) | SSAR Section 8.3.1.1.2.1 |
| AEOD/T302 | Postulated Loss of Auxiliary Feedwater System Resulting from a Turbine Driven Auxiliary Feedwater Pump Steam Supply Line Rupture (2/83) | Not Applicable AP600 design has no turbine driven pumps |
| AEOD/T303 | Seat Degradation in Henry Pratt Butterfly Valves (3/83) | Not Applicable Procurement issue |
| AEOD/T304 | Cause of Containment Isolation Valve F042A to Close (3/83) | Not Applicable BWR only |
| AEOD/T305 | Flow Blockage in essential Raw Cooling Water System Due to Asiatic Clam Intrusion at Sequoyah Unit 1 (3/83) | SSAR Section 16.2 |
| AEOD/T306 | Scram Discharge Volume Level Switch Failure at Hatch Unit 2 (4/83) | Not Applicable BWR only |
| AEOD/T307 | Condensate Demineralizer Resin Migration Through the Plant Vent and the Standby Gas Treatment System at Pilgrim Unit 1 (4/83) | Not Applicable Plant specific issue |
| AEOD/T308 | Undetectable Failure in westinghouse Solid State Protection Systems (4/83) | Not Applicable Procurement/Procedural issue |
| AEOD/T309 | Air in Reactor Water Cleanup system Instrument Sensing Lines at Brunswick Unit 2 (4/83) | Not Applicable Installation/Procedural issue |
| AEOD/T310 | Blocking of Automatic Safety Injection Signals (4/83) | Not Applicable Personnel error |
| AEOD/T311 | Rod Control Urgent Failure on 6/25/82 at Surry Unit 2 (5/83) | Not Applicable Random failure, no issue |
| AEOD/T312 | Failure of 5 KV Cable Terminations (5/83) | Not Applicable Installation issue |
| AEOD/T313 | Capped Containment Pressure Sensing Lines (5/83) | Not Applicable Procedural issue |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|---|
| AEOD/T314 | Improper size of Inlet piping to Primary Safety Valves (5/83) | Not Applicable CE facilities only |
| AEOD/T315 | Events Involving Losses of or Perturbations in a Single 120 Volt AC Vital Power Supply Inverter and Attendant Distribution Bus Which Resulted in Inadvertent Actuations of Safety Systems (5/83) | Not Applicable CE facilities only |
| AEOD/T316 | Thermal Non-Repeatability Problem with Barton Models 763 and 764 Electronic Transmitters (5/83) | Not Applicable Procurement issue |
| AEOD/T317 | Problems With Diesel Driven Containment Spray Pumps at Zion Unit 2 on 12/16/82 (6/83) | Not Applicable AP600 design has no containment spray |
| AEOD/T318 | Failure of Recirculation Spray Service Water Motor Operated Valves (6/83) | SSAR Section 16.2 |
| AEOD/T319 | Design Deficiency in Control Circuits of Feedwater Isolation Valves and Boron Injection Tank Recirculation Valves (6/83) | SSAR Section 7.3.1.1.1 |
| AEOD/T320 | Inadvertent Safety Injections Attributed to Personnel Error at Summer (6/83) | Not Applicable Procedural issue |
| AEOD/T321 | Check Valve Installed Backwards in Instrument Air Line to the Power Operated Relief Valve at Surry Unit 1 (6/83) | Not Applicable Installation issue |
| AEOD/T322 | Gouges in Main Coolant System Piping at Diablo Canyon on 4/19/83 (6/83) | Not Applicable Personnel action |
| AEOD/T323 | Turbine Trip Bypass Delay at Grand Gulf Unit 1 (6/83) | Not Applicable BWR only |
| AEOD/T324 | Event Involving Two or More Simultaneously Dropped Rod Control Cluster Assemblies (7/83) | Not Applicable No issue |
| AEOD/T325 | Leakage in Static "O" Ring Pressure Switches (8/83) | Not Applicable Procurement issue |
| AEOD/T326 | Safety Relief Valve Corrosion at a Foreign Reactor (8/83) | Not Applicable Maintenance issue |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/T327 | Auxiliary Feedwater Header Problems at Babcock & Wilcox Plants (8/83) | Not Applicable B&W facilities only |
| AEOD/T328 | Two of Three Emergency Core Cooling System Accumulators Inoperable at Surry Unit 1 (8/83) | Not Applicable Procedural issue |
| AEOD/T329 | Leak in Reactor Water Cleanup System "B" Regenerative Heat Exchanger Relief Line (8/83) | Not Applicable Plant specific issue |
| AEOD/T330 | Steam Generator Tube Rupture at Oconee Unit 2 (8/83) | Not Applicable Procedural issue |
| AEOD/T331 | Review of Events at Operating Nuclear Plants Involving Plant Computers (8/83) | Not Applicable Procedural/Maintenance issue |
| AEOD/T332 | Reactor Vessel Drainage (10/83) | Not Applicable BWR only |
| AEOD/T333 | Degradation of Salt Water Cooling System at San Onofre Unit 1 Due to a Loss of Instrument Air (10/83) | SSAR Section 16.2 |
| AEOD/T334 | Reactor Vessel Drainage at Grand Gulf Unit 1 (11/83) | See AEOD/T332 |
| AEOD/T335 | Simultaneous Safety Injection Actuation Signal and Recirculation Actuation Signal at San Onofre Unit 3 (11/83) | Not Applicable No action, no issue |
| AEOD/T336 | Design Deficiency Resulting in Isolation of Both Loops of the Emergency Condenser System at Nine Mile Point Unit 1 (11/83) | Not Applicable Plant specific issue |
| AEOD/T337 | Water Hammer in the Main Feedwater System Resulting in a Feedwater Line Crack at Maine Yankee (11/83) | SSAR Section 10.4.7.2.2 |
| AEOD/T338 | Water Leak Through Containment Spray Block Valves at San Onofre 1 (11/83) | Not Applicable No action, no issue |
| AEOD/T339 | Redundant Emergency Core Cooling System Pump Room Air Coolers Out of Service for 22 Hours at Calvert Cliffs Unit 1 (11/83) | Not Applicable Procedural/Training issue |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|---|
| AEOD/T340 | Evaluation of a Control Rod Mismanipulation Event at Hatch Unit 2 (12/83) | Not Applicable Procedural/Training issue |
| AEOD/T341 | Corrosion of Carbon Steel Pipe in Service Water Headers (12/83) | SSAR Section 16.2 |
| AEOD/T401 | Failures of Containment Air Monitors at Farley Units 1 and 2 (3/84) | Not Applicable Procurement/Maintenance issue |
| AEOD/T402 | Chemical Contamination of Primary and Secondary Systems in Light Water Reactors (3/84) | SSAR Sections 5.2.3.2, 10.3.5 |
| AEOD/T403 | Setpoint Drift of Barton Model 288 Switches (3/84) | Not Applicable Procurement issue |
| AEOD/T404 | Cable Fire and Loss of Control Power to Engineered Safeguards Valves (4/84) | Not Applicable No issue/no action |
| AEOD/T405 | Cold Weather Events 1983-1984 (4/84) | Not Applicable No issue/no action |
| AEOD/T406 | Improper Spare Parts Procurement Event at Grand Gulf Unit (4/84) | Not Applicable Procurement issue |
| AEOD/T407 | Failure of 4 KV Circuit Breaker to Trip (4/84) | Not Applicable No issue/no action |
| AEOD/T408 | Diesel Generator Inoperability Due to Overheating of Ventilation Cowling (5/84) | Not Applicable Plant specific issue |
| AEOD/T409 | Multiple Failure of Bell and Howell Dual Potentiometer Modules That Occurred at the Fort Calhoun Nuclear Station (5/84) | Not Applicable Procurement issue |
| AEOD/T410 | Failure of Injection Valve for the High Pressure Coolant Injection System To Open During a Surveillance Test (5/84) | Not Applicable Installation issue |
| AEOD/T411 | Contamination of the Nitrogen System at Sacramento Municipal Utility District (6/84) | Not Applicable Plant specific issue |
| AEOD/T412 | Failure of an Access Door Between the Drywell and the Wetwell (6/84) | Not Applicable BWR only |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/T413 | Failure of Fire Damper in Safeguards Ventilation System (6/84) | Not Applicable Plant specific issue |
| AEOD/T414 | Station Operator Restrictions for Lost or Out-of-Service Power Transformers Through Which Electrical Power is Supplied to the Emergency Buses (7/84) | Not Applicable Offsite transformer not safety-related |
| AEOD/T415 | Destruction of Charging Pump (7/84) | Not Applicable Procedural/Training issue |
| AEOD/T416 | Loss of Engineered Safety Feature Auxiliary Feedwater Pump Capability at Trojan on 1/22/83 (8/84) | Not Applicable Procedural/Training issue |
| AEOD/T417 | Excessive Cooldown Rate Event at LaSalle Unit 1 (8/84) | Not Applicable Procedural/Training issue |
| AEOD/T418 | Events Involving Fires or Other Related Abnormalities in Motor Control Centers with Aluminum Bus Bars (8/84) | Not Applicable Procurement issue |
| AEOD/T419 | Contamination of Snubber Bleed Screw and Lockup Poppet Valve (8/84) | Not Applicable Plant specific issue |
| AEOD/T420 | Failure of an Isolation Valve of the Reactor Core Isolation Cooling System To Open Against Operating Reactor Pressure (8/84) | Not Applicable BWR only |
| AEOD/T421 | Design Deficiency in Standby Gas Treatment System (8/84) | Not Applicable Plant specific issue |
| AEOD/T422 | Inoperability of Safety Injection Pump at Salem Unit 1 on 10/17/83 (8/84) | Not Applicable Plant specific issue |
| AEOD/T423 | Inoperability of Helium Circulator Overspeed trip Channels Due to Impedance Variations in Speed Sensing Cables Exposed to Steam Leak (10/84) | SSAR Sections 3.11, Appendix 3D |
| AEOD/T424 | Fire Water Main Leakage Into 4 KV Switchgear Room at San Onofre Unit 1 (11/84) | Not Applicable Construction issue |
| AEOD/T501 | Failure of Automatic Protection for Boron Dilution Event at Callaway Unit 1 (1/85) | Not Applicable Procedural issue |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/T502 | Comparative Analysis of Recent Feedline Water Hammer Events at Maine Yankee, Calvert Cliffs, Palisades, and Salem (3/85) | Not Applicable Procedural issue |
| AEOD/T503 | Pressurizer Level Instrumentation at Combustion Engineering Reactor Units (5/85) | Not Applicable CE facilities only |
| AEOD/T504 | Loss of Instrument Air and Subsequent Pressure Transient at Callaway Unit 1 (5/85) | Not Applicable Plant specific issue |
| AEOD/T505 | Beaver Valley Component Cooling Water Pump Damage (7/85) | Not Applicable Installation/Maintenance issue |
| AEOD/T506 | Primary System Release Due to Pressurizer Degas Relief Valve Lifting (7/85) | Not Applicable Plant specific issue |
| AEOD/T507 | Standby Liquid Control System Pressure Relief Valve Lift at a Pressure Lower Than Reactor Coolant Pressure (8/85) | Not Applicable Plant specific issue |
| AEOD/T508 | Browns Ferry Nuclear Plant High Pressure Coolant Injection System Performance Assessment (8/85) | Not Applicable BWR only |
| AEOD/T509 | Inadequate Surveillance Testing Procedures for Degraded Voltage and Undervoltage Relays Associated With 4160-Volt Emergency Buses (8/85) | Not Applicable Procedural issue |
| AEOD/T510 | Xenon Induced Power Oscillations at Catawba (9/85) | Not Applicable Procedural issue |
| AEOD/T511 | Technicians Perform Work on Wrong Control Rod Drive Mechanism (9/85) | Not Applicable Procedural/Training issue |
| AEOD/T512 | Incorrect Plugging of Steam Generator Tubes (10/85) | Not Applicable Procedural issue |
| AEOD/T513 | Flooding of Safety-Related Valves in Pits (11/85) | SSAR Section 3.4 |
| AEOD/T514 | Potential Loss of Component Cooling Water Due to Misadjustment of Relief Valves (11/85) | Not Applicable Isolated event |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/T515 | Residual Heat Removal Service Water Booster Pump Air Binding at Brunswick Unit 1 (12/85) | Not Applicable Plant specific issue |
| AEOD/T516 | High Pressure Coolant Injection Overspeed Trip Loss Events and Subsequent Damage Due to Water Hammer (12/85) | Not Applicable Plant specific issue |
| AEOD/T601 | Pressure Sensitive Temperature Switch Results in Spurious Actuation of Fire Suppression System (1/86) | Not Applicable Plant specific procurement issue |
| AEOD/T602 | Emergency Diesel Generator Cooling Water Design Deficiencies at Maine Yankee and Haddam Neck (4/86) | SSAR Section 8.3.1.1.2.1 |
| AEOD/T603 | Inadvertent Pump Suction Transfer and Potential Auxiliary Feedwater Pump Cavitation at Davis Besse (4/86) | Not Applicable AP600 design has no safety-related AFW system or similar safety-related system |
| AEOD/T604 | Events Resulting From Deficiencies in Labeling and Identification Systems (5/86) | Not Applicable Procurement/Installation issue |
| AEOD/T605 | Failure of Main Steam Safety Valves to Reseat (6/86) | Not Applicable B&W facilities only |
| AEOD/T606 | Inadvertent Recirculation Actuation Signal at Combustion Engineering Plants (8/86) | Not Applicable CE facilities only |
| AEOD/T607 | Occurrence of Events Involving Wrong Unit/Wrong Train/ Wrong Component - Update Through June 1986 (9/86) | Not Applicable Procedural/Training issue |
| AEOD/T608 | Hydrogen Fire and Failure of Detection System (11/86) | Not Applicable Isolated occurrence |
| AEOD/T609 | Foreign Material and Debris in Safety-Related Fluid Systems (12/86) | Not Applicable Part of COL |
| AEOD/T610 | ADS/RCIC System Interaction Events at River Bend Unit 1 (12/86) | Not Applicable BWR only |
| AEOD/T611 | Denied Access Due to Negative Room Pressure (12/86) | Not Applicable Plant specific event |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|---|
| AEOD/T612 | Degradation of Safety Systems Due to Component Misalignment and/or Mispositioned Control/Selector Switches (12/86) | Not Applicable Procedural/Personnel issue |
| AEOD/T701 | Compression Fitting Failures (1/87) | Not Applicable Maintenance/Installation issue |
| AEOD/T702 | Leaking Pulsation Dampener Leads to Loss of Charging System (3/87) | SSAR Section 16.2 |
| AEOD/T703 | Potential for Loss of Emergency Feedwater Caused by Pump Runout During Certain Transients (3/87) | Not Applicable AP600 design has no safety-related AFW or similar safety-related system |
| AEOD/T704 | Pressurizer Code Safety Valve Reliability (3/87) | Not Applicable Maintenance issue |
| AEOD/T705 | Occurrence of Events Involving Wrong Unit/Wrong Train/ Wrong Component - Update Through 1986 (5/87) | Not Applicable Procedural/Training issue |
| AEOD/T706 | Recent Events Involving Turbine Runbacks at PWRs (6/87) | Not Applicable Conclusion satisfactory, no action |
| AEOD/T707 | Undetected Loss of reactor water (8/87) | SSAR Section 7.5 |
| AEOD/T708 | Problems with High Head Safety Injection Systems in Westinghouse PWRs (8/87) | Not Applicable Random Failure/no action |
| AEOD/T709 | Recent New Plant Operational Experience (10/87) | Not Applicable No conclusions |
| AEOD/T710 | Heating, Ventilation, and Air Conditioning System Problems (11/87) | Not Applicable Random Failure/no action |
| AEOD/T711 | No Title (no date) | Not Issued |
| AEOD/T712 | Unplanned Criticality Events at U.S. Power Reactors Similar to That at Oskarshamn Unit 3 on 7/30/87 (11/87) | Not Applicable Procedural/Training issue |
| AEOD/T713 | Mispositioning of "Reverse Acting" Valve Controllers (12/87) | Not Applicable Plant specific |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|--|--|
| AEOD/T801 | Perry Nuclear Power Plant Unit 1 - Unexpected MSIVs Closure and Reopening (1/88) | Not Applicable Plant specific event |
| AEOD/T802 | No Title (no date) | Not Issued |
| AEOD/T803 | Summary of Early operational Experience of Foreign Commercial Nuclear Reactors (5/88) | Not Applicable Administrative communication |
| AEOD/T804 | "Precursor" Operational Events That Occurred From November 1, 1987 Through March 1988 (5/88) | Not Applicable No issue |
| AEOD/T805 | Insights From Significant Events in 1987 (5/88) | Not Applicable Administrative issue |
| AEOD/T806 | Recent Operational Experience Trends at Fermi 2 (5/88) | Not Applicable Plant specific event |
| AEOD/T807 | Technical Review Report - Recent Operational Experience Trends at Indian Point 2 (6/88) | Not Applicable Plant specific event |
| AEOD/T808 | A Technical Basis for Granting Test Frequency Relief (6/88) | SSAR Section 3.9.6.2 Surveillance issue |
| AEOD/T809 | Blocked Thimble Tubes/Stuck Incore Detector (6/88) | Not Applicable Maintenance issue |
| AEOD/T810 | An Analysis of NPRDS Date for Hatch Plant (7/88) | Not Applicable Administrative issue |
| AEOD/T811 | Degradation of Ice Condenser Containment Functional Capability (11/88) | Not Applicable Ice condenser only |
| AEOD/T901 | Millstone Unit 1 - Safety/Relief Valve Discharge Line Vacuum Breakers Failed Open (1/89) | Not Applicable Procurement issue |
| AEOD/T902 | Inadvertent Reactor Trips Due to RCS Flow Instrumentation Maintenance Activities (2/89) | Not Applicable Maintenance issue |
| AEOD/T903 | Generic Implication of Browns Ferry Fire on 11/2/87 (3/89) | Not Issued |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-----------|---|---|
| AEOD/T904 | Design Deficiency of Safety Injection Block Switch (4/89) | Not Applicable Procurement issue |
| AEOD/T905 | Failure of 4160 Volt GE Magneblast Breaker to Trip Open (4/89) | Not Applicable Procurement issue |
| AEOD/T906 | Broken Lifting Beam Bolts in HPCI Terry Turbine (4/89) | Not Applicable AP600 has no turbine driven pumps |
| AEOD/T907 | Component Degradation Due to Indiscriminate Painting (4/89) | Not Applicable Procedural/Maintenance issue |
| AEOD/T908 | A nonreactor Report (no date) | Not applicable to commercial nuclear power plants |
| AEOD/T909 | Operating Events Involving Dampers (5/89) | Not Applicable Procedural issue |
| AEOD/T910 | Investigation of Cracked Control rod Drive Seal Housings at Palisades (6/89) | Not Applicable Plant specific issue |
| AEOD/T911 | Evaluation of Individually Reported Safety System LERs for Their Combined Significance (6/89) | Not Applicable Procedural issue |
| AEOD/T912 | Selected Maintenance Rework (6/89) | Not Applicable Procedural/Maintenance issue |
| AEOD/T913 | Comparison of the Proposed Maintenance Effectiveness (ME) Indicator With Catawba and Farley Nuclear Plants Regarding Inspections (7/89) | Not Applicable Procedural/Inspection issue |
| AEOD/T914 | Overview of Design/Installation Fabrication Errors in 1988 (9/89) | Not Applicable No issue |
| AEOD/T915 | EDG Ground Fault Detection and Trip Circuit at Perry Unit 1 (9/89) | Not Applicable Plant specific issue |
| AEOD/T916 | Debris in Containment Recirculation Sumps (9/89) | Not Applicable Procedural issue |
| AEOD/T917 | No Title (no date) | Not Issued |
| AEOD/T918 | Check Valve Failure Rates From NPRDS Data (9/89) | Not Applicable Administrative issue |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-------------|--|---|
| AEOD/T919 | Failure of Overcurrent Protective Devices at Palisades Unit 1 (9/89) | Not Applicable Maintenance issue |
| AEOD/T920 | No Title (no date) | Not Issued |
| AEOD/T921 | Inadequate Capacity of 4160 Volt Switchgear at FitzPatrick (10/89) | Not Applicable Procedural issue |
| AEOD/T922 | Failure of HPCI Turbine Due to High Moisture in Lube Oil (11/89) | Not Applicable BWR only |
| AEOD/T923 | Delaminating Foil Insulation in Primary Containment (11/89) | Not Applicable Isolated event |
| AEOD/T924 | No Title (no date) | Not Issued |
| AEOD/T925 | Evaluation of Safety Equipment Outages for Significance at Zion (12/89) | Not Applicable No issue |
| AEOD/T926 | Evaluation of Two Beaver Valley 2 Nuclear Plant Equipment Degradation Events for Their Combined Significance (12/89) | Not Applicable Maintenance issue |
| AEOD/T927 | Follow-up on Steam Binding of AFW Pumps (12/89) | Not Applicable AP600 design has no safety-related AFW or similar safety-related system |
| AEOD/T928 | Inadequate Overpressure Protection for Auxiliary Steam Headers at the Oconee Plants (12/89) | Not Applicable Plant specific issue |
| AEOD/T90-01 | PNOs Issued in First Quarter of 1989 (1/90) | Not Applicable Administrative issue |
| AEOD/T90-02 | Insights Regarding Commonwealth Edison Plant Root-Cause Investigation Related to Maintenance Effectiveness (1/90) | Not Applicable Maintenance issue |
| AEOD/T90-03 | Improper Installation of Heat Shrinkable Tubing (3/90) | Not Applicable Installation issue |
| AEOD/T90-04 | Reverse (Backward) Acting Valve Manual Handwheels (3/90) | Not Applicable Procurement issue |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-------------|---|---|
| AEOD/T90-05 | Association Between Nuclear Plant Utilization and Incentive Regulation by Station Public Utility Commissioners (3/90) | Not Applicable Administrative issue |
| AEOD/T90-06 | Aquatic Life in Emergency Cooling Ponds (5/90) | Not Applicable AP600 design has no cooling ponds |
| AEOD/T90-07 | Reversed Sensing Lines Connections (5/90) | Not Applicable Installation/maintenance issue |
| AEOD/T90-08 | Turbine Bypass malfunctions (6/90) | SSAR Chapter 15 |
| AEOD/T90-09 | Inadvertent Partial Draining of Condensate Storage Tanks (6/90) | Not Applicable No issue/no action |
| AEOD/T90-10 | Evaluations of Maintenance trends at Five Selected sites (7/90) | Not Applicable Maintenance issue |
| AEOD/T90-11 | Effects of High Energy Line Breaks on Chilled Water Systems at Nuclear Power Plants (8/90) | Not Applicable AP600 design has no safety-related chilled water system |
| AEOD/T90-12 | Loss of Offsite Power to Comply With NRC Regulations (9/90) | Not applicable Plant specific issue |
| AEOD/T90-13 | Corrosion and Failure of Service Water Pump Impeller Snap Rings (10/90) | Not applicable Plant specific issue |
| AEOD/T90-14 | Seal Problems in Boric Acid Transfer Pumps (10/90) | Not Applicable Plant specific issue |
| AEOD/T90-15 | Salem 1 & 2 Evaluation of Operating Experience (10/90) | Not Applicable Administrative communication |
| AEOD/T90-16 | Impact of Pipe Liner Failure of Pump Operation (11/90) | SSAR Section 16.2 |
| AEOD/T90-17 | Inadvertent Containment Spray Actuations (12/90) | Not Applicable AP600 design has no containment spray system |
| AEOD/T91-01 | Causes of Incorrect System Flows (2/91) | Not Applicable AP600 design has no safety-related service water system |

AEOD TECHNICAL REVIEW REPORTS

| NUMBER | TITLE | COMMENT |
|-------------|--|---|
| AEOD/T91-02 | Incorrect Rotation of PDP (2/91) | Not Applicable Maintenance issue |
| AEOD/T91-03 | Overloaded Emergency Buses (3/91) | Not Applicable AP600 design has no Class 1E AC Power System |
| AEOD/T91-04 | Turbine Overspeed trip Due to Steam Valve Leakage and Condensate (4/91) | Not Applicable AP600 design has no turbine driven pumps |
| AEOD/T91-05 | Setpoint Testing of Pressurizer Safety Valves With Water-Filled Loop Seals (5/91) | Not Applicable Procedural issue |
| AEOD/T91-06 | Deficiencies in External Flood Protection (6/91) | SSAR Section 3.4 |
| AEOD/T91-07 | Evaluation of Partial Loss of Station Power Events at Prairie Island Unit No. 2 on 12/21 and 12/26/89 (7/91) | Not Applicable Plant specific issue |