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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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The Honorable Gary Hart
Chairman
Subcommittee on Nuclear Regulation
Committee on Environment and Public Works
United States Senate
Washington, D.C. 20510

Dear Mr. Chairman:

Enclosed for your Subcommittee information is the "For Comment" edition of NUREG-0313, Rev. 1, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping." This report constitutes the staff's resolution of the NRC's Generic Technical Activity A-42, "Pipe Cracks in Boiling Water Reactors," which had been designated as an "Unresolved Safety Issue" pursuant to Section 210 of the Energy Reorganization Act of 1974.

NUREG-0313, Rev. 1 sets forth the NRC staff's revised acceptable methods to reduce the intergranular stress corrosion cracking susceptibility of BWR ASME Code Class 1 & 2 pressure boundary piping and safe ends. For plants that cannot fully comply with the staff's guidelines, varying degrees of augmented inservice inspection and leak detection requirements are provided.

We intend to issue this document for a 60 day public comment period. Also enclosed for your information is a Federal Register Notice we plan to issue on this matter within the next few days.

Sincerely,

Darrell G. Eisenhut

Darrell G. Eisenhut, Acting Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

Enclosures:

1. NUREG-0313, Rev. 1
2. Federal Register Notice

cc: The Honorable Alan Simpson

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NUCLEAR REGULATORY COMMISSION
NUREG-0313, REV. 1
NOTICE OF ISSUANCE AND AVAILABILITY
TECHNICAL REPORT ON
MATERIAL SELECTION AND PROCESSING
GUIDELINES FOR BWR COOLANT PRESSURE BOUNDARY PIPING

A task force with members from the Nuclear Regulatory Commission (NRC) has prepared a report entitled "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping" (NUREG-0313, Rev. 1), dated October 1979. This report constitutes the resolution of the NRC's Generic Activity A-42, "Pipe Cracks in Boiling Water Reactors," which was an "Unresolved Safety Issue" pursuant to Section 210 of the Energy Reorganization Act of 1974.

This generic study was initiated because of the recent publication of NUREG-0531, "Investigation and Evaluation of Stress-Corrosion Cracking in Piping of Light Water Reactor Plants," by the NRC 1978 Pipe Crack Study Group (PCSG). The new Study Group was specifically chartered among others to reexamine the conclusions and recommendations of the 1975 PCSG report (NUREG-75/067) in view of cracks recently discovered in large diameter pipes, and to evaluate the significance of safe end cracking at Duane Arnold relative to similar material and design aspects at other facilities. Because of the new ideas and issues addressed in NUREG-0531, the implementation document NUREG-0313, which is based on the 1975 PCSG report, needs to be updated to incorporate the latest recommendations made by the 1978 PCSG.

NUREG-0313, Rev. 1 sets forth the NRC staff's revised acceptable methods to reduce the intergranular stress corrosion cracking susceptibility of BWR ASME Code Class 1 & 2 pressure boundary piping and safe end. For plants that cannot fully

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comply with the material selection, testing, and processing guidelines of this report, varying degrees of augmented inservice inspection and leak detection requirements are presented.


Public comments are being solicited from interested organizations, groups, and individuals. These comments will have bearing on final Commission action, particularly with regard to implementation.

Copies of the report will be available after October 1979. Copies will be sent directly to utilities, utility industry groups and associations, and environmental and public interest groups. Other copies will be available for review at the NRC Public Document Room, 1717 H Street, N. W. Washington, D.C. and the Commission's local public document room located in the vicinity of existing nuclear power plants. Addresses of these local public document rooms can be obtained by contacting the Chief, Local Public Document Room Branch, Mail Stop 309, Nuclear Regulatory Commission, Washington, D.C. 20555, telephone (301) 492-7356. A single copy of NUREG-0313, Rev. 1 will be provided free of charge, while the supply lasts, upon written request of a full participant in an ongoing NRC proceeding. This request must identify the requester as a participant and should be addressed to Director, Division of Technical Information and Document Control, Nuclear Regulatory Commission, Washington, D.C. 20555.

Comments should be forwarded to Dr. Stephen H. Hanauer, Director, Unresolved Safety Issues Program, Nuclear Regulatory Commission, Washington, D.C. 20555.

Dated at Bethesda, Maryland, this 17th day of October, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION


Darrell G. Eisenhut, Acting Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

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