

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Waterford 3 Steam Electric Station										DOCKET NUMBER (2) 0 5 0 0 0 3 8 2				PAGE (3) 1 OF 0 3		
TITLE (4) Reactor Trip On Inadvertent Closure of Main Steam Isolation Valve																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
									N/A				0 5 0 0 0			
0 5	0 5	8 5	8 5	0 1 7	0 0	0 6	0 4	8 5	N/A				0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
1		20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)		
0 1 1 7		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME O.D. Hayes, Operations Superintendent										TELEPHONE NUMBER AREA CODE 5 0 4 4 6 4 1 - 3 1 1 8						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS						
X	SIB	IISIV	W21515	N												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At 0007 hours on May 5, 1985 Waterford 3 Steam Electric Station was at 17% reactor power when an uncomplicated reactor trip occurred as a result of an inadvertent closure of Main Steam Isolation Valve number 2. Because of a pressure transducer malfunction, coupled with the back leakage of oil through a check valve, oil pressure in the valve actuator decreased until the valve fully closed. The faulty components have since been repaired.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Waterford 3 Steam Electric Station	0 5 0 0 0 3 8 2	8 5	— 0 1 7	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

On May 4, 1985 Waterford 3 Steam Electric Station was at 17% reactor power when plant personnel observed that, contrary to its intended function, the motor driven hydraulic pumps for Main Steam Isolation Valve number 2 were constantly running. An investigation by Plant Personnel revealed that, due to a faulty hydraulic pressure transducer, the pumps were constantly cycling in an effort to compensate for the faulty pressure indication. The pressure transducer was subsequently repaired; however, at 2352 hours on May 4, 1985 Operations Personnel received an intermediate position indication for Main Steam Isolation Valve number 2. Plant Personnel investigated the problem and found that one of the motor driven hydraulic pumps had tripped on thermal overload (this was the result of the constant cycling of the pump described above) and, as a result of back leakage through the discharge check valve, was rotating backwards due to reverse flow from the operating motor driven hydraulic pump. Before the pump could be isolated oil pressure in the valve actuator decreased until, at 0006 hours on May 5, 1985, Main Steam Isolation Valve number 2 fully closed. Shortly thereafter, at 0007 hours, as a result of the valve closure, the Reactor Coolant System cold leg temperatures differed by greater than 18 degrees Fahrenheit precipitating an auxiliary trip in the Core Protection Calculator. This was signified as an uncomplicated reactor trip on low DNBR and high Local Power Density.

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in an actuation of the Reactor Protective System during the initial Startup testing in which no primary systems parameters were exceeded. Since the Control Element Assemblies and the Reactor Protective System functioned as designed, and since Main Steam Isolation Valve number 2 failed in the safe condition, the event in no way placed Waterford 3 in a degraded safety condition.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

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YEAR

SEQUENTIAL
NUMBERREVISION
NUMBER

Waterford 3 Steam Electric Station

0 5 0 0 0 3 8 2 8 5 - 0 1 7 - 0 0 0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 385A's) (17)

CORRECTIVE ACTION

The Main Steam Isolation Valve closed as a result of a malfunction of two components; namely, the hydraulic pressure transducer and the check valve on the discharge of the motor driven hydraulic pump. These components have since been repaired. Since environmental conditions (heat) may contribute to component failure, Station Modification Request 441 has been written to evaluate the possibility of moving several components located in the vicinity of the valve yoke.

SIMILAR EVENTS

NONE

PLANT CONTACT

O.D. Hayes, Operations Superintendent, 504/464-3118



LOUISIANA
POWER & LIGHT

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June 4, 1985

W3P85-1404
A4.05

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-017-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(iv).

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, D.M. Crutchfield, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), B.W. Churchill,
W.M. Stevenson

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