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Florida
Power
CORPORATION

July 16, 1979

File: 3-0-3-a-3

Mr. J. P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Office of Inspection & Enforcement
101 Marietta Street, Suite 3100
Atlanta, GA 30303

Subject: Crystal River Unit #3
Docket No. 50-302
Operating License No. DPR-72
IE Bulletin 79-13

Dear Mr. O'Reilly:

Enclosed please find our response to Item 5 of IE Bulletin 79-13. Our response to Item 6 will be submitted on or before July 25, 1979.

Should you have any questions, please contact me.

Very truly yours,

FLORIDA POWER CORPORATION

W. P. Stewart
W. P. Stewart
Manager, Nuclear Operations

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Response to Item 5

Item 5a

Provide your schedule for inspection if required by Item 1.

Response:

This item is not applicable to Crystal River Unit 3.

Item 5b

Address the adequacy of your operating and emergency procedures to recognize and respond to a feedwater line break accident.

Response:

Emergency Procedure EP-108, Loss of Steam Generator Feed, addresses recognition and response to a feedwater line break accident. Reference to EP-105, Steam Supply System Rupture, is given as the required immediate action. These procedures have been revised extensively to reflect the latest operating guidelines during accident conditions as a result of the TMI-2 events. These revised procedures were included in the training program at CR #3 and have been reviewed and found acceptable by both the NRC (I&E and NRR) and Florida Power Corporation.

Item 5c

State the methods and sensitivity of detection of feedwater leaks in containment.

Response:

Per EP-108, the following symptoms are to be analyzed to verify feedwater system rupture (included are instrument number, span, and tolerance):

<u>Symptoms</u>	<u>Instrument #</u>	<u>Span</u>	<u>Tolerance</u>
1. Increasing FW Flow	SP-8A & 8B-FE	6000x10 ³ lbs/hr	+ 60 x 10 ³ lbs/hr
2. Decreasing Steam Generator Level	SP-1A & 1B-LT2 SP-1A & B-LT3	291.51 in. H ₂ O	+ 1%
3. Decreasing Main Steam Pressure	SP-6A & 6B -PT1 SP-6A & 6B -PT2	1200 psig	+ 18 psig