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September 25, 1992
Fort St. Vrain
Unit No. 1
P-92283

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

ATTN: Dr. Seymour H. Weiss, Director
Non-Power Reactor, Decommissioning and
Environmental Project Directorate

Docket No. 50-267

SUBJECT: Decommissioning Technical Specifications, Supplemental
Information

REFERENCE: PSC Letter, Crawford to Weiss, dated March 19, 1992 (P-92115)

Dear Dr. Weiss:

This letter provides supplemental information for the Fort St. Vrain (FSV) Decommissioning Technical Specifications that Public Service Company of Colorado (PSC) had previously submitted via the referenced letter. During recent discussions with the NRC, PSC agreed to make several changes for clarification and to more accurately reflect decommissioning conditions.

The FSV Decommissioning Technical Specification changes reflected in the attachments to this submittal include the following:

- Replaced "radiation safety" with "nuclear safety". This term is used to distinguish activities that are subject to various administrative controls, such as safety committee reviews and record retention.
- Revised Applicability requirements for specifications dealing with reactor building confinement integrity, to ensure that the limiting conditions apply whenever activated graphite blocks are removed from the Prestressed Concrete Reactor Vessel, not whenever these blocks are removed from the shield water.

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- Clarified the items involving unreviewed safety questions that must be reviewed by the Decommissioning Safety Review Committee.
- Revised the Semiannual Radioactive Effluent Release Report to an Annual Radioactive Effluent Release Report.
- Clarified high radiation area boundary requirements from "roped off" areas to "barricaded" areas.

The supplemental information for these changes is included in the following attachments:

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| Attachment 1 - | Summary of Proposed Changes |
| Attachment 2 - | Proposed Decommissioning Technical Specifications, revised pages, with margin marks to show changes from the previous submittal |
| Attachment 3 - | Safety Analysis, justifying changes from previously submitted Decommissioning Technical Specifications |
| Attachment 4 - | No Significant Hazards Consideration Analysis |

Attachment 2 includes only those pages revised by this submittal. To assist in your review, changes from the previous submittal have been marked in the margin and are identified in Attachment 1.

The supplemental pages provided in Attachment 2, combined with the Decommissioning Technical Specifications previously submitted in the referenced letter, are intended to completely replace the existing FSV Technical Specifications. The No Significant Hazards Consideration Analysis provided in Attachment 4 addresses this total Technical Specification replacement, consistent with the decommissioning accident analyses in the FSV Proposed Decommissioning Plan. This analysis has not been affected by the revisions provided in this supplementary submittal. Accordingly, Attachment 4 is identical to the No Significant Hazards Consideration Analysis provided in the referenced letter, and is provided here for completeness.

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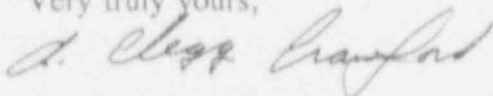
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PSC requests that the NRC approve the proposed Facility License and Decommissioning Technical Specifications submitted via the referenced letter, as supplemented herein. PSC requests that these requirements be effective upon issuance, and that a 14 day implementation period be provided, to allow all the associated procedures and other controls to be placed in effect.

If you have any questions regarding these proposed changes, please contact Mr. M. H. Holmes at (303) 620-1701.

Very truly yours,



A. Clegg Crawford
Vice President
Electric Production

ACC/SWC

cc: Regional Administrator, Region IV

Mr. J. B. Baird
Senior Resident Inspector
Fort St. Vrain

Mr. Robert M. Quillin, Director
Radiation Control Division
Colorado Department of Health