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GEORGE G. CREEL
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September 16, 1992

U. S. Nuclear Regulatory Commission
Washington, DC 20555

ATTENTION: Document Control Desk

SUBJECT: Calvert Cliffs Nuclear Power Plant
Unit No. 1; Docket No. 50-317
1992 Refueling Outage; Summary of Completed Licensing Actions

Gentlemen:

Enclosure (1) summarizes various open licensing actions for Baltimore Gas and Electric Company. The actions were completed during the Refueling Outage that ended August 16, 1992.

Should you have any further questions regarding this matter we will be pleased to discuss them with you.

Very truly yours,

for
G. C. Creel
Senior Vice President

GCC/JMO/dlm

Enclosure

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ENCLOSURE (1)

1992 REFUELING OUTAGE SUMMARY

SUBJECT: NRC Bulletin 90-01, Rosemount Transmitters (TAC No. M76537)

REFERENCE: Letters from BG&E to NRC, dated July 17, 1990 and April 29, 1992, same subject

As described in the referenced letters, all Unit 1 and Common Rosemount transmitters were replaced by August 31, 1992. Replacement transmitters were manufactured after July 11, 1989.

SUBJECT: NRC Generic Letter 88-17, Loss of Decay Heat Removal (TAC No. M69730)

REFERENCE: Letter from BG&E to NRC, dated January 27, 1989 and June 11, 1991, same subject

As described in the referenced letters, the following Unit 1 modifications were completed:

- a. Reactor Coolant System wide- and narrow-range level indications and alarms;
- b. Control Room Shutdown Cooling Flow indication alarm;
- c. Control Room Low Pressure Safety Injection suction pressure indication; and
- d. Core Exit Thermocouple audible alarm.

SUBJECT: Plant Modifications to Correct Control Room Human Engineering Deficiencies (HED) (TAC No. M56110)

REFERENCE: Letters from BG&E to NRC, dated July 6, 1990 and June 30, 1988, same subject

As described in the referenced letters, all Unit 1 HED modifications have been completed with the following exceptions:

FCR 88-208 Meters on plane B of Panels 17, 18 and 20 will be rearranged.

The FCR was cancelled for Units 1 and 2 as described in our letter dated March 6, 1993. These panel meters will be rearranged as part of installing two new Class 1E emergency diesel generators.

ENCLOSURE (1)

1992 REFUELING OUTAGE SUMMARY

FCR 88-210 Remount Control Room key lock switches that are not in the standard position.

The FCR was cancelled for Units 1 and 2 during an HED Assessment Committee meeting February 7, 1992. As described in our letter dated June 30, 1988, if reassessment of a previously determined corrective action is deemed necessary, it will be formally re-evaluated to like standards used during the original assessment. The committee concluded the modification was not necessary. The amount of time it takes to turn the key over from teeth-up to teeth-down or vice versa, is negligible compared to the time required to retrieve the key from the key locker.

FCR 88-218 Enhance Control Room lighting.

As described in our letter dated July 6, 1990, the modification will be performed concurrently to both units. It will be completed prior to the restart of Unit 2 from the Spring 1993 Unit 2 Refueling Outage.

FCR 88-225 Add Auxiliary Feedwater (AFW) Pump suction pressure indication to Control Room Panel 1/2 C04.

The FCR was cancelled for Units 1 and 2 during HED Assessment Committee meeting January 17, 1992. The committee concluded that the modification was not necessary. The only available point to measure suction pressure is at the common suction header. Consequently, operators would still be required to go to the AFW pump room to evaluate AFW flow problems. There is a low suction pressure alarm in the Control Room. Secondary supporting indications are displayed in the Control Room - valve line-up, discharge pressure and Condensate Storage Tank level.

SUBJECT: Reconsideration of ASME Code Repair of Pipe Flaw (TAC No. M81522)

REFERENCE: Letters from BG&E to NRC, dated April 25, 1991, and September 6, 1991, same subject

As described in the referenced letters, a flaw existed in a 6-inch weld-on inspection port to a 30-inch header down stream of the No. 12 Saltwater/Service Water Heat Exchanger. The header was replaced with new pipe.

ENCLOSURE (1)

1992 REFUELING OUTAGE SUMMARY

SUBJECT: Technical Specification Amendment to Isolation Valve Surveillance Requirement (TAC No. M82363)

REFERENCE: Letter from BG&E to NRC, dated December 31, 1991, same subject

As described in the referenced letter, MOV-644 was repaired during the recently completed Unit 1 outage. As we anticipated, only the valve stem was damaged. There was no damage to the valve seat or valve gate due to over-torquing.

SUBJECT: Increase in Auxiliary Feedwater (AFW) Actuation Delay Time (TAC No. M72075)

REFERENCE: Letter from NRC to BG&E, dated December 4, 1990, same subject

As described in the referenced letter, Technical Specification Amendment 149 was issued to increase AFW actuation delay time from 54.5 seconds to 180 seconds. FCR 89-026, which modifies the steam admission system including the time delay opening of control valves, was completed.

SUBJECT: Station Blackout (SBO) Modification Status (TAC No. M68525)

REFERENCE: Letter from BG&E to NRC, dated December 17, 1991, same subject

As described in the referenced letter, we have completed the SBO modifications associated with Unit 1. Specifically, we have provided DC-powered indication for some Unit 1 isolation valves and have provided AC power (through the inverters) to the Reactor Vessel Level Monitoring System. The remaining work can be performed during power operation or will be performed during the Unit 2 refueling outage (ninth refueling outage). The schedule for completion of all SBO modifications (except the addition of emergency diesel generators) has not changed.

ENCLOSURE (1)

1992 REFUELING OUTAGE SUMMARY

SUBJECT: Containment Pressure Instrument Tubing Modification (TAC No. M79911)

REFERENCE: Letter from BG&E to NRC, dated February 3, 1992, same subject

We have completed the modifications required to bring the Unit 1 pressure instrument tubing into conformance with approved seismic analyses. As discussed in the referenced letter, we determined that the modification could not be done safely during power operation and, therefore, took advantage of the refueling outage to complete the activity on Unit 1.