

NORTHEAST UTILITIES

THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

General Offices • Selden Street, Berlin, Connecticut

P.O. BOX 270
HARTFORD, CONNECTICUT 06141-0270
(203) 665-5000

May 30, 1985

Docket No. 50-336
A04840

Director of Nuclear Reactor Regulation
Attn: Mr. James R. Miller, Chief
Operating Reactors Branch #3
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

- References: (1) J. R. Miller letter to W. G. Council, dated April 18, 1985.
(2) W. G. Council letter to J. R. Miller, dated August 14, 1984.

Gentlemen:

Millstone Nuclear Power Station, Unit No. 2
Inadequate Core Cooling System
NUREG-0737, Item II.F.2

In Reference (1), Northeast Nuclear Energy Company (NNECO) was requested to provide additional information regarding the schedule for the implementation of emergency operating procedures (EOPs) related to the Inadequate Core Cooling (ICC) system. NNECO hereby informs the NRC Staff that appropriate revisions to our EOPs will be implemented within sixty (60) days after successful completion of the ICC system testing and calibration.

Section 3.0 of the NRC Staff's Safety Evaluation requests verification that the Class 1E backup displays will be available in the control room. The Class 1E backup displays are located in the old computer room which is adjacent to the control room and within the control room envelope. A saturation/superheat trouble alarm, core exit temperature (CET) high alarm, and low reactor level alarm, however, will be provided on the main control boards. We know of no NRC Staff criteria which necessitate the location of the backup displays in the control room proper. The Millstone Unit No. 2 Emergency Operating Procedures (EOPs) will be task analyzed as part of our ongoing Control Room Design Review (CRDR). Any human engineering discrepancies (HEDs) associated with the location of the backup displays, in the unlikely event of a loss of the primary display, will be adequately addressed as part of our CRDR efforts.

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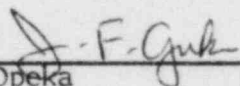
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Reference (1) does not mention the temporary use of an independent and isolated CRT to provide the primary display capability for the ICC system since the ultimate primary display (i.e., the Safety Parameter Display System) will not be installed by the end of the current refueling outage. We presume Reference (1) is based upon the final configuration of the ICC system as described to the NRC Staff in Reference (2).

We trust that this submittal adequately responds to Reference (1).

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY



J. F. Opeka
Senior Vice President