

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

APRIL 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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## I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to an artificial cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982, and commercial power operation was commenced on January 1, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

April 7-11

April 7, 1515 hours - Reactor Critical  
April 8, 0830 hours - Generator Synchronized to the  
grid  
April 10, 2300 hours - Reactor power at 81%.  
April 11, 2235 hours - Reactor scram on turbine trip  
due to high vibration on bearing  
number 11.

The reactor had been critical for 103 hours and 20  
minutes in April.

April 13-20

April 13, 0212 hours - Reactor Critical.  
April 13, 0900 hours - Generator Synchronized to the  
grid.  
April 14, 2300 hours - Reactor power at 71%  
April 15, 0345 hours - Reactor power reduced to 57% in  
order to manipulate control  
rods.  
April 17, 0700 hours - Reactor power at 93%.  
April 21, 1130 hours - Reactor power reduced to 75% in  
order to manipulate control rods.  
April 22, 2300 hours - Reactor power at 99%.  
April 28, 0140 hours - Reactor power reduced to 53% in  
order to manipulate control rods.  
April 30, 2300 hours - Reactor power at 98%

The reactor was critical for a total of 532 hours and 8  
minutes in April.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no amendments to the facility license or Technical Specification during this reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L48035	Outboard Feed Water Check Valve "B"	Air leak on supply to solenoid valve.	Possibility to hinder attempts to open this valve using air pressure.	Repaired leak.
L46617	Diesel Generator "A"	Malfunctioning shutdown solenoid.	Could not shutdown diesel.	Repaired and adjusted shutdown solenoid.
L48043	RHR "B" Full Flow Test Valve.	Limit switches out of adjustment.	Loss of "Open" indication.	Adjusted limit switches
L47883	Deluge Valves to Unit One SBT train.	Leaking Valve.	Water leaked into charcoal causing possibly degraded SBT performance.	Rebuilt deluge valves and leak tested charcoal.
L26597	Outboard Feed Water Check Valve "A".	Limit switches out of adjustment.	Improper Valve position indication.	Adjusted limit switches.
L26596	Outboard Feed Water Check Valve "B".	Limit switches out of adjustment.	Improper Valve position indication.	Adjusted limit switches.
L47827	ADS Valve Recorder point Number 19.	Faulty equipment.	Temperature indicated upscale.	Repaired faulty equipment.
L48158	Division II Post Hydrogen and Oxygen Monitor.	Monitor out of calibration	Incorrect indication of Hydrogen and Oxygen Concentration.	Recalibrated Monitor.
L48075	Division I Post LOCA Hydrogen and Oxygen Monitor.	Monitor out of calibration	Incorrect indication of Hydrogen and Oxygen concentration.	Recalibrated Monitor
L48199	Control Rod 22-43.	Faulty equipment.	Rod periodically tripped.	Repaired faulty equipment.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L48041	RHR "B" Flow Transmitter.	Out of calibration.	No flow indication.	Recalibrated Transmitter.
L48412	Ammonia Detector for "A" Control Room HVAC train.	Faulty cassette mechanism	Alarmed continuously.	Repaired cassette mechanism.
L48031	Feed Water Stop Valve "A"	Faulty Valve.	Valve would not close.	Repaired valve.
L47804	Outboard Feed Water Check Valve "B"	Faulty seats.	Excessive leakage.	Changed seats.
L48167	HCU Accumulator for CRD 38-55.	Leaking isolation valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L48037	HCU Accumulator for CRD 02-43.	Leaking isolation valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L47803	Inboard Feed Water Check Valve "A"	Faulty Valve.	Excessive leakage.	Repaired Valve.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, April 1 through April 30, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-027-00	3-13-85	Unsecured High Radiation Area
85-028-00	3-27-85	Three Safety Relief Valves opened and reactor scrammed on low level.
85-029-00	3-21-85	Reactor Scram due to Instrument valving error.
85-030-00	3-25-85	Ammonia Detector ESF Actuation.
85-031-00	3-11-85	Discharge of Radwaste Discharge Tank 2WF05T with incorrect Setpoint.
85-032-00	3-28-85	Inboard Feedwater Check Valve Local Leak Rate Test Failure
85-033-00	4-2-85	Missed Noble Gas Sample for Unit One standby Gas Treatment System.
85-034-00	3-30-85	1E12-F009 Valve Isolation.
85-035-00	4-11-85	Main Turbine High Vibration Trip and Reactor Scram.
85-036-00	4-3-85	Shutdown Cooling System Isolation on Suction Line High Flow



D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373

UNIT LaSalle One

DATE May 9, 1985

COMPLETED BY Richard J. Rohrer

TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: April, 1985 GROSS HOURS IN REPORTING PERIOD: 719
  2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
  3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
  4. REASONS FOR RESTRICTION (IF ANY): N/A
- |  | THIS MONTH     | YR TO DATE     | CUMULATIVE      |
|--|----------------|----------------|-----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL    | <u>532.1</u>   | <u>2302.2</u>  | <u>8583</u>     |
| 6. REACTOR RESERVE SHUTDOWN HOURS          | <u>27.6</u>    | <u>98.6</u>    | <u>1265</u>     |
| 7. HOURS GENERATOR ON LINE                 | <u>508.1</u>   | <u>2229.4</u>  | <u>8285</u>     |
| 8. UNIT RESERVE SHUTDOWN HOURS             | <u>0.0</u>     | <u>0.0</u>     | <u>0.0</u>      |
| 9. GROSS THERMAL ENERGY GENERATED (MWH)    | <u>1407910</u> | <u>6249577</u> | <u>23072866</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH)     | <u>460260</u>  | <u>2064310</u> | <u>7534953</u>  |
| 11. NET ELEC. ENERGY GENERATED (MWH)       | <u>443442</u>  | <u>1984684</u> | <u>7179746</u>  |
| 12. REACTOR SERVICE FACTOR                 | <u>74.0%</u>   | <u>79.3%</u>   | <u>73.4%</u>    |
| 13. REACTOR AVAILABILITY FACTOR            | <u>77.8%</u>   | <u>82.7%</u>   | <u>84.3%</u>    |
| 14. UNIT SERVICE FACTOR                    | <u>70.7%</u>   | <u>76.8%</u>   | <u>70.9%</u>    |
| 15. UNIT AVAILABILITY FACTOR               | <u>70.7%</u>   | <u>76.8%</u>   | <u>70.9%</u>    |
| 16. UNIT CAPACITY FACTOR (USING MDC)       | <u>59.5%</u>   | <u>66.0%</u>   | <u>59.3%</u>    |
| 17. UNIT CAPACITY FACTOR(USING DESIGN MWe) | <u>57.2%</u>   | <u>63.4%</u>   | <u>57.0%</u>    |
| 18. UNIT FORCED OUTAGE RATE                | <u>29.3%</u>   | <u>15.1%</u>   | <u>14.7%</u>    |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)  
Unit one is scheduled for a refueling, maintenance, modification, and surveillance outage beginning September 3, 1985 and lasting 26 weeks.
  20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373

UNIT: LASALLE ONE

DATE: May 9, 1985

COMPLETED BY: Richard J. Rohrer

TELEPHONE: (815) 357-6761

MONTH: APRIL, 1985

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1. _____ -14 _____	17. _____ 1000 _____
2. _____ -13 _____	18. _____ 995 _____
3. _____ -14 _____	19. _____ 1046 _____
4. _____ -14 _____	20. _____ 1039 _____
5. _____ -15 _____	21. _____ 910 _____
6. _____ -16 _____	22. _____ 1010 _____
7. _____ -15 _____	23. _____ 1073 _____
8. _____ 124 _____	24. _____ 1052 _____
9. _____ 527 _____	25. _____ 1045 _____
10. _____ 787 _____	26. _____ 1028 _____
11. _____ 721 _____	27. _____ 1027 _____
12. _____ -14 _____	28. _____ 701 _____
13. _____ 309 _____	29. _____ 876 _____
14. _____ 716 _____	30. _____ 1020 _____
15. _____ 747 _____	
16. _____ 836 _____	

## ATTACHMENT E

## 3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April 1985DOCKET NO. 050-373UNIT NAME LaSalle OneDATE May 9, 1985COMPLETED BY Richard J. RohrerTELEPHONE (815)357-6761

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
6	850321	F	176.50	G	3	Continuation of outage begun on March 21, 1985. Reactor Scrammed on Low Vessel Level and High Vessel pressure signals due to a valving error during installation of an Environmental Qualification Modification.
7	850411	F	34.42	A	3	Reactor scram on Turbine trip due to high vibrations in bearing number 11.
8	850428	S	0.00	H	1	Power reduced to manipulate control rods.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO &amp; TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
4-11-85	1B21-F013E	1Automatic	978 psig	Actuated to relieve pressure following reactor scram.

## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
0-265-85	Diesel Generator "0" Feed Breaker	Replace Puffer Pistons.
0-266-85	Diesel Generator "0"	Inspection
0-267-85	Diesel Generator "0"	Recalibration
0-268-85	ODO01P	Surveillance
0-269-85	Diesel Generator "0"	Lubrication
0-270-85	Diesel Generator "0"	Recalibrations
0-272-85	Diesel Generator "0" Air Compressors	Lubrication
0-278-85	Diesel Generator "0"	Repair Air Start Solenoid
1-319-85	1E12-N010CA/CB	Repipe switches
1-320-85	1E22-N012 A/B	Repipe Switches
1-326-85	1DG08CB	Repair Fitting
1-345-85	1E12-F024B	Repair Limit Switches
1-355-85	LPCS Minimum Flow Valve	Technical Specification Requirement
1-366-85	1E12-F053A	Technical Specification Requirement
1-367-85	1E12-F053A	Environmental Qualification Modification (EQ Mod).
1-377-85	1E21-N004	Rotate Switch
1-378-85	LPCS Minimum Flow Valve	Technical Specification Requirement.
1-383-85	1E12-C003	Lubrication
1-385-85	Shutdown Cooling Suction Header	Requirement of Special Test LTS-85-45.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-390-85	RCIC	Remove Governor
1-391-85	RCIC	Change oil

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

APRIL 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18



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    - 1. Safety/Relief Valve Operations
    - 2. ECCS System Outages
    - 3. Off-Site Dose Calculation Manual Changes
    - 4. Major Changes to Radioactive Waste Treatment System

## I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to an artificial cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984, and commercial power operation was commenced on June 19, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

APRIL 1-30

The reactor was subcritical, and the generator was off-line for the entire month of April. Unit Two is in a scheduled outage for maintenance, testing, and modifications.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

There were no amendments to the facility operating license or Technical Specifications during this reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L48422	5-Valve manifold for 2E12-N010CB.	Plugged Equalizing line.	Difficulty in calibration or maintenance of 2E12-N010CB.	Replaced 5-Valve manifold.
L48202	RHR "B" Heat Exchanger Vent Valve.	Faulty indicator.	Loss of position indication.	Repaired indicator.
L47681	Unit Two 250 Volt batteries.	Gaps between batteries and side rails exceeded seismic specifications.	Possible seismic hazard.	Placed material between batteries and siderails.
L47325	MSIV Leakage Control Valve.	Rotor and Finger base cracked.	Would not operate.	Replaced rotors and finger base.
L46895	RHR Temperature Recorder.	Dirty selector switch.	Points 3,5, and 6 sporadically printed upscale.	Cleaned selector switch.
L48304	Diesel Generator "A" Air Compressor "B"	Faulty relief valve.	Sporadically cycling relief valve.	Repaired valve.
L48026	LPCI Manual Injec- tion Valve "A".	Broken Manual Actuator.	Handwheel rotated without stroking valve.	Repaired actuator.
L40429	HCU Accumulator for CRD 22-55.	Leaking isolation valve and fitting.	Potential to cause failure to scram this rod if combined with other events.	Cleaned valve and re- placed fitting.
L43534	2E12-F064C	Faulty packing.	Valve leaked at packing.	Repaired packing.
L46964	Reactor Well Drain Downstream Isolation Valve.	Faulty Valve.	Excessive leakage.	Repaired valve.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, March 1 through March 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-012-00	3-31-85	Inadvertant RHR Shutdown Cooling Isolation.
85-013-00	3-28-85	Loss of Shutdown Cooling due to Group 6 Division 2 Isolation.
85-014-00	4-9-85	Spurious Division I Isolation of Shutdown Cooling.
85-015-00	4-1-85	Loss of Continuous Conductivity Indication.
85-016-00	4-18-85	Missed Surveillance of 2E51-F091.
85-017-00	4-13-85	Group VI Isolation.
85-018-00	4-13-85	Shutdown Cooling Isolation on Blown Fuse.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

## 1. OPERATING DATA REPORT

DOCKET NO. 050-374UNIT LaSalle TwoDATE May 9, 1985COMPLETED BY Richard J. RohrerTELEPHONE (815)357-6761

## OPERATING STATUS

1. REPORTING PERIOD: April, 1985 GROSS HOURS IN REPORTING PERIOD: 719
  2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
  3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
  4. REASONS FOR RESTRICTION (IF ANY):
- |   | THIS MONTH   | YR TO DATE     | CUMULATIVE     |
|---|--------------|----------------|----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL     | <u>0.0</u>   | <u>1399.8</u>  | <u>3011.6</u>  |
| 6. REACTOR RESERVE SHUTDOWN HOURS           | <u>0.0</u>   | <u>0.0</u>     | <u>125.3</u>   |
| 7. HOURS GENERATOR ON LINE                  | <u>0.0</u>   | <u>1397.3</u>  | <u>2934.7</u>  |
| 8. UNIT RESERVE SHUTDOWN HOURS              | <u>0.0</u>   | <u>0.0</u>     | <u>0.0</u>     |
| 9. GROSS THERMAL ENERGY GENERATED (MWH)     | <u>0.0</u>   | <u>4387385</u> | <u>8894977</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH)      | <u>0.0</u>   | <u>1460387</u> | <u>2945373</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH)        | <u>-8965</u> | <u>1391590</u> | <u>2783707</u> |
| 12. REACTOR SERVICE FACTOR                  | <u>0.0%</u>  | <u>48.2%</u>   | <u>64.4%</u>   |
| 13. REACTOR AVAILABILITY FACTOR             | <u>0.0%</u>  | <u>48.2%</u>   | <u>67.0%</u>   |
| 14. UNIT SERVICE FACTOR                     | <u>0.0%</u>  | <u>48.1%</u>   | <u>62.7%</u>   |
| 15. UNIT AVAILABILITY FACTOR                | <u>0.0%</u>  | <u>48.1%</u>   | <u>62.7%</u>   |
| 16. UNIT CAPACITY FACTOR (USING MDC)        | <u>-1.2%</u> | <u>46.3%</u>   | <u>57.4%</u>   |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>-1.2%</u> | <u>44.5%</u>   | <u>55.2%</u>   |
| 18. UNIT FORCED OUTAGE RATE                 | <u>0.0%</u>  | <u>0.0%</u>    | <u>5.1%</u>    |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
An outage for maintenance and surveillance was begun at 0520 on February 28, 1985.
  20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP 5-30-85



2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374

UNIT: LASALLE TWO

DATE: May 9, 1985

COMPLETED BY: Richard J. Rohrer

TELEPHONE: (815) 357-6761

MONTH: April 1985

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	-14
2.	-12
3.	-13
4.	-13
5.	-14
6.	-15
7.	-14
8.	-14
9.	-12
10.	-12
11.	-13
12.	-13
13.	-13
14.	-13
15.	-13
16.	-12

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	-13
18.	-12
19.	-12
20.	-13
21.	-13
22.	-12
23.	-12
24.	-11
25.	-11
26.	-11
27.	-11
28.	-10
29.	-13
30.	-11

## ATTACHMENT E

## 3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374

UNIT NAME LaSalle Two

DATE MAY 9, 1985

COMPLETED BY Richard J. Rohrer

TELEPHONE (815)357-6761

REPORT MONTH APRIL 1985

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
3	850228	S	719.00	B	2	Maintenance and Surveillance Outage begun 2-28-85 continues

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES</u> <u>ACTUATED</u>	<u>NO &amp; TYPE</u> <u>ACTUATIONS</u>	<u>PLANT</u> <u>CONDITION</u>	<u>DESCRIPTION</u> <u>OF EVENT</u>
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There were no safety relief valves actuated during this reporting period.

## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-487-85	RCIC Trip and Throttle Repack Limit Switch Valve.	
2-515-85	2E12-N010B and 2E12-N015B	Environmental Qualification Modification (EQ Mod).
2-516-85	2E12-N015C and 2E12-N010C	EQ Mod
2-517-85	2E12-N016B and 2E12-N019B	EQ Mod
2-518-85	2E12-N016 C and 2E12-N019C	EQ Mod
2-519-85	Division II.	Equipment Protection and Technical Specification Requirement.
2-528-85	RCIC transmitter.	EQ Mods.
2-532-85	2E12-F065A	Reposition Solenoid.
2-533-85	2E12-F065B	Replace Solenoid.
2-534-85	2E51-F054	Reposition Solenoid
2-535-85	2E12-F064C	Repack Valve.
2-536-85	Residual Heat Removal "C"	Change Oil.
2-537-85	Diesel Generator "A"	Calibration.
2-538-85	Diesel Generator "A"	Inspection
2-539-85	Disel Generator "A"	Inspection
2-540-85	Diesel Generator "A" Feed to Bus 242Y.	Replace Puffer Piston.
2-541-85	Diesel Generator "A"	Lubrication.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-542-85	Residual Heat Removal "B" Service Water Process Radiation Monitor	Cleaning
2-543-85	2E12-F006 A and B	Verify Jumpers
2-544-85	Diesel Generator "A" Fuel Oil Transfer Pump	Surveillance
2-545-85	Diesel Generator "A" Cooling Water Pump	Lubrication
2-547-85	Residual Heat Removal "B" Full Flow Test Valve.	Repair Limitorque
2-548-85	Residual Heat Removal "B" Full Flow Test Valve.	Repair Limitorque
2-553-85	2E22-F005 and 2E51-F066	Reposition Solenoids.
2-568-85	Division III Batteries.	Paint battery rack.
2-579-85	Safety Relief Valve "E".	Replace Valve
2-580-85	Safety Relief Valve "E"	Replace Valve
2-581-85	Safety Relief Valve "R"	Replace Valve.
2-582-85	Safety Relief Valve "R"	Replace Valve.
2-587-85	Residual Heat Removal "B" and "C"	Surveillance
2-588-85	Residual Heat Removal "B" and "C" Water Leg Pump.	Lubrication
2-590-85	Diesel Generator "A" Air Compressor "B" Relief Valve.	Limit cycling.
2-597-85	Diesel Generator "A" Air Compressor "B" Relief Valve	Repair Valve.
2-601-85	Residual Heat Removal "B" Minimum Flow Valve	Calibration
2-606-85	HPCS Pump	Inspect Coupling

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-611-85	Diesel Generator "A" Air Compressor "B"	Repair Relief Valve
2-623-85	Residual Heat Removal "B" and "C"	Surveillance
2-639-85	2E12-N010CB	Replace 5-Valve Manifold
2-640-85	2E12-F064B	Surveillance
2-647-85	Residual Heat Removal "C" Flow Transmitter	Replace Valve Manifold
2-650-85	Shutdown Cooling	Surveillance
2-652-85	2E12-F041C	Replace Control Switch
2-664-85	2E21-F006	Reposition Solenoid
2-667-85	LPCS Pump	Repair Puffer Piston
2-668-85	2E12-N001 and 2E12-N009	EQ Mod
2-669-85	2E12-N016A and 2E12-N019A	EQ Mod
2-670-85	2E21-N004	EQ Mod
2-674-85	2E12-N010A	EQ Mod
2-676-85	Residual Heat Removal "A" Shutdown Cooling.	Calibrations.
2-677-85	Residual Heat Removal "A" Minimum Flow Valve	EQ Mod
2-678-85	LPCS Minimum Flow Valve.	EQ Mod
2-689-85	Residual Heat Removal Pump "A"	Sample Oil
2-691-85	Diesel Generator "A" Air Compressor "B"	Repair relief valve

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the radioactive waste treatment system during this reporting period.



**Commonwealth Edison**  
LaSalle County Nuclear Station  
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May 9, 1985

Director, Office of Management Information  
and Program Control  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering  
LaSalle County Nuclear Power Station for the period covering April 1 through  
April 30, 1985.

Very truly yours,

G. J. Diederich  
Station Manager  
LaSalle County Station

GJD/RJR/crh

Enclosure

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NRC Resident Inspector LaSalle  
Gary Wright, Ill. Dept. of Nuclear Safety  
D. P. Galle, CECO  
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