

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

DOCKET NUMBER (2)

PAGE (3)

Wolf Creek Generating Station

0 5 0 0 0 4 8 1 2 1 OF 0 1 2

TITLE (4)

ESF Actuation - Containment Purge, Fuel Building, and Control Room Ventilation Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	4	1	2	8	5	0	0	9	0	0	0
0	4	1	2	8	5	0	0	0	5	1	0
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)								
5			20.402(b)			20.406(c)			X 50.73(a)(2)(iv)		
POWER LEVEL (10)			20.406(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)		
01 01 0			20.406(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)		
			20.406(a)(1)(iii)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(A)		
			20.406(a)(1)(iv)			50.73(a)(2)(iii)			50.73(a)(2)(viii)(B)		
			20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)		
									73.71(b)		
									73.71(c)		
									OTHER (Specify in Abstract below and in Text, NRC Form 366A)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Merlin G. Williams - Superintendent of Regulatory, Quality, and Administrative Services	AREA CODE 3 1 6 3 6 4 - 8 8 3 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
B				N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/>	<input checked="" type="checkbox"/>				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1114 CST on April 12, 1985, an Engineered Safety Features Actuation Signal (ESFAS) was initiated when several radiation monitors were deenergized due to a ground fault indication tripping a power supply breaker. This resulted in a Containment Purge Isolation Signal (CPIS), Fuel Building Ventilation Isolation Signal (FBVIS) and Control Room Ventilation Isolation Signal (CRVIS). All required engineered safety features equipment responded properly except for SGK05A, a Class 1E Electrical Equipment Air Conditioning Unit, which had previously been taken out of service for maintenance.

The breaker was reset and power restored to the affected radiation monitors at 1247.

The plant was in Mode 5 prior to initial criticality at the time of the event and redundant radiation monitors were operable.

No radiation above normal background was present and this event posed no threat to the public health and safety.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)  Wolf Creek Generating Station	DOCKET NUMBER (2)  0 5 0 0 0 4 8 2 8 5 - 0 0 9 - 0 0 0 2 of 0 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1114 CST on April 12, 1985, a 480 volt feeder breaker to the NG02B 120v distribution panel tripped on a ground fault indication causing loss of power to several containment, fuel building and control room radiation monitors. The affected radiation monitors were GT-RE-31 (Containment atmosphere monitors), GT-RE-33 (Containment purge noble gas monitor), GG-RE-28 (Fuel Building exhaust monitor) and GK-RE-04 (Control Room intake monitor). This resulted in a Containment Purge Isolation Signal (CPIS), Fuel Building Ventilation Isolation Signal (FBVIS) and Control Room Ventilation Isolation Signal (CRVIS). All required engineered safety features equipment responded properly except SGK05A, a Class 1E Electrical Equipment Air Conditioning Unit, which had previously been taken out of service for maintenance.

The plant was in Mode 5 prior to initial criticality at the time of the event and redundant radiation monitors were operable.

Troubleshooting identified a current imbalance between legs at the 480 volt feeder breaker as the probable cause of the breaker trip. The breaker was reset and power restored to the radiation monitors at 1247. There have been no further trips of the breaker since it was reset.

Restoration of the actuated systems to a normal configuration in accordance with plant procedures was completed at 1855.

This event posed no threat to the public health and safety.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER  
VICE PRESIDENT - NUCLEAR

May 10, 1985

COPY FOR

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Mr. R.P. Denise, Director  
Wolf Creek Task Force  
U.S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive, Suite 1000  
Arlington, Texas 76011

KMLNRC 85-108  
Re: Docket No. STN 50-482  
Subj: Licensee Event Report 85-009-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a)(2)(iv) concerning an Engineered Safety Feature Actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

*Glenn L. Koester*

GLK:bb  
Enc.  
xc:PO'Connor (2), w/a  
JCummins, w/a

IE22  
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