

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) PALISADES PLANT										DOCKET NUMBER (2) 0 5 0 0 0 2 5 5					PAGE (3) 1 OF 0 2	
TITLE (4) Inability to Perform Reactor Internals Vibration Monitoring																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 2	1 8	8 5	8 5	0 0 3	0 0	0 5	0 7	8 5	N/A				0 5 0 0 0			
									N/A				0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)														
POWER LEVEL (10)		20.402(b)				20.406(c)				80.73(a)(2)(iv)				73.71(b)		
0 1 9 1 7		20.406(a)(1)(i)				80.36(a)(1)				80.73(a)(2)(v)				73.71(a)		
		20.406(a)(1)(ii)				80.36(a)(2)				80.73(a)(2)(vi)				<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 306A)		
		20.406(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(vii)(A)				Voluntary		
		20.406(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(vii)(B)						
		20.406(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME										TELEPHONE NUMBER						
James G. Lewis, Technical Director, Palisades										AREA CODE						
										6 1 6		7 6 4 - 8 9 1 3				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS						
X	J H	C P V	H 1 6 5	N												
SUPPLEMENTAL REPORT EXPECTED (14)																
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO		EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Technical Specification surveillance requirement 4.13 (c) requires reactor internals vibration monitoring to be performed, as a minimum, once per 7 days for Phase I measurements and one per 31 days for Phase II measurements. As a result of hardware failures in data processing equipment, vibration monitoring was not performed for a weekly test in late November, 1984 and has not been obtained, as required, since February 11, 1985.

The equipment necessary to record the vibration data has been procured. A vendor was located with the capability to reduce the recorded data. A Technical Specification change request was submitted to eliminate the reactor internals vibration monitoring requirements. The Director, Nuclear Reactor Regulation, was notified of the current difficulties with vibration monitoring and the inability to satisfy the surveillance requirements.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) PALISADES PLANT	DOCKET NUMBER (2) 0 5 0 0 0 2 5 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	0 0 3	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 305A's) (17)

Technical Specification 4.13 (c) requires reactor internals vibration monitoring to be performed, as a minimum, once per 7 days for Phase I measurements and once per 31 days for Phase II measurements. As a result of hardware failures in data processing equipments, vibration monitoring was not performed for a weekly test in late November, 1984 and has not been obtained, as required, since February 1985.

The reactor internals vibration monitoring surveillance is recorded and analyzed with a Hewlett-Packard Fourier Analyzer [JH, CPU]. The analyzer was repaired after difficulties encountered in November, 1984, but since February, 1985, all attempts to reestablish vibration monitoring capabilities with the existing equipment have been unsuccessful.

As an interim measure, the equipment necessary to record the internals vibration data has been procured. A vendor has been located with the capability to reduce the recorded data. The surveillance will be performed in this manner on a 90 day interval such that the requirements of ASME Standard OM-05, 1981, will be satisfied. The Director, Nuclear Reactor Regulation, was notified of the methods and intervals to be applied to this surveillance.

A Technical Specification change was requested on March 29, 1985 to eliminate the internals vibration monitoring requirements. The testing will continue on a 90 day interval as recommended by ASME Standard OM-05, 1981. The surveillance will be administratively controlled external to Technical Specifications.

The Palisades Plant reported neutron noise that was felt to be indicative of core barrel motion in June of 1973. A subsequent reactor internals inspection revealed internals wear due to a loss of the core barrel clamping force. Modifications were provided and the internals monitoring instituted to establish the adequacy of the design change. No further problems have been identified.

The failure to complete the vibration monitoring, although contrary to Technical Specifications, did not result in any Plant operation prohibited by Technical Specifications. Since no Limiting Condition of Operation or Action Statement is provided, a condition prohibited by Technical Specification did not occur. The reporting criteria of 10CFR50.73 are not considered applicable to this occurrence. Therefore, this report has been provided as a voluntary Licensee Event Report.



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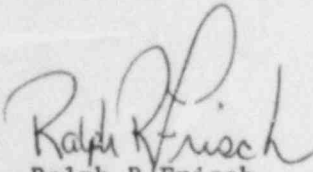
General Offices: 1945 West Parnall Road, Jackson, MI 49201 • (517) 788-0550

May 7, 1985

Nuclear Reactor Regulation
US Nuclear Regulatory Commission
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 - PALISADES PLANT -
LICENSEE EVENT REPORT 85-003 - INABILITY TO PERFORM REACTOR INTERNALS
VIBRATION MONITORING (VOLUNTARY REPORT)

Licensee Event Report (LER) 85-003 (Inability to Perform Reactor Internals Vibration Monitoring) is attached. This event is being reported to the NRC as a voluntary report. We have determined that there is not a specific 10CRF50.73 reporting requirement applicable to this event. Our basis for this determination is specified in the Federal Register notice (48 FR 33858, July 26, 1983) which published the final rule which states the scope is reporting events or conditions prohibited by Technical Specifications rather than events where a Technical Specification action statement is not met. The vibration monitoring surveillance is an example of Technical Specifications that were not met but whereby the Plant was operated within Technical Specifications in that a condition (ie Limiting Condition for Operation (LCO) or Action Statement) provided in Technical Specifications was not exceeded. This particular Technical Specification does not have a LCO or an Action Statement. A change to the Palisades Technical Specifications was submitted by letter dated March 29, 1985.


Ralph R Frisch
Senior Licensing Analyst

CC Administrator, Region III, USNRC
NRC Resident Inspector - Palisades

Attachment

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