

APPROVED OMB NO. 3150-0104  
EXPIRES: 8/31/85

FACILITY NAME (1) <b>EDWIN I. HATCH, UNIT II</b>														DOCKET NUMBER (2) <b>0 5 0 0 0 3 6 6 1 OF 0 4</b>						PAGE (3) <b>1</b>	
TITLE (4) <b>FAILURE OF VALVES TO PASS LOCAL LEAK RATE TEST</b>																					
EVENT DATE (5)			LER NUMBER (8)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (6)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)								
													<b>0 5 0 0 0</b>								
<b>0 4 0</b>	<b>9 8</b>	<b>5</b>	<b>8 5</b>	<b>0 1 0</b>	<b>0 0</b>	<b>0 5</b>	<b>0 3</b>	<b>8 5</b>					<b>0 5 0 0 0</b>								
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																			
<b>5</b>		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)							
POWER LEVEL (10)		20.406(a)(1)(i)				50.36(c)(1)				<b>X</b> 50.73(a)(2)(v)				73.71(c)							
<b>0 0 1 0</b>		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)							
		20.406(a)(1)(iii)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(A)											
		20.406(a)(1)(iv)				<b>X</b> 50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)											
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME										TELEPHONE NUMBER											
<b>Steven B. Tipps, Superintendent of Regulatory Compliance</b>										AREA CODE		<b>9 1 2 3 6 7 7 8 5 1</b>									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS												
<b>X</b>	<b>B 0</b>	<b>I S V P 3</b>	<b>0 5</b>	<b>Y</b>	<b>X</b>	<b>L F</b>	<b>I S V Y</b>	<b>0 1 0</b>	<b>Y</b>												
<b>X</b>	<b>B B</b>	<b>I S V F 1</b>	<b>3 0</b>	<b>Y</b>	<b>X</b>	<b>S J</b>	<b>I S V A</b>	<b>5 8 5</b>	<b>Y</b>												
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
<b>X</b> YES (If yes, complete EXPECTED SUBMISSION DATE)												<b>0</b>	<b>7</b>	<b>0 1 8 5</b>							
NO																					

**ABSTRACT** (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During performance of the "PRIMARY CONTAINMENT PERIODIC TYPE B AND TYPE C LEAKAGE TESTS" procedure (42SV-TET-001-2) as required by Tech. Specs. section 4.6.1.2.d, plant personnel determined that certain valves (see narrative for details) were leaking in excess of the limits specified in Tech. Specs. sections 3.6.1.2.b and c and the ASME section XI criteria specified in the procedure.

These valves will be repaired and tested to ensure that they meet the requirements of Tech. Specs. and ASME section XI prior to the startup of the unit.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT II	0 5 0 0 0 3 6 6 8 5	—	0 1 0	—	0 0 0	2 OF 4	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is required by 10CFR50.73(a)(2)(ii) and 10CFR50.73(a)(2)(v)(c) due to the assumption that the plant did not meet the requirements of Tech. Specs. sections 3.6.1.2.b and 3.6.1.2.c, 10CFR50 appendix J and ASME section XI.

When local leak rate testing was being performed per the "PRIMARY CONTAINMENT PERIODIC TYPE B AND TYPE C LEAKAGE TESTS" procedure (42SV-TET-001-2), the unit was in cold shutdown for a refueling outage.

A. At the time of leak rate testing, plant personnel assumed that the leakage of the following valves was such that the plant could not meet the 0.60  $L_a$  requirement of Tech. Specs. section 3.6.1.2.b.1.

1. On 04/09/85, the RHR Return to Recirculation Isolation Valve (2E11-F015B) was tested.
2. On 04/12/85, the Vent Purge Return inboard Isolation Valve (2T48-F319) was tested.
3. On 04/15/85, the Primary Containment Service Air Isolation Valve (2P51-F651) was tested.

B. At the time of leak rate testing, plant personnel assumed that the leakage of the following valves was such that the plant could not meet the lesser than 0.009  $L_a$  requirement of Tech. Specs. section 3.6.1.2.b.2.

1. On 04/07/85, the Primary Feedwater outboard isolation valve (2B21-F077A) was tested.
2. On 04/08/85, the Primary Feedwater inboard isolation valves (2B21-F010A and 2B21-F010B) were tested.

C. At the time of testing, plant personnel assumed that the leakage of the following valves was such that the plant could not meet the requirements of the individual valve leakage rates specified as a result of ASME section XI.

1. On 04/07/85, the "A" RHR Testable Check Valve (2E11-F050A) was tested.
2. On 04/10/85, the "B" RHR Testable Check Valve (2E11-F050B) was tested.
3. On 04/11/85, the "B" RHR pump suction inboard Isolation Valve (2E11-F004B) was tested.

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TEXT (If more space is required, use additional NRC Form 365A's) (17)

The actual or potential safety consequences of these events have not been determined at this time. An engineering evaluation will be performed and an update report will be submitted.

These events could have affected other systems on Unit 2, but had no impact on the operation of Unit 1.

These are similar events as last reported on LER number 50-366/1984-04. The previous corrective action taken was sufficient at the time; however, valve failures of this type are recurrent.

The valves will be repaired and satisfactorily functionally tested prior to unit startup.

An update report will be filed within 30 days after unit startup explaining the causes of these events and the corrective actions taken.



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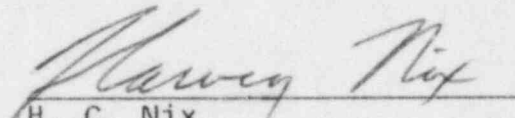
Edwin I. Hatch Nuclear Plant


May 3, 1985  
GM-85-391

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-366

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-010. This report is required by 10CFR50.73(a)(2)(ii) and 10CFR50.73(a)(2)(v).

  
H. C. Nix  
General Manager

  
HCN/SBT/vlz

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