



Nebraska Public Power District

COOPER NUCLEAR STATION
P.O. BOX 98, BROWNVILLE, NEBRASKA 68321
TELEPHONE (402)825-3811
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NLS960112

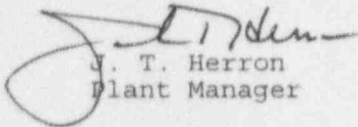
June 10, 1996

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555-0001

Dear Sir:

Cooper Nuclear Station Licensee Event Report 96-005 is forwarded as an attachment to this letter.

Sincerely,


J. T. Herron
Plant Manager

/cct

Attachment

cc: Regional Administrator
USNRC - Region IV

Senior Project Manager
USNRC - NRR Project Directorate IV-1

Senior Resident Inspector
USNRC

NPG Distribution

INPO Records Center

W. Turnbull
MidAmerica Energy

170015

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NRC FORM 366 <small>(4-95)</small>		U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 04/30/98 <small>ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.</small>																													
<h2 style="margin: 0;">LICENSEE EVENT REPORT (LER)</h2> <p style="margin: 5px 0;">(See reverse for required number of digits/characters for each block)</p>																																	
FACILITY NAME (1) Cooper Nuclear Station				DOCKET NUMBER (2) 05000298		PAGE (3) 1 OF 3																											
TITLE (4) Partial ESF Actuation of Containment Isolation Due to Personnel Error																																	
EVENT DATE (5) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th>MONTH</th> <th>DAY</th> <th>YEAR</th> </tr> <tr> <td style="text-align: center;">05</td> <td style="text-align: center;">09</td> <td style="text-align: center;">96</td> </tr> </table>			MONTH	DAY	YEAR	05	09	96	LER NUMBER (6) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th>YEAR</th> <th>SEQUENTIAL NUMBER</th> <th>REVISION NUMBER</th> </tr> <tr> <td style="text-align: center;">96</td> <td style="text-align: center;">-- 005</td> <td style="text-align: center;">-- 00</td> </tr> </table>			YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	96	-- 005	-- 00	REPORT DATE (7) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th>MONTH</th> <th>DAY</th> <th>YEAR</th> </tr> <tr> <td style="text-align: center;">06</td> <td style="text-align: center;">10</td> <td style="text-align: center;">96</td> </tr> </table>		MONTH	DAY	YEAR	06	10	96	OTHER FACILITIES INVOLVED (8) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th>FACILITY NAME</th> <th>DOCKET NUMBER</th> </tr> <tr> <td style="height: 20px;"></td> <td></td> </tr> <tr> <td style="height: 20px;"></td> <td></td> </tr> </table>		FACILITY NAME	DOCKET NUMBER				
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POWER LEVEL (10) 100		<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:33%;">20.2201(b)</td> <td style="width:33%;">20.2203(a)(2)(v)</td> <td style="width:33%;">50.73(a)(2)(i)</td> <td style="width:33%;">50.73(a)(2)(viii)</td> </tr> <tr> <td>20.2203(a)(1)</td> <td>20.2203(a)(3)(i)</td> <td>50.73(a)(2)(ii)</td> <td>50.73(a)(2)(x)</td> </tr> <tr> <td>20.2203(a)(2)(i)</td> <td>20.2203(a)(3)(ii)</td> <td>50.73(a)(2)(iii)</td> <td>73.71</td> </tr> <tr> <td>20.2203(a)(2)(ii)</td> <td>20.2203(a)(4)</td> <td><input checked="" type="checkbox"/> 50.73(a)(2)(iv)</td> <td>OTHER</td> </tr> <tr> <td>20.2203(a)(2)(iii)</td> <td>50.36(c)(1)</td> <td>50.73(a)(2)(v)</td> <td rowspan="2" style="vertical-align: top; font-size: small;">Specify in Abstract below or in NRC Form 366A</td> </tr> <tr> <td>20.2203(a)(2)(iv)</td> <td>50.36(c)(2)</td> <td>50.73(a)(2)(vii)</td> </tr> </table>					20.2201(b)	20.2203(a)(2)(v)	50.73(a)(2)(i)	50.73(a)(2)(viii)	20.2203(a)(1)	20.2203(a)(3)(i)	50.73(a)(2)(ii)	50.73(a)(2)(x)	20.2203(a)(2)(i)	20.2203(a)(3)(ii)	50.73(a)(2)(iii)	73.71	20.2203(a)(2)(ii)	20.2203(a)(4)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	OTHER	20.2203(a)(2)(iii)	50.36(c)(1)	50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A	20.2203(a)(2)(iv)	50.36(c)(2)	50.73(a)(2)(vii)				
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LICENSEE CONTACT FOR THIS LER (12)																																	
NAME Calvin C. Taylor, Licensing and Compliance Specialist				TELEPHONE NUMBER (Include Area Code) (402) 825-3811																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																	
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS																							
SUPPLEMENTAL REPORT EXPECTED (14)							EXPECTED SUBMISSION DATE (15)																										
YES <small>(If yes, complete EXPECTED SUBMISSION DATE).</small>				<input checked="" type="checkbox"/> NO		<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th>MONTH</th> <th>DAY</th> <th>YEAR</th> </tr> <tr> <td style="height: 20px;"></td> <td></td> <td></td> </tr> </table>		MONTH	DAY	YEAR				<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th>MONTH</th> <th>DAY</th> <th>YEAR</th> </tr> <tr> <td style="height: 20px;"></td> <td></td> <td></td> </tr> </table>		MONTH	DAY	YEAR															
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16) <p>At 2127 CDT on May 9, 1996, a partial Group 5 Containment Isolation was received causing RCIC-MOV-MO15 (Reactor Containment Isolation Cooling Steam Inboard Isolation) to close. A non-licensed station operator had removed the cover on relay RCIC-REL-K31 (Steam Line Space Excess Temperature Relay) to allow better observation of relay actuation during the performance of a RCIC steam line space temperature functional surveillance test. Removal or installation of the cover is not prescribed by the surveillance procedure. The Station Operator was reinstalling the cover when the relay mechanism was bumped causing the Group 5 isolation Channel B to trip. All systems functioned as required for the inadvertent ESF actuation.</p> <p>The Group 5 Channel B logic was reset at 2129 and RCIC-MOV-MO15 returned to the normal full open position at 2130. No other equipment actuated as a result of this signal and RCIC-MOV-MO15 remained off its open seat for approximately two minutes while operating personnel verified that a valid actuation signal was not present.</p> <p>The cause of this event is the failure of management to establish expectations concerning the removal of sensitive relay covers not prescribed by procedure, (NUREG 1022, Appendix B, Root Cause Code E, "Management/Quality Assurance Deficiency"). Operations management has communicated to Operations personnel that sensitive relay covers are not to be removed or installed outside of procedural guidance. Procedures that require the removal of sensitive relay covers for monitoring will be revised to provide the proper step sequence for the activity.</p>																																	

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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COOPER NUCLEAR STATION	05000298	96	-- 05	-- 00	2 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PLANT CONDITIONS

The plant was at 100% power operation during the event.

EVENT DESCRIPTION

At 2127 CDT on May 9, 1996, a partial Group 5 Containment Isolation was received causing RCIC-MOV-MO15 (RCIC Steam Inboard Isolation) to close. A non-licensed station operator had removed the cover on relay RCIC-REL-K31 (Steam Line Space Excess Temperature Relay) to allow better observation of relay actuation during the performance of a RCIC steam line space temperature functional surveillance test. Removal or installation of the cover is not prescribed by the surveillance procedure. The Turbine Building Station Operator was reinstalling the cover when the relay was bumped causing the Group 5 isolation Channel B to trip. All systems functioned as required for the inadvertent ESF actuation.

Control room personnel received annunciation of the actuation and observed RCIC-MO-MO15 in the intermediate position. The station operator was contacted and informed the Control Room staff that he had heard a relay click while reinstalling the cover on RCIC-REL-K31.

After verification of the cause and no valid ESF signals present, the Group 5 Channel B logic was reset at 2129 and RCIC-MOV-MO15 was throttled open to re-pressurize the steam line and returned to the normal full open position at 2130. No other equipment actuated as a result of this signal and RCIC-MOV-MO15 remained off its open seat for approximately two minutes.

CAUSE

The cause of this event is the failure of management to establish expectations concerning the removal of sensitive relay covers not prescribed by procedure, (NUREG 1022, Appendix B, Root Cause Code E, "Management/Quality Assurance Deficiency").

SAFETY SIGNIFICANCE

The RCIC system provides makeup water to the reactor vessel following a reactor vessel isolation in order to prevent the release of radioactive materials to the environs as a result of inadequate core cooling. During the time that the isolation signal was active (not reset), the RCIC system was incapable of performing its intended function without operator action. Technical Specification 3.5.D.2 requires that the HPCI system be operable if RCIC is made or found to be inoperable and during this time period HPCI was verified operable. Therefore, this event presented no adverse potential consequences for public health and safety.

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CORRECTIVE ACTIONS

Operations management has communicated to Operations personnel that sensitive relay covers are not to be removed and installed if not prescribed by procedure.

Procedures that require the removal of sensitive relay covers for monitoring will be revised to provide the proper step sequence for the activity.

PREVIOUS EVENTS

LER 88-006 reported an unplanned automatic actuation of Diesel Generator 1 starting logic when the normal power supply breaker tripped open. At the time of the event, a cover for an over current relay for the breaker was being replaced by a maintenance worker. The relay involved was a different type than RCIC-REL-(13A-K31) which is an HGA relay and involved a maintenance activity. The cause of the event was the target reset lever attached to the cover inadvertently contacting the over current relay seal-in contacts. Corrective actions were assigned to discuss the event with all relay technicians, revise appropriate procedures to alleviate the possibility of future inadvertent relay actuations during relay cover removal/replacement activities, and include the event in the appropriate craft training activities.

Correspondence No: NLS960112

The following table identifies those actions committed to by the District in this document. Any other actions discussed in the submittal represent intended or planned actions by the District. They are described to the NRC for the NRC's information and are not regulatory commitments. Please notify the Licensing Manager at Cooper Nuclear Station of any questions regarding this document or any associated regulatory commitments.

COMMITMENT	COMMITTED DATE OR OUTAGE
Procedures that require the removal of sensitive relay covers for monitoring will be revised to provide the proper step sequence for the activity.	