



GPU Nuclear Corporation

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June 11, 1996

6710-96-2198

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for May 1996

Enclosed are two copies of the May 1996 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

J. Knubel
Vice President and Director, TMI

170072

WGH

cc: Administrator, Region I
TMI Senior Resident Inspector
96001

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PDR ADOCK 05000289
R PDR

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OPERATIONS SUMMARY

May 1996

The plant entered the month operating at 100% power and remained at that power level for the remainder of the month. Net unit electrical output averaged approximately 807 MWe during May.

MAJOR SAFETY RELATED MAINTENANCE

There were no major safety related maintenance items completed during the month.

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE June 11, 1996
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

OPERATING STATUS

1. UNIT NAME: THREE MILE ISLAND UNIT 1
 2. REPORTING PERIOD: MAY 1996
 3. LICENSED THERMAL POWER: 2568
 4. NAMEPLATE RATING (GROSS MWe): 872
 5. DESIGN ELECTRICAL RATING (NET MWe): 819
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

THIS MONTH YR-TO-DATE CUMMULATIVE

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	3647.0	190632.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	3647.0	113190.1
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	3647.0	112028.3
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1,908,743.0	9,357,586.6	275,472,512.6
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	635,043.0	3,146,726.0	92,600,200.1
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	600,416.0	2,973,777.0	87,017,181.1
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	58.8
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	58.8
21. UNIT CAPACITY FACTOR (USING MDC NET)		102.7	103.7	58.1
22. UNIT CAPACITY FACTOR (USING DER NET)		98.5	99.6	55.7
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	35.1
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60761.2

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH): _____

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE June 11, 1996
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

MONTH: MAY

DAY AVERAGE DAILY POWER LEVEL
(MWe-NET)

1	812
2	808
3	808
4	807
5	808
6	810
7	813
8	811
9	808
10	804
11	801
12	815
13	818
14	817
15	814
16	813

DAY AVERAGE DAILY POWER LEVEL
(MWe-NET)

17	808
18	799
19	789
20	785
21	789
22	796
23	795
24	802
25	812
26	814
27	813
28	813
29	813
30	812
31	810

REPORT MONTH May 1996

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE June 11, 1996
 COMPLETED BY W. G. Heysek
 TELEPHONE (717) 948-8191.

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report#	System Code ⁴ & ⁵	Component Code ³ & ⁴	Cause & Corrective Action to Prevent Recurrence
						None			

¹
 F Forced
 S Scheduled

²
 Reason
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensing Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

⁴
 Exhibit G - Instructions for
 preparation of Data Entry Sheets
 for Licensee Event Report (LER)
 File (NUREG-0161)

⁵ Exhibit 1 same source

⁶ Actually used exhibits F & II NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 5, 1997**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **Yes. To support GPU Nuclear plans to do independent reload analyses for Cycle 12 as discussed in response to question 6 below, T.S. 6.9.5.2 would require revision to include references to the GPU Nuclear analysis methods applied to the reload.**
5. Scheduled date(s) for submitting proposed licensing action and supporting information: **A Technical Specification Change Request for the changes as discussed above would be submitted.**
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: **GPU Nuclear Letter 6710-96-2092, dated March 28, 1996 confirmed plans to perform independent reload design evaluations for Cycle 12, the next operation cycle, based on NRC approved methods described in GPU Nuclear Topical Reports TR-091 (core physics), TR-087 (core thermal hydraulics), TR-078 (FSAR safety analyses) and TR-092P (design and setpoints methodology) submitted to the NRC.**

The NRC has issued an SER for TR-071 on February 21, 1996 and has recently indicated that Generic Letter 83-11 Supplement 1 is on hold. Accordingly NRC review and approval is needed for each of the three remaining GPU Nuclear Topical Reports. A meeting with NRC staff reviewers was held on May 30, 1996 during which the Cycle 12 reload analysis schedules were discussed. At this time, completion of the NRC review and issuance of NRC SERs by the following critical approval dates is achievable:

TR-087	8/1/96
TR-078	10/1/96
TR-092P	10/1/96

The GPU Nuclear Cycle 12 reload program and results are expected to be available for NRC review in the March to April 1997 time frame.

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) **177** (b) **864**
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained. Phase II is expected to be started in 2002.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.