

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Peach Bottom Atomic Power Station - Unit 3

DOCKET NUMBER (2)

0 5 0 0 0 2 7 8

PAGE (3)

1 OF 3

TITLE (4)  
Automatic Start of E-1 and E-3 Diesel Generators

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER (8)
0	4	10	85	85	010	00	05	06		0 5 0 0 0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)										
OPERATING MODE (9)		N		20.402(b)		20.406(a)		N		60.73(a)(2)(iv)
POWER LEVEL (10)		0 9 0		20.406(a)(1)(ii)		60.34(a)(1)				72.71(b)
				20.406(a)(1)(iii)		60.34(a)(2)				72.71(c)
				20.406(a)(1)(iv)		60.73(a)(2)(ii)				OTHER (Specify in Abstract below and in Test, NRC Form 364J)
				20.406(a)(1)(v)		60.73(a)(2)(iii)				
				20.406(a)(1)(vi)		60.73(a)(2)(iv)				
				20.406(a)(1)(vii)		60.73(a)(2)(v)				
				20.406(a)(1)(viii)		60.73(a)(2)(vi)				
				20.406(a)(1)(ix)		60.73(a)(2)(vii)				
				20.406(a)(1)(x)		60.73(a)(2)(viii)				

LICENSEE CONTACT FOR THIS LER (12)

NAME  
J. C. Nagle, Engineer - Special Projects

TELEPHONE NUMBER

AREA CODE

2115 8411-151814

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) ☐ NO ☒

EXPECTED SUBMISSION DATE (15)

MONTH

DAY

YEAR

ABSTRACT (Limit to 1000 words, i.e., approximately fifteen single-spaced typewritten lines) (16)

Abstract: 3-85-010

On April 10, 1985, with Unit 3 at 90% power, the E-1 and E-3 diesel generators automatically started and the drywell coolers tripped after receiving a false reactor low level signal. Reactor power was decreased to 50% in anticipation of a possible accelerated shutdown. The signal was caused by defective test equipment used during performance of a surveillance test. After the initiation signal was cleared, the diesel generators were shut down and the drywell coolers were returned to normal operation.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 11/31/86

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of the Event:

On April 10, 1985, at 12:25 p.m., with Unit 3 at 90% power, the E-1 and E-3 emergency diesel generators automatically started upon receipt of a false reactor low level signal. E-1 and E-3 diesel generators started properly but were not required to load onto their respective buses because the buses were being supplied by the offsite power sources. After confirming that the initiation signal was false, both diesel generators were shut down and returned to their automatic start mode.

Coincident with the diesel generator automatic initiations, the drywell coolers received automatic trip signals. All drywell coolers that were in service at the time of the event shut down properly. As a result of losing drywell cooling, drywell pressure increased to approximately 0.92 psig. Reactor power was decreased to 50% in anticipation of a possible accelerated shutdown. After the trip signals were cleared, the drywell coolers were placed in service to return drywell pressure to normal.

Consequences of the Event:

All affected equipment functioned properly upon receipt of the inadvertent reactor low level signal. There were no adverse consequences.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

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FACILITY NAME (1)

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Peach Bottom Atomic  
Power Station - Unit 3

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7	8	8	5	-	0
1	0	-	0	0	0
0	3	OF	0	3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Cause of the Event:

At the time of the event, Instrument technicians were performing Surveillance Test ST-2.10.12 (Functional Check of ECCS A/C-1 Card File). During this test, special test equipment is connected into the 'A' core spray logic to monitor the status of two sets of series contacts. When the test equipment was connected into the circuit, a short circuit between two wires in the multi-pin connector produced a current path around the series contacts and caused relay 14A-K11A to energize. Energizing 14A-K11A starts the E-1 and E-3 diesel generators and trips the drywell coolers.

Failure of the test equipment was caused by frayed wire insulation on the test equipment cable.

Corrective Actions:

The defective test equipment was disassembled and repaired. All other similar test equipment was disassembled and inspected. No other similar problems were discovered. To prevent recurrence, the test equipment will be inspected once every six months. A calibration sticker has been attached to each of these test devices to identify the next required inspection date.

Previous Similar Occurrences:

None.

PHILADELPHIA ELECTRIC COMPANY

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P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

May 6, 1985

Docket No. 50-278

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U.S. Nuclear Regulatory Commission  
Washington, DC 20555

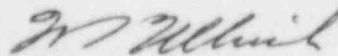
SUBJECT: Licensee Event Report  
Peach Bottom Atomic Power Station - Unit 3

This LER concerns the automatic actuation of E-1 and E-3 diesel generators and the temporary loss of drywell cooling.

Reference:	Docket No. 50-278
Report Number:	3-85-010
Revision Number:	00
Event Date:	April 10, 1985
Report Date:	May 6, 1985
Facility:	Peach Bottom Atomic Power Station RD #1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,



W. T. Ullrich  
Superintendent  
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator  
Region I, USNRC

Mr. T. P. Johnson, Resident Inspector

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