NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
R.E. Ginna Nuclear Power Plant	0 5 0 0 0 2 4 4	1 OF 0 2

TITLE (4)
Automatic Actuation of Reactor Protection System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 4	0 7	8 5	8 5	— 0 0 8	— 0 0	0 5	0 7	8 5		0 5 0 0 0
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OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 1 3		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
G.F. Larizza, Operations Manager	AREA CODE 3 1 5 5 2 4 - 4 4 4 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	XX NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 7, 1985, while controlling steam generator levels in manual and transferring control from the bypass valves to the main feedwater regulating valves, the reactor tripped on low steam generator level. The cause of the reactor trip has been attributed to Operations personnel being unable to maintain steam generator levels above the trip setpoints while transferring level control.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1039 on April 7, 1985, while controlling steam generator levels in manual and transferring control from the bypass valves to the main feedwater regulating valves, the reactor tripped on low steam generator level. The reactor trip was the result of the "A" steam generator level reducing below 17% narrow range level on two out of three channels. Reactor power, which was at approximately 13%, was being increased at that time to allow for increased steam generator blowdown, to enhance secondary plant chemistry cleanup prior to heating up the turbine. Following the reactor trip the "A" steam generator level reduced below 16% narrow range level and the pressurizer level reduced below 12% for a short period of time, which made the "A" reactor coolant loop (Technical Specification 3.1.1.1a and 4.3.5.5) and the pressurizer (Technical Specification 3.1.1.5) inoperable. All systems operated as designed and the reactor coolant system was stabilized at hot shutdown conditions.

The cause of the reactor trip has been attributed to Operations personnel being unable to maintain steam generator levels above the trip setpoints while transferring level control. Steam generator level control at low power is accomplished by viewing steam generator narrow range level, as steam flow and feedwater flow indication are at the very bottom of their range. The increase in feedwater flow when transferring to the main feedwater regulating valves caused the indicated steam generator level to decrease. The decrease in steam generator level and pressurizer level following the reactor trip are anticipated, and are only for a short period of time. The "A" steam generator returned to operable status in two minutes and the pressurizer returned to operable status in four minutes. This event is similar in nature to the event of April 6, 1985, which is described in LER 85-007.

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

ROGER W. KOBER
VICE PRESIDENT
ELECTRIC & STEAM PRODUCTION

TELEPHONE
AREA CODE 716 546-2700

May 7, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Subject: LER 85-008, Automatic Actuation of the Reactor Protection System (RPS)
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a)(2)(iv) which requests a report of, "any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF) including the Reactor Protection System (RPS)," the attached Licensee Event Report LER 85-008 is hereby submitted.

Very truly yours,

Roger W. Kober

Roger W. Kober

RWK/eeg

xc: U.S. Nuclear Regulatory Commission
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