

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 8 8				PAGE (3) 1 OF 0 2		
TITLE (4) RCIC Inboard Steam Supply Valve Isolation.																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 4	1 7	8 5	8 5	0 1 4	0 0	0 5	0 7	8 5					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)														
1		20.402(b)				20.405(c)				<input checked="" type="checkbox"/>		50.73(a)(2)(iv)		73.71(b)		
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)						50.73(a)(2)(v)		73.71(c)		
1 0 0		20.405(a)(1)(ii)				50.36(c)(2)						50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)				50.73(a)(2)(i)						50.73(a)(2)(viii)(A)				
		20.405(a)(1)(iv)				50.73(a)(2)(ii)						50.73(a)(2)(viii)(B)				
		20.405(a)(1)(v)				50.73(a)(2)(iii)						50.73(a)(2)(ix)				
LICENSEE CONTACT FOR THIS LER (12)																
NAME D.J. Gandenberger - Power Production Engineer										TELEPHONE NUMBER						
										AREA CODE						
										7 1 7		5 4 2 - 3 9 1 4				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD						
X	BIN	TISR	2 7 8	Y												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On April 17, 1985, at 2344, the Reactor Core Isolation Cooling (RCIC) system inboard steam supply valve isolated on equipment area high temperature. Investigation determined that temperature indicating switch E51-N600B was malfunctioning. The defective switch was replaced on April 18, 1985, and the isolation signal cleared. The Unit continued 100% power operation during the event.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Susquehanna Steam Electric Station Unit 2	DOCKET NUMBER (2)  0   5   0   0   0   3   8   8   8   5   —   0   1   4   —   0   0   0   2   OF   0   2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On April 17, 1985, at 2344, the inboard steam supply valve for the Reactor Core Isolation Cooling (RCIC) system isolated on equipment area high temperature. The trip setpoint of the temperature indicating switch is 155 degrees F. At the time of the isolation, the temperature indicating switch was reading 80 degrees F. Inspection of the equipment room revealed no abnormalities. I&C personnel determined that temperature indicating switch E51-N600B was malfunctioning. The switch was replaced on April 18, 1985, and the isolation signal cleared. The Unit maintained 100% power operation during the event.



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

May 7, 1985

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 85-014-00  
ER 100450 FILE 841-23  
PLAS- 072

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Docket No. 50-388  
License No. NPF-22

Attached is Licensee Event Report 85-014-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that the Reactor Core Isolation Cooling System inboard steam supply valve isolated due to instrument failure.

H.W. Keiser  
Superintendent of Plant-Susquehanna

DJG/pjg

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